

June 22, 1992

Docket Nos. STN 50-454, STN 50-455
and STN 50-456, STN 50-457

Mr. Thomas J. Kovach
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Dear Mr. Kovach:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M72872, M72873, M72874, AND M72875)

The Commission has issued the enclosed Amendment No. 47 to Facility Operating License No. NPF-37 and Amendment No. 47 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment No. 36 to Facility Operating License No. NPF-72 and Amendment No. 36 to Facility Operating License No. NPF-77 for the Braidwood Station, Unit Nos. 1 and 2, respectively. The amendments are in response to your application dated March 17, 1989, as supplemented on August 25, 1989, March 12, 1990, and June 10, 1991.

The amendments modify Technical Specification (TS) 4.5.2.b venting surveillance requirements by eliminating the requirement of venting the ECCS discharge piping inside of containment. The proposed amendment to TS 4.5.2.b clarifies which ECCS piping vent valve locations require venting by stating that only venting of the ECCS pump casing and discharge piping high points outside of containment will be performed at least once per 31 days. Due to the absence of the high point vents inside containment for Byron Unit 2 and Braidwood Unit 2, the change will only effect the conduct of the surveillance on Byron Unit 1 and Braidwood Unit 1.

A copy of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:
Anthony H. Hsia, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 47 to NPF-37
2. Amendment No. 47 to NPF-66
3. Amendment No. 36 to NPF-72
4. Amendment No. 36 to NPF-77
5. Safety Evaluation
6. Notice of Issuance

cc w/enclosures:
See next page

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Mr. Thomas J. Kovach
Commonwealth Edison Company

cc:

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Joliet, Illinois 60434



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-454

BYRON STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 47
License No. NPF-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated March 17, 1989, as supplemented on August 25, 1989, March 12, 1990, and June 10, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-37 is hereby amended to read as follows:

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PDR

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 47 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 22, 1992



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-455

BYRON STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 47
License No. NPF-66

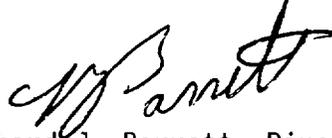
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated March 17, 1989, as supplemented on August 25, 1989, March 12, 1990, and June 10, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 47 and revised by Attachment 2 to NPF-66, and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-37, dated February 14, 1985, are hereby incorporated into this license. Attachment 2 contains a revision to Appendix A which is hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 22, 1992

ATTACHMENT TO LICENSE AMENDMENT NOS. 47 AND 47

FACILITY OPERATING LICENSE NOS. NPF-37 AND NPF-66

DOCKET NOS. STN 50-454 AND STN 50-455

Revise the Appendix A Technical Specifications by removing the page identified below and inserting the attached page. The revised page is identified by the captioned amendment number and contains a marginal line indicating the area of change.

Remove Pages

3/4 5-4

Insert Pages

3/4 5-4

EMERGENCY CORE COOLING SYSTEMS

SURVEILLANCE REQUIREMENTS

4.5.2 Each ECCS subsystem shall be demonstrated OPERABLE:

- a. At least once per 12 hours by verifying that the following valves are in the indicated positions with power to the valve operators removed:

<u>Valve Number</u>	<u>Valve Function</u>	<u>Valve Position</u>
MOV SI8806	Suction to the SI Pumps	Open
MOV SI8835	SI Pump Discharge To RCS Cold Legs	Open*
MOV SI8813	SI Pump Recirculation To The RWST	Open
MOV SI8809A	RHR Pump Discharge to RCS Cold Legs	Open*
MOV SI8809B	RHR Pump Discharge to RCS Cold Legs	Open*
MOV SI8840	RHR Pump Discharge to RCS Hot Legs	Closed
MOV SI8802A	SI Pump Discharge to RCS Hot Legs	Closed
MOV SI8802B	SI Pump Discharge to RCS Hot Legs	Closed

- b. At least once per 31 days by:
- 1) Venting the ECCS pump casings and discharge piping high points outside of containment, and
 - 2) Verifying that each valve (manual, power-operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- c. By a visual inspection which verifies that no loose debris (rags, trash, clothing, etc.) is present in the containment which could be transported to the containment sump and cause restriction of the pump suctions during LOCA conditions. This visual inspection shall be performed:

*Valves may be realigned for testing pursuant to Specification 4.4.6.2.2.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-456

BRAIDWOOD STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36
License No. NPF-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated March 17, 1989, as supplemented on August 25, 1989, March 12, 1990, and June 10, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 36 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 22, 1992



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-457

BRAIDWOOD STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36
License No. NPF-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated March 17, 1989, as supplemented on August 25, 1989, March 12, 1990, and June 10, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 36 and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-72, dated July 2, 1987, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 22, 1992

ATTACHMENT TO LICENSE AMENDMENT NOS. 36 AND 36

FACILITY OPERATING LICENSE NOS. NPF-72 AND NPF-77

DOCKET NOS. STN 50-456 AND STN 50-457

Revise the Appendix A Technical Specifications by removing the page identified below and inserting the attached page. The revised page is identified by the captioned amendment number and contains marginal lines indicating the area of change.

Remove Pages

3/4 5-4

Insert Pages

3/4 5-4

EMERGENCY CORE COOLING SYSTEMS

SURVEILLANCE REQUIREMENTS

4.5.2 Each ECCS subsystem shall be demonstrated OPERABLE:

- a. At least once per 12 hours by verifying that the following valves are in the indicated positions with power to the valve operators removed:

<u>Valve Number</u>	<u>Valve Function</u>	<u>Valve Position</u>
MOV SI8806	Suction to the SI Pumps	Open
MOV SI8835	SI Pump Discharge To RCS Cold Legs	Open*
MOV SI8813	SI Pump Recirculation To The RWST	Open
MOV SI8809A	RHR Pump Discharge to RCS Cold Legs	Open*
MOV SI8809B	RHR Pump Discharge to RCS Cold Legs	Open*
MOV SI8840	RHR Pump Discharge to RCS Hot Legs	Closed
MOV SI8802A	SI Pump Discharge to RCS Hot Legs	Closed
MOV SI8802B	SI Pump Discharge to RCS Hot Legs	Closed

- b. At least once per 31 days by:
- 1) Venting the ECCS pump casings and discharge piping high points outside of containment, and
 - 2) Verifying that each valve (manual, power-operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- c. By a visual inspection which verifies that no loose debris (rags, trash, clothing, etc.) is present in the containment which could be transported to the containment sump and cause restriction of the pump suction during LOCA conditions. This visual inspection shall be performed:

*Valves may be realigned for testing pursuant to Specification 4.4.6.2.2.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 47 TO FACILITY OPERATING LICENSE NO. NPF-37,
AMENDMENT NO. 47 TO FACILITY OPERATING LICENSE NO. NPF-66,
AMENDMENT NO. 36 TO FACILITY OPERATING LICENSE NO. NPF-72,
AND AMENDMENT NO. 36 TO FACILITY OPERATING LICENSE NO. NPF-77
COMMONWEALTH EDISON COMPANY
BYRON STATION, UNIT NOS. 1 AND 2
BRAIDWOOD STATION, UNIT NOS. 1 AND 2
DOCKET NOS. STN 50-454, STN 50-455, STN 50-456 AND STN 50-457

1.0 INTRODUCTION

By letter dated March 17, 1989, as supplemented by letters dated August 25, 1989, March 12, 1990 and June 10, 1991, Commonwealth Edison Company (CECo, the licensee) submitted a Technical Specification (TS) amendment request to discontinue the performance of the venting surveillance required by TS 4.5.2.b for the Emergency Core Cooling System (ECCS) piping inside containment. The change is requested for the purpose of reducing radiation exposure in accordance with as-low-as reasonably achievable (ALARA) guidelines without reducing the safe operation of the ECCS equipment.

2.0 DISCUSSION

CECo's submittal proposes to modify the existing TS 4.5.2.b venting surveillance by eliminating the requirement to vent the ECCS discharge piping vent valve locations by stating that only venting of the ECCS pump casing and the discharge piping high points outside containment for Byron Unit 1 and Braidwood Unit 1 is required. The change will only effect the conduct of the surveillance on Byron Unit 1 and Braidwood Unit 1.

Elimination of the surveillance requirement is anticipated to reduce the annual radiation exposure to site personnel by approximately 0.6 person rem which represents 0.5% of the total non-outage dosage.

3.0 EVALUATION

The purpose of the surveillance is to demonstrate operability of the ECCS by verifying the piping is full. Any trapped air or gas is vented to prevent a water hammer event when the system is actuated. Water hammer was generically addressed and resolved in March 1984 with the publication of NUREG-0927,

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"Evaluation of Water Hammer in Nuclear Power Plants - Technical Findings Relevant to Unresolved Safety Issue (USI) A-1." The staff is satisfied with the resolution, particularly as it relates to pressurized water reactors and the emergency core cooling systems. The proposed request is encompassed by the resolution of USI A-1.

Specific to this submittal, additional water hammer analysis was performed by the licensee. A voided volume was assumed in the residual heat removal discharge piping; however, it should be noted that the probability of this voided volume existing was not considered. The analysis demonstrated that with a completely voided discharge line, the resultant forces on the pipe supports were below design capacities. Also assuming a voided volume of 19.54 ft³, which represents the largest volume between the two high vent points inside containment, the resultant forces on the supports were below the design capacities. Therefore, if air is trapped as a result of not venting the ECCS piping inside containment, the system is capable of withstanding the resulting water hammer event.

However, the Illinois Department of Nuclear Safety (IDNS) had concerns regarding the consequences with the proposed change to the venting surveillance. An analysis performed by IDNS determined the maximum pressure peak as a function of voided pipe volume. The analysis indicated that when a relatively small void volume exists (approaching the zero limit), the peak pressures experienced by the piping during a water hammer event are similar to those caused by the sudden opening of valves, pump startups, etc., and are of no concern. However, the worst case scenario is represented by a voided volume of approximately 12 ft³. At this volume, the peak pressure was calculated to exceed 600 psig, the setpoint of the discharge relief valve. A loss of low head ECCS capability or an intersystem loss of coolant accident may result if the relief valve opens and fails to reseal once the pressure is relieved.

In general, the calculations and analytical methods used in determining the effects of water hammer are uncertain in nature due to computer code limitations. Therefore, the accuracy of the IDNS study was not evaluated by the staff. However, to resolve the concern raised by IDNS, the staff reviewed the likelihood of air intrusion in the piping system and the adequacy of licensee controls to ensure a water filled system. Consideration was given to maintenance practices, operational experience, and procedural controls.

Interviews with operations personnel and review of the surveillance documentation by the licensee concluded that essentially no air had been detected during the surveillance activity. Operational experience since licensing has indicated that air intrusion during normal operation is highly unlikely. Therefore, venting of the ECCS piping outside containment is sufficient to remove trapped air. In addition, the venting procedure requires notification of shift management for further investigation if air is detected during the surveillance.

The refueling water storage tank (RWST) is maintained at an elevation higher than the discharge piping and essentially acts as a keep-fill system. The

RWST level could be below the discharge piping during the refueling mode; however, subsequent lineups and outage activities provide assurance that any air is removed.

Procedures which require a fill and vent after maintenance and prior to the startup of a pump minimize the amount of air introduced to the system during maintenance activities. In addition, the monthly required venting of the piping outside containment would remove trapped air resulting from an inadequate fill and vent operation.

The staff concludes that adequate controls have been implemented and provide assurance that air intrusion is unlikely. Therefore, elimination of the requirement to vent the ECCS piping inside containment does not constitute a safety concern.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact has been prepared and published in the Federal Register on June 16, 1992 (57 FR 26878).

Accordingly, based upon the environmental assessment, the Commission has determined that the issuance of this amendment will not have a significant effect on the quality of the human environment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Ann Marie Bongiovanni

Date: June 22, 1992

UNITED STATES NUCLEAR REGULATORY COMMISSION

COMMONWEALTH EDISON COMPANY

BYRON STATION, UNIT NOS. 1 AND 2

BRAIDWOOD STATION, UNIT NOS. 1 AND 2

NOTICE OF ISSUANCE OF AMENDMENT

FACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 47 to Facility Operating License No. NPF-37, Amendment No. 47 to Facility Operating License No. NPF-66, Amendment No. 36 to Facility Operating License No. NPF-72, and Amendment No. 36 to Facility Operating License No. NPF-77, issued to Commonwealth Edison Company (CECo, the licensee), which revised the Technical Specifications for operation of the Byron Station, Unit Nos. 1 and 2, and Braidwood Station, Unit Nos. 1 and 2, located in Ogle County and Will County, Illinois, respectively. The amendments are effective as of the date of issuance.

The amendments modified the Technical Specifications to eliminate the surveillance requirement of venting the ECCS discharge piping inside the containment. This change will only effect the conduct of the surveillance on Byron Unit 1 and Braidwood Unit 1.

The application for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments.

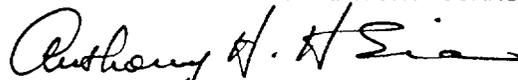
Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER on June 25, 1991 (56 FR 28934). No request for a hearing or petition for leave to intervene was filed following this notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of these amendments will not have a significant effect on the quality of the human environment.

For further details with respect to the action see (1) the application for amendments dated March 17, 1989, as supplemented on August 25, 1989, March 12, 1990, and June 10, 1991, (2) Amendment Nos. 47, 47, 36, 36 to Licensee Nos. NPF-37, NPF-66, NPF-72, NPF-77, respectively, and (3) the Commission's related Safety Evaluation and Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street N.W., Washington, DC and at the local public document rooms located at: for Byron, the Byron Public Library, 109 N. Franklin, P. O. Box 434, Byron, Illinois 61010; for Braidwood, the Wilmington Township Public Library, 201 S. Kankakee Street, Wilmington, Illinois 60481. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Reactor Projects - III/IV/V.

Dated at Rockville, Maryland this 22 day of June 1992.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony H. Hsia, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation