

April 21, 1994

Docket Nos. STN 50-454, STN 50-455
and STN 50-456, STN 50-457

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III, Suite 500
1400 OPUS Place
Downers Grove, Illinois 60515

DISTRIBUTION:	PDIII-2 r/f
NRC & Local PDRs	Docket File
J. Roe	J. Zwolinski
J. Dyer	G. Dick
R. Assa	C. Hawes
OGC	D. Hagan
G. Hill (8)	B. Clayton RIII
C. Grimes	ACRS (10)
OPA	OC/LFDCB
J. Norberg	J. Rajan

Dear Mr. Farrar:

SUBJECT: CORRECTION TO ISSUANCE OF AMENDMENTS (TAC NOS. M87748, M87749,
M87750, AND M87751)

On March 11, 1994, the Commission issued the enclosed Amendment No. 60 to Facility Operating License No. NPF-37 and Amendment No. 60 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment No. 48 to Facility Operating License No. NPF-72 and Amendment No. 48 to Facility Operating License No. NPF-77 for the Braidwood Station, Unit Nos. 1 and 2, respectively.

Four of the amended Technical Specifications (TS) pages were in error. Specifically, TS 4.7.8.b for Byron, Units 1 and 2, did not reference the amendment number. TS 4.7.8.b for Braidwood, Units 1 and 2, did not reference the amendment number and did not include all of the changes proposed by licensing personnel. TS page B 3/4 7-6 for Byron, Units 1 and 2, and TS page 3/4 7-21 for Braidwood, Units 1 and 2, did not have marginal bars indicating areas of change. Please substitute the enclosed replacement pages for the corresponding pages previously issued.

We regret any inconvenience this error may have created.

Sincerely,

Original Signed By:
George F. Dick, Jr., Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Enclosures:
TS pages 3/4 7-20
and 3/4 7-22

cc w/enclosures:
See next page

ENCLOSURE COPY CP-1

OFC	LA:PDIII-2	PM:PDIII-2	PM:PDIII-2	D:PDIII-2	OGC	
NAME	CHAWES <i>CMN</i>	GDICK <i>GD</i>	RASSA <i>RA</i>	JDYER <i>JD</i>		
DATE	4/19/94	4/19/94	4/20/94	4/19/94	1/94	1/94
COPY	(YES/NO)	(YES/NO)	(YES/NO)	(YES/NO)	YES/NO	YES/NO

9404260021 940421
PDR ADDCK 0500454
P PDR

DP01

April 21, 1994

Docket Nos. STN 50-454, STN 50-455
and STN 50-456, STN 50-457

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III, Suite 500
1400 OPUS Place
Downers Grove, Illinois 60515

DISTRIBUTION:	PDIII-2 r/f
NRC & Local PDRs	Docket File
J. Roe	J. Zwolinski
J. Dyer	G. Dick
R. Assa	C. Hawes
OGC	D. Hagan
G. Hill (8)	B. Clayton RIII
C. Grimes	ACRS (10)
OPA	OC/LFDCB
J. Norberg	J. Rajan

Dear Mr. Farrar:

SUBJECT: CORRECTION TO ISSUANCE OF AMENDMENTS (TAC NOS. M87748, M87749, M87750, AND M87751)

On March 11, 1994, the Commission issued the enclosed Amendment No. 60 to Facility Operating License No. NPF-37 and Amendment No. 60 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment No. 48 to Facility Operating License No. NPF-72 and Amendment No. 48 to Facility Operating License No. NPF-77 for the Braidwood Station, Unit Nos. 1 and 2, respectively.

Four of the amended Technical Specifications (TS) pages were in error. Specifically, TS 4.7.8.b for Byron, Units 1 and 2, did not reference the amendment number. TS 4.7.8.b for Braidwood, Units 1 and 2, did not reference the amendment number and did not include all of the changes proposed by licensing personnel. TS page B 3/4 7-6 for Byron, Units 1 and 2, and TS page 3/4 7-21 for Braidwood, Units 1 and 2, did not have marginal bars indicating areas of change. Please substitute the enclosed replacement pages for the corresponding pages previously issued.

We regret any inconvenience this error may have created.

Sincerely,

Original Signed By:
George F. Dick, Jr., Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Enclosures:
TS pages 3/4 7-20
and 3/4 7-22

cc w/enclosures:
See next page

OFC	LA:PDIII-2	PM:PDIII-2	PM:PDIII-2	D:PDIII-2	OGC	
NAME	CHAWES <i>CMN</i>	GDICK <i>GA</i>	RASSA <i>RA</i>	JDYER <i>to</i>		
DATE	4/19/94	4/19/94	4/20/94	4/21/94	1/94	1/94
COPY	(YES/NO)	(YES/NO)	(YES/NO)	(YES/NO)	(YES/NO)	(YES/NO)

Mr. D. L. Farrar
Commonwealth Edison Company

cc:

Mr. William P. Poirier
Westinghouse Electric Corporation
Energy Systems Business Unit
Post Office Box 355, Bay 236 West
Pittsburgh, Pennsylvania 15230

Joseph Gallo, Esquire
Hopkins and Sutter
888 16th Street, N.W., Suite 700
Washington, D.C. 20006

Regional Administrator
U. S. NRC, Region III
801 Warrenville Road
Lisle, Illinois 6013

Ms. Bridget Little Rorem
Applesseed Coordinator
117 North Linden Street
Essex, Illinois 60935

Mr. Edward R. Crass
Nuclear Safeguards and Licensing
Division
Sargent & Lundy Engineers
55 East Monroe Street
Chicago, Illinois 60603

U. S. Nuclear Regulatory Commission
Resident Inspectors Office
Rural Route #1, Box 79
Braceville, Illinois 60407

Mr. Ron Stephens
Illinois Emergency Services
and Disaster Agency
110 East Adams Street
Springfield, Illinois 62706

Howard A. Learner
Environmental Law and Policy
Center of the Midwest
203 North LaSalle Street
Suite 1390
Chicago, Illinois 60601

EIS Review Coordinator
U.S. Environmental Protection Agency
77 W. Jackson Blvd.
Chicago, Illinois 60604-3590

Chairman
Will County Board of Supervisors
Will County Board Courthouse
Joliet, Illinois 60434

Byron/Braidwood Power Stations

U. S. Nuclear Regulatory Commission
Byron/Resident Inspectors Office
4448 North German Church Road
Byron, Illinois 61010-9750

Ms. Lorraine Creek
Rt. 1, Box 182
Manteno, Illinois 60950

Mrs. Phillip B. Johnson
1907 Stratford Lane
Rockford, Illinois 61107

Attorney General
500 South 2nd Street
Springfield, Illinois 62701

Michael Miller, Esquire
Sidley and Austin
One First National Plaza
Chicago, Illinois 60690

George L. Edgar
Newman & Holtzinger, P.C.
1615 L Street, N.W.
Washington, D.C. 20036

Commonwealth Edison Company
Byron Station Manager
4450 North German Church Road
Byron, Illinois 61010

Illinois Dept. of Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704

Commonwealth Edison Company
Braidwood Station Manager
Rt. 1, Box 84
Braceville, Illinois 60407

Chairman, Ogle County Board
Post Office Box 357
Oregon, Illinois 61061

PLANT SYSTEMS

3/4.7.8 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.8 All snubbers shall be OPERABLE. Snubbers excluded from this requirement are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed would have no adverse effect on any safety-related system.

APPLICABILITY: MODES 1, 2, 3, and 4. MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES.

ACTION:

With one or more snubbers inoperable, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.8g. on the attached component or declare the attached system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.8 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Inspection Types

As used in this specification, type of snubber shall mean snubbers of the same design and manufacturer, irrespective of capacity.

b. Visual Inspections

Snubbers are categorized as inaccessible or accessible during reactor operation. Each of these categories (inaccessible and accessible) may be inspected independently according to the schedule determined by Table 4.7-2. The visual inspection interval for each type shall be determined based upon the criteria provided in Table 4.7-2 and the first inspection interval determined using this criteria shall be based upon the previous inspection interval as established by the requirements in effect prior to Amendment No. 60.

9404260030 940421
PDR ADDCK 05000454
P PDR

PLANT SYSTEMS

3/4.7.8 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.8 All snubbers shall be OPERABLE. Snubbers excluded from this requirement are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed would have no adverse effect on any safety-related system.

APPLICABILITY: MODES 1, 2, 3, and 4. MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES.

ACTION:

With one or more snubbers inoperable, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.8g. on the attached component or declare the attached system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.8 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Inspection Types

As used in this specification, type of snubber shall mean snubbers of the same design and manufacturer, irrespective of capacity.

b. Visual Inspections

Snubbers are categorized as inaccessible or accessible during reactor operation. Each of these categories (inaccessible and accessible) may be inspected independently according to the schedule determined by Table 4.7-2. The visual inspection interval for each type of snubber shall be determined based upon the criteria provided in Table 4.7-2 and the first inspection interval determined using this criteria shall be based upon the previous inspection interval as established by the requirements in effect before Amendment No. 48.

PLANT SYSTEMS

BASES

SNUBBERS (Continued)

that the frequency of snubber failures and initiating events is constant with time and that the failure of any snubber on that system could cause the system to be unprotected and to result in failure during an initiating event. USNRC Generic Letter 90-09 "Alternate Requirements for Snubber Visual Inspection Intervals and Corrective Actions" provides information necessary to establish a method of extending or shortening the subsequent visual inspection frequency based upon the failure rates from the previous inspection.

The acceptance criteria are to be used in the visual inspection to determine OPERABILITY of the snubbers. For example, if a fluid port of a hydraulic snubber is found to be uncovered, the snubber shall be declared inoperable and shall not be determined OPERABLE via functional testing.

To provide assurance of snubber functional reliability at least 10% of each type of snubber shall be functionally tested at least once per 18 months with an additional 10% tested for each functional testing failure.

Permanent or other exemptions from the surveillance program for individual snubbers may be granted by the Commission if a justifiable basis for exemption is presented and, if applicable, snubber life destructive testing was performed to qualify the snubber for the applicable design conditions at either the completion of their fabrication or at a subsequent date. Snubbers so exempted shall be listed in the list of individual snubbers indicating the extent of the exemptions.

The service life of a snubber is established via manufacturer input and information through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubbers, seal replaced, spring replaced, in high radiation area, in high temperature area, etc.). The requirement to monitor the snubber service life is included to ensure that the snubbers periodically undergo a performance evaluation in view of their age and operating conditions. These records will provide statistical bases for future consideration of snubber service life.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

c. Visual Inspection Acceptance Criteria

Visual inspections shall verify that: (1) there are no visible indications of damage or impaired OPERABILITY, (2) attachments to the foundation or supporting structure are functional and (3) fasteners for attachment of the snubber to the component and to the snubber anchorage are functional. Snubbers which appear inoperable as a result of visual inspections shall be classified as unacceptable and may be reclassified acceptable for the purpose of establishing the next visual inspection interval, provided that: (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers irrespective of type that may be generically susceptible; and (2) the affected snubber is functionally tested in the as-found condition and determined OPERABLE per Specification 4.7.8f. All snubbers found connected to an inoperable common hydraulic fluid reservoir shall be counted as unacceptable for determining the next inspection interval. A review and evaluation shall be performed and documented to justify continued operation with an unacceptable snubber. If continued operation cannot be justified, the snubber shall be declared inoperable and the ACTION requirements shall be met.

d. Transient Event Inspection

An inspection shall be performed of all snubbers attached to sections of systems that have experienced unexpected, potentially damaging transients as determined from a review of operational data and a visual inspection of the systems within 6 months following such an event. In addition to satisfying the visual inspection acceptance criteria, freedom-of-motion of mechanical snubbers shall be verified using at least one of the following: (1) manually induced snubber movement; or (2) evaluation of in-place snubber piston setting; or (3) stroking the mechanical snubber through its full range of travel.