

October 29, 1986

Docket Nos. STN 50-454
and STN 50-455

Mr. Dennis L. Farrar
Director of Nuclear Licensing
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

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Dear Mr. Farrar:

SUBJECT: AMENDMENT NO. 4 TO BYRON TECHNICAL SPECIFICATIONS

The Commission has issued the enclosed Amendment No. 4 to Facility Operating License NPF-37 for Byron Station, Unit 1. This amendment also applies to the Technical Specifications for Byron Station, Unit 2, although Unit 2 does not have an operating license. Byron Station, Units 1 and 2, have common Technical Specifications (NUREG-1113). This amendment is in response to your applications dated July 30, 1986 and August 5, 1986.

The amendment approves five changes to the Technical Specifications:

1. Replace "0%" with "predicted value" at end of cycle life for determining target flux difference.
2. Delete exact grid plane locations.
3. Delete the maximum total weight of uranium.
4. Remove minimum level, but retain minimum gallons, for the Diesel Fuel Supply System day tank, and
5. Delete "during shutdown" for the 18-month surveillance of seismic monitoring instrumentation.

Notices of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested actions were published in the FEDERAL REGISTER on September 10, 1986 (51 FR 32265) for the August 5, 1986 request and on September 24, 1986 (51 FR 33945) for the July 30, 1986 request. The State of Illinois Department of Nuclear Safety, by telephone call on October 20, 1986, stated that no adverse findings exist as a result of their review of the proposed amendments. No requests for hearing were received.

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A copy of our related Safety Evaluation is also enclosed. This action will appear in the Commission's bi-weekly notice in the FEDERAL REGISTER.

Sincerely,

Leonard N. Olshan, Project Manager
Project Directorate #3
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 4 - NPF-37
2. Related Safety Evaluation

cc: See next page

PD#3
CVogan
10/23/86
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Mr. Dennis L. Farrar
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Byron Station
Units 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-454

BYRON STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 4
License No. NPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Commonwealth Edison Company (the licensee) dated July 30, 1986 and August 5, 1986 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPR-37 is hereby amended to read as follows:

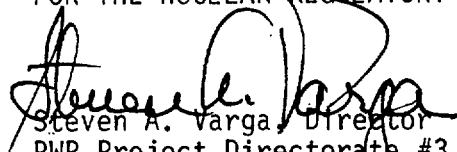
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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 4, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Director
PWR Project Directorate #3
Division of PWR Licensing-A, NRR

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 29, 1986



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT NO. 4

FACILITY OPERATING LICENSE NO. NPR-37

DOCKET NO. STN 50-454

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages (*) have been provided to maintain document completeness.

Remove Pages

Insert Pages

-----	3/4 2-1*
3/4 2-2	3/4 2-2
3/4 2-7	3/4 2-7
-----	3/4 2-8*
-----	3/4 3-43*
3/4 3-44	3/4 3-44
-----	3/4 7-3*
3/4 7-4	3/4 7-4
-----	5-3*
5-4	5-4

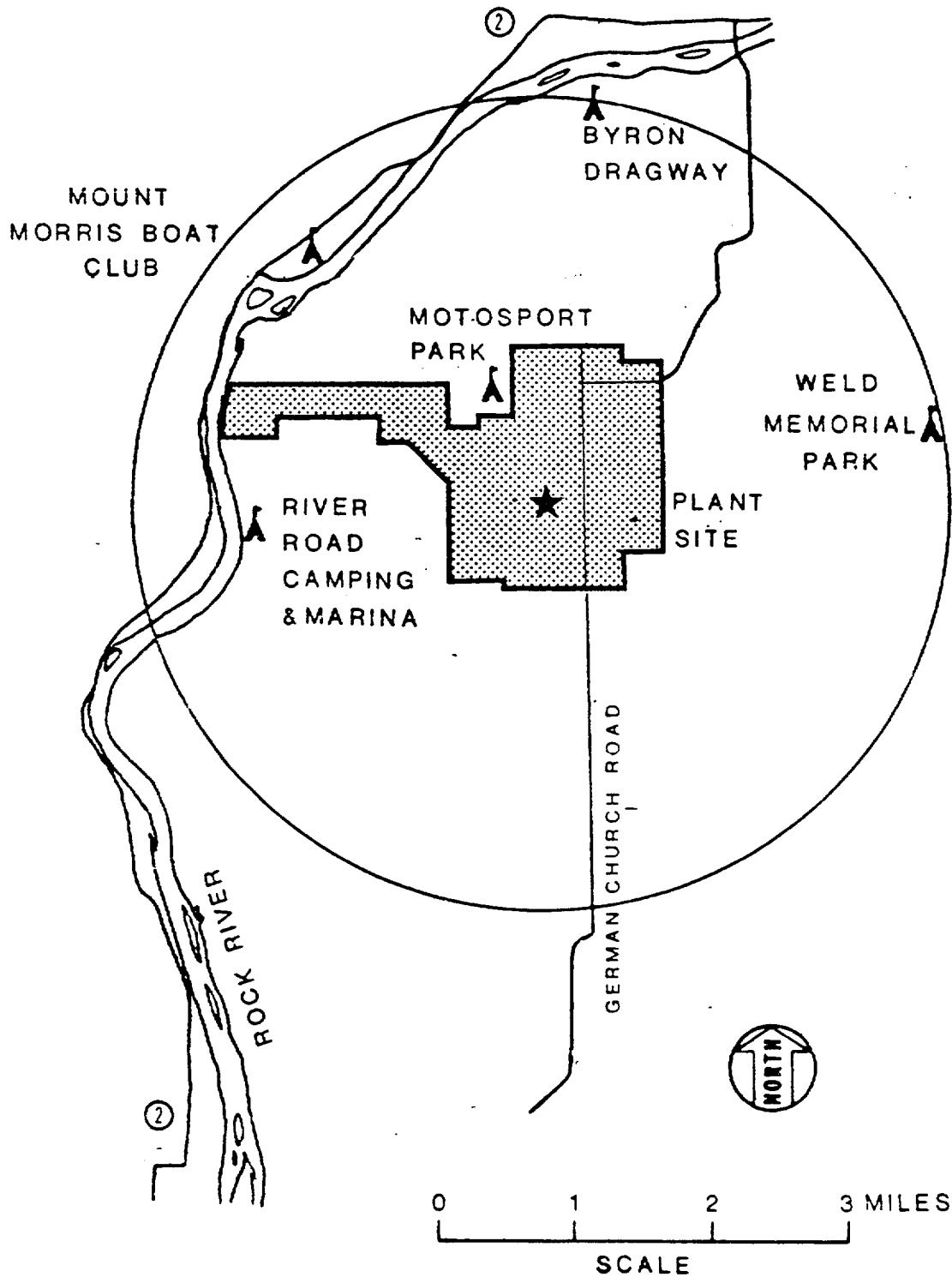


FIGURE 5.1-2

LOW POPULATION ZONE (L.P.Z.)
PUBLIC FACILITIES AND INSTITUTIONS WITHIN 3 MILES OF THE SITE

DESIGN FEATURES

5.3 REACTOR CORE

FUEL ASSEMBLIES

5.3.1 The core shall contain 193 fuel assemblies with each fuel assembly containing 264 fuel rods clad with Zircaloy-4. Each fuel rod shall have a nominal active fuel length of 144 inches. The initial core loading shall have a maximum enrichment of less than 3.20 weight percent U-235. Reload fuel shall be similar in physical design to the initial core loading and shall have a maximum enrichment of 4.0 weight percent U-235.

CONTROL ROD ASSEMBLIES

5.3.2 The core shall contain 53 full-length and no part-length control rod assemblies. The full-length control rod assemblies shall contain a nominal 142 inches of absorber material. All control rods shall be hafnium, clad with stainless steel tubing.

5.4 REACTOR COOLANT SYSTEM

DESIGN PRESSURE AND TEMPERATURE

- 5.4.1 The Reactor Coolant System is designed and shall be maintained:
- In accordance with the Code requirements specified in Section 5.2 of the FSAR, with allowance for normal degradation pursuant to the applicable Surveillance Requirements,
 - For a pressure of 2485 psig, and
 - For a temperature of 650°F, except for the pressurizer which is 680°F.

VOLUME

5.4.2 The total water and steam volume of the Reactor Coolant System is 12,257 cubic feet at a nominal T_{avg} of 588.4°F.

5.5 METEOROLOGICAL TOWER LOCATION

5.5.1 The meteorological tower shall be located as shown on Figure 5.1-1.

PLANT SYSTEMS

AUXILIARY FEEDWATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.2 At least two independent steam generator auxiliary feedwater pumps and associated flow paths shall be OPERABLE with:

- a. One motor-driven auxiliary feedwater pump capable of being powered from an ESF Bus, and
- b. One direct-driven diesel auxiliary feedwater pump capable of being powered from a direct-drive diesel engine and an OPERABLE Diesel Fuel Supply System consisting of a day tank containing a minimum of 420 gallons of fuel.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With one auxiliary feedwater pump inoperable, restore the required auxiliary feedwater pumps to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With both auxiliary feedwater pumps inoperable, be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.

SURVEILLANCE REQUIREMENTS

4.7.1.2.1 Each auxiliary feedwater pump shall be demonstrated OPERABLE:

- a. At least once per 31 days on a STAGGERED TEST BASIS by:
 - 1) Verifying that the pump develops a differential pressure of greater than or equal to 1825 psid at a flow of greater than or equal to 85 gpm on the recirculation flow when tested pursuant to Specification 4.0.5;

TABLE 3.7-2
STEAM LINE SAFETY VALVES PER LOOP

<u>VALVE NUMBER</u>	<u>LIFT SETTING ($\pm 1\%$)*</u>	<u>ORIFICE SIZE</u>
MS013(A-D)	1235 psig	16 in ²
MS014(A-D)	1220 psig	16 in ²
MS015(A-D)	1205 psig	16 in ²
MS016(A-D)	1190 psig	16 in ²
MS017(A-D)	1175 psig	16 in ²

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

INSTRUMENTATION

SEISMIC INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.3 The seismic monitoring instrumentation shown in Table 3.3-7 shall be OPERABLE.

APPLICABILITY: At all times.

ACTION:

- a. With one or more of the above required seismic monitoring instruments inoperable for more than 30 days, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 10 days outlining the cause of the malfunction and the plans for restoring the instrument(s) to OPERABLE status.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.3.1 The seismic monitoring instrumentation shall be determined OPERABLE:

- a. At least once per 31 days by verifying operable status indications of the seismic monitoring instrumentation.
- b. At least once per 92 days by verifying that:
 - 1) The triaxial acceleration sensors and the time-history accelerographs properly process the equipment internal test signals.
 - 2) The response spectrum analyzer properly executes its diagnostic routine.
- c. At least once per 184 days by verifying that the triaxial acceleration sensors and the time-history accelerographs properly record the equipment internal test signals. The test may be performed in lieu of the test required by Specification 4.3.3.3.1.b.1), and
- d. At least once per 18 months by:
 - 1) Verifying the electronic calibration of the time-history accelerographs.
 - 2) Installing fresh magnetic recording plates in the triaxial peak accelerographs.

4.3.3.3.2 Upon actuation of the seismic monitoring instruments, the equipment listed in Table 3.3-7 shall be restored to OPERABLE status within 24 hours following the seismic event. Data shall be retrieved from actuated instruments and analyzed to determine the magnitude of the vibratory ground motion. A Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2 within 14 days describing the magnitude, frequency spectrum and resultant effect upon facility features important to safety.

INSTRUMENTATION

MOVABLE INCORE DETECTORS

LIMITING CONDITION FOR OPERATION

- 3.3.3.2 The Movable Incore Detection System shall be OPERABLE with:
- At least 75% of the detector thimbles,
 - A minimum of two detector thimbles per core quadrant, and
 - Sufficient movable detectors, drive, and readout equipment to map these thimbles.

APPLICABILITY: When the Movable Incore Detection System is used for:

- Recalibration of the excore neutron flux detection system, or
- Monitoring the QUADRANT POWER TILT RATIO, or
- Measurement of $F_{\Delta H}^N$, $F_Q(Z)$ and F_{xy} .

ACTION:

With the Movable Incore Detection System inoperable, do not use the system for the above applicable monitoring or calibration functions. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.3.3.2 The Movable Incore Detection System shall be demonstrated OPERABLE at least once per 24 hours by normalizing each detector output when required for:
- Recalibration of the Excore Neutron Flux Detection System, or
 - Monitoring the QUADRANT POWER TILT RATIO, or
 - Measurement of $F_{\Delta H}^N$, $F_Q(Z)$, and F_{xy} .

POWER DISTRIBUTION LIMITS

SURVEILLANCE REQUIREMENTS (Continued)

- 2) When the F_{xy}^C is less than or equal to the F_{xy}^{RTP} limit for the appropriate measured core plane, additional power distribution maps shall be taken and F_{xy}^C compared to F_{xy}^{RTP} and F_{xy}^L at least once per 31 EFPD.
- e. The F_{xy} limits for RATED THERMAL POWER (F_{xy}^{RTP}) shall be 1.71 for all core planes containing Bank "D" control rods and 1.55 for all unrodded core planes;
- f. The F_{xy} limits of Specification 4.2.2.2e., above, are not applicable in the following core planes regions as measured in percent of core height from the bottom of the fuel:
 - 1) Lower core region from 0 to 15%, inclusive,
 - 2) Upper core region from 85 to 100%, inclusive,
 - 3) Within $\pm 2\%$ of grid plane regions such that no more than 20% of the total core height in the center core region is affected, and
 - 4) Core plane regions within $\pm 2\%$ of core height (± 2.88 inches) about the bank demand position of the Bank "D" control rods.
- g. With F_{xy}^C exceeding F_{xy}^L , the effects of F_{xy} on $F_Q(Z)$ shall be evaluated to determine if $F_Q(Z)$ is within its limits.

4.2.2.3 When $F_Q(Z)$ is measured for other than F_{xy} determinations, an overall measured $F_Q(Z)$ shall be obtained from a power distribution map and increased by 3% to account for manufacturing tolerances and further increased by 5% to account for measurement uncertainty.

POWER DISTRIBUTION LIMITS

3/4.2.3 RCS FLOW RATE AND NUCLEAR ENTHALPY RISE HOT CHANNEL FACTOR

LIMITING CONDITION FOR OPERATION

3.2.3 Indicated Reactor Coolant System (RCS) total flow rate and $F_{\Delta H}^N$ shall be maintained as follows for four loop operation.

- a. RCS Total Flowrate \geq 390,400 gpm, and
- b. $F_{\Delta H}^N \leq 1.55 [1.0 + 0.3 (1.0-P)]$

where:

Measured values of $F_{\Delta H}^N$ are obtained by using the movable incore detectors. An appropriate uncertainty of 4% (nominal) or greater shall then be applied to the measured value of $F_{\Delta H}^N$ before it is compared to the requirements, and

$$P = \frac{\text{THERMAL POWER}}{\text{RATED THERMAL POWER}}$$

APPLICABILITY: MODE 1.

ACTION:

With RCS total flow rate or $F_{\Delta H}^N$ outside the region of acceptable operation:

- a. Within 2 hours either:
 1. Restore RCS total flow rate and $F_{\Delta H}^N$ to within the above limits, or
 2. Reduce THERMAL POWER to less than 50% of RATED THERMAL POWER and reduce the Power Range Neutron Flux-High Trip Setpoint to less than or equal to 55% of RATED THERMAL POWER within the next 4 hours.

LIMITING CONDITION FOR OPERATION

ACTION (Continued)

- c. With the indicated AFD outside of the above required target band for more than 1 hour of cumulative penalty deviation time during the previous 24 hours and with THERMAL POWER less than 50% but greater than 15% of RATED THERMAL POWER, the THERMAL POWER shall not be increased equal to or greater than 50% of RATED THERMAL POWER until the indicated AFD is within the above required target band.

SURVEILLANCE REQUIREMENTS

4.2.1.1 The indicated AFD shall be determined to be within its limits during POWER OPERATION above 15% of RATED THERMAL POWER by:

- a. Monitoring the indicated AFD for each OPERABLE excore channel:
 - 1) At least once per 7 days when the AFD Monitor Alarm is OPERABLE, and
 - 2) At least once per hour for the first 24 hours after restoring the AFD Monitor Alarm to OPERABLE status.
- b. Monitoring and logging the indicated AFD for each OPERABLE excore channel at least once per hour for the first 24 hours and at least once per 30 minutes thereafter, when the AFD Monitor Alarm is inoperable. The logged values of the indicated AFD shall be assumed to exist during the interval preceding each logging.

4.2.1.2 The indicated AFD shall be considered outside of its target band when two or more OPERABLE excore channels are indicating the AFD to be outside the target band. Penalty deviation outside of the above required target band shall be accumulated on a time basis of:

- a. One minute penalty deviation for each 1 minute of POWER OPERATION outside of the target band at THERMAL POWER levels equal to or above 50% of RATED THERMAL POWER, and
- b. One-half minute penalty deviation for each 1 minute of POWER OPERATION outside of the target band at THERMAL POWER levels between 15% and 50% of RATED THERMAL POWER.

4.2.1.3 The target flux difference of each OPERABLE excore channel shall be determined by measurement at least once per 92 Effective Full Power Days. The provisions of Specification 4.0.4 are not applicable.

4.2.1.4 The target flux difference shall be updated at least once per 31 Effective Full Power Days by either determining the target flux difference pursuant to Specification 4.2.1.3 above or by linear interpolation between the most recently measured value and the predicted value at the end of the cycle life. The provisions of Specification 4.0.4 are not applicable.

3/4.2 POWER DISTRIBUTION LIMITS

3/4.2.1 AXIAL FLUX DIFFERENCE

LIMITING CONDITION FOR OPERATION

3.2.1 The indicated AXIAL FLUX DIFFERENCE (AFD) shall be maintained within the following target band (flux difference units) about the target flux difference:

- a. $\pm 5\%$ for Cycle 1 core average accumulated burnup of less than or equal to 5000 MWD/MTU, and
- b. $+ 3\%$, -9% for Cycle 1 core average accumulated burnup of greater than 5000 MWD/MTU, and
- c. $+ 3\%$, -12% for each subsequent cycle.

The indicated AFD may deviate outside the above required target band at greater than or equal to 50% but less than 90% of RATED THERMAL POWER provided the indicated AFD is within the Acceptable Operation Limits of Figure 3.2-1 and the cumulative penalty deviation time does not exceed 1 hour during the previous 24 hours.

The indicated AFD may deviate outside the above required target band at greater than 15% but less than 50% of RATED THERMAL POWER provided the cumulative penalty deviation time does not exceed 1 hour during the previous 24 hours.

APPLICABILITY: MODE 1 above 15% of RATED THERMAL POWER*.

ACTION:

- a. With the indicated AFD outside of the above required target band and with THERMAL POWER greater than or equal to 90% of RATED THERMAL POWER, within 15 minutes, either:
 1. Restore the indicated AFD to within the above required target band limits, or
 2. Reduce THERMAL POWER to less than 90% of RATED THERMAL POWER.
- b. With the indicated AFD outside of the above required target band for more than 1 hour of cumulative penalty deviation time during the previous 24 hours or outside the Acceptable Operation Limits of Figure 3.2-1 and with THERMAL POWER less than 90% but equal to or greater than 50% of RATED THERMAL POWER, reduce:
 1. THERMAL POWER to less than 50% of RATED THERMAL POWER within 30 minutes, and
 2. The Power Range Neutron Flux - High[#] Setpoints to less than or equal to 55% of RATED THERMAL POWER within the next 4 hours.

*See Special Test Exceptions Specification 3.10.2.

[#]Surveillance testing of the Power Range Neutron Flux channel may be performed pursuant to Specification 4.3.1.1 provided the indicated AFD is maintained within the Acceptable Operation Limits of Figure 3.2-1. A total of 16 hours operation may be accumulated with the AFD outside of the above required target band during testing without penalty deviation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 4 TO FACILITY OPERATING LICENSE NO. NPF-37

COMMONWEALTH EDISON COMPANY

BYRON STATION, UNIT NO. 1

DOCKET NO. STN 50-454

1.0 INTRODUCTION

By applications dated July 30, 1986 and August 5, 1986, Commonwealth Edison Company (the licensee) request amendments to the Technical Specifications for Byron Station, Units 1 and 2. The July 30, 1986 letter requested three changes: (1) replace "0%" with "predicted value" at end of cycle life for determining target flux difference, page 3/4 2-2; (2) delete exact grid plane locations, page 3/4 2-7; and (3) delete the maximum total weight of uranium, page 5-4. The August 5, 1986 letter requested two changes: (1) remove minimum level, but retain minimum gallons, for the Diesel Fuel Supply day tank, page 3/4 7-4; and (2) delete "during shutdown" for the 18-month surveillance of seismic monitoring instrumentation, page 3/4 3-44.

2.0 DISCUSSION AND EVALUATION

2.1 Target Flux Difference

Since the target flux difference at the end of cycle is a predicted value which is not necessarily zero and may vary from cycle to cycle, the proposed change will result in more accurate interpolation and is, therefore, acceptable.

2.2 Grid Plane Locations

Core reloads may utilize two regions of fuel, one with a reduced rod bow and the other with standard rod bow design. The grid plane locations are slightly different for these regions and thus deleting the exact grid plane locations from the Technical Specifications will accommodate these reload specific differences. The staff was concerned, however, that this might allow more of the core to be exempt from meeting the F_{xy} limits. The licensee proposed adding the phrase "such that no more than 20% of the total core height in the central core region is affected" to eliminate this problem. We have reviewed this change with this addition and find it acceptable.

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2.3 Weight of Uranium

This change is similar to maximum fuel weight changes recently made on other units (Catawba Units 1 and 2, Trojan and Farley Units 1 and 2). Due to variations in the fuel manufacturing process and a new fuel pellet design, a slight difference in the maximum uranium weight per fuel rod has arisen. This may continue to arise in future reloads. The proposed change is sought to account for differences in fuel between the two units and from cycle to cycle. The maximum fuel rod weight is not used as a direct input to any safety analysis and no safety analyses are affected by the deletion of this value. Thus, we find this change acceptable.

2.4 Day Tank Level

This change deletes reference to "level of 71%" but retains 420 gallons as the minimum amount of fuel in the day tank for the Diesel Fuel Supply System. The licensee proposed this change to accommodate differences in the instruments and tank dimensions for Units 1 and 2.

Using only gallons in the Limiting Condition for Operation is consistent with the Westinghouse Standard Technical Specifications for similar tanks. Furthermore, the amount of fuel required is not being changed and is still consistent with the accident analyses. Therefore, the staff finds the change acceptable.

2.5 Delete "during shutdown"

The original Technical Specifications required that a certain surveillance of seismic monitoring instrumentation be performed at least once per 18-months, during shutdown. "During shutdown" was included because it was thought that doing this surveillance during shutdown would reduce occupational exposure. However, although the seismic instrumentation is located inside containment, the surveillance tests are performed outside containment. Thus, occupational exposure is not a consideration.

Furthermore, some of this instrumentation is common to both units and both units are usually not shutdown at the same time. Thus, it is not practical to perform these surveillances during shutdown.

Therefore, the staff finds that deletion of the words "during shutdown" is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

The staff has determined that the amendment involves no increase in the amounts, and no change in the types, of any effluents that may be released offsite and that there is no increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding

that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 PRINCIPAL CONTRIBUTORS:

M. Chatterton
L. Olshan

Dated: October 29, 1986