Docket File

Docket No.: STN 50-454

JAN 28 1985

Mr. Dennis L. Farrar
Director of Nuclear Licensing
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

The Commission has issued the enclosed Amendment No. 1 to Facility Operating License No. NPF-23 for Byron Station, Unit 1. The amendment consists of the addition of a footnote to Table 3.6-1, Containment Isolation Valves, that allows certain valves to be opened on an intermittent basis under administrative control. The amendment is in response to your application dated January 18, 1985, telecopied to the NRC on that date. The amendment was approved by telephone call from the Assistant Director for Licensing on January 18, 1985.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance and Final Determination of No Significant Hazards Consideration and Opportunity for Hearing will be included in the Commission's Monthly Notice.

Sincerely,

/s/

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Enclosures:
1. Amendment No. 1 to NPF-23
2. Safety Evaluation

cc: See next page

CONCURRENCES:

DL:LB#1	OELD	DL:LB#1
LOlshan:es		BJYoungblood
1/24/85	1/24/85	1/27/85

No legal objection

Notified by OELD for picking on 1/28/85

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-454

BYRON STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 1
License No. NPF-23

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Commonwealth Edison Company (the licensee) dated January 18, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I:

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-23 is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the attachment to this license amendment:

   Revise paragraph 2.C.(2) to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 1, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment was effective January 18, 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

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B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: JAN 28 1985

CONCURRENCES:

DL:LB#1	OELD	DL:LB#1
LOlshan:es	per Oscar Silver	BJYoungblood
1/24/85	1/28/85	1/24/85

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 1, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment was effective January 18, 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 28, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 1

FACILITY OPERATING LICENSE NO. NPF-23

DOCKET NO. STN 50-454

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Pages

3/4 6-21

3/4 6-22

3/4 6-23

3/4 6-24

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

PENETRATION	VALVE NO.	FUNCTION	MAXIMUM ISOLATION TIME (SEC)
6. Main Steam Isolation (Continued)			
85	1MS101B*	Main Steam	10.0
86	1MS101C*	Main Steam	10.0
7. Feedwater Isolation			
76	1FW009D*	Main Feedwater	5.0
76	1FW043D*	Main Feedwater	6.0
79	1FW009A*	Main Feedwater	5.0
79	1FW043A*	Main Feedwater	6.0
84	1FW009B*	Main Feedwater	5.0
84	1FW043B*	Main Feedwater	6.0
87	1FW009C*	Main Feedwater	5.0
87	1FW043C*	Main Feedwater	6.0
99	1FW035D*	Main Feedwater	6.0
99	1FW039D*	Main Feedwater	6.0
100	1FW035A*	Main Feedwater	6.0
100	1FW039A*	Main Feedwater	6.0
101	1FW035B*	Main Feedwater	6.0
101	1FW039B*	Main Feedwater	6.0
102	1FW035C*	Main Feedwater	6.0
102	1FW039C*	Main Feedwater	6.0
8. Remote Manual			
68	1RH8701A*,#	RH Suction	N/A
68	1RH8701B*,#	RH Suction	N/A
75	1RH8702A*,#	RH Suction	N/A
75	1RH8702B*,#	RH Suction	N/A
59	1SI8881*	Hot Leg Safety Injection	N/A
73	1SI8824*	Hot Leg Safety Injection	N/A
66	1SI8825*	Hot Leg RH Injection	N/A
60	1SI8823*	Cold Leg Safety Injection	N/A
50	1SI8890A*	Cold Leg RH Injection	N/A
51	1SI8890B*	Cold Leg RH Injection	N/A
26	1SI8843*	Cold Leg Safety Injection	N/A
33	1CV8355A*	RCP Seal Injection	N/A
33	1CV8355D*	RCP Seal Injection	N/A
53	1CV8355B*	RCP Seal Injection	N/A
53	1CV8355C*	RCP Seal Injection	N/A

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

PENETRATION	VALVE NO.	FUNCTION	MAXIMUM ISOLATION TIME (SEC)
8. Remote Manual (Continued)			
59	1SI8802A*	Hot Leg Safety Injection	N/A
73	1SI8802B*	Hot Leg Safety Injection	N/A
60	1SI8835*	Hot Leg Safety Injection	N/A
50	1SI8809A*	RH Cold Leg Injection	N/A
51	1SI8809B*	RH Cold Leg Injection	N/A
66	1SI8840*	Hot Leg Safety Injection	N/A
100	1AF013A*	Feedwater	N/A
100	1AF013E*	Feedwater	N/A
101	1AF013B*	Feedwater	N/A
101	1AF013F*	Feedwater	N/A
102	1AF013C*	Feedwater	N/A
102	1AF013G*	Feedwater	N/A
99	1AF013D*	Feedwater	N/A
99	1AF013H*	Feedwater	N/A
9. Manual			
37	1CV8346*	RCS Loop Fill	N/A
I3	1VQ016	Instrument Penetration	N/A
I3	1VQ017	Instrument Penetration	N/A
I3	1VQ018	Instrument Penetration	N/A
I3	1VQ019	Instrument Penetration	N/A
I5	1RY075	Instrument Penetration	N/A
30	1WM190	Make-Up Demin	N/A
57	1FC009	Spent Fuel Pool Cleaning	N/A
57	1FC010	Spent Fuel Pool Cleaning	N/A
32	1FC011	Spent Fuel Pool Cleaning	N/A
32	1FC012	Spent Fuel Pool Cleaning	N/A
77	1MS021D*,#	Main Steam	N/A
78	1MS021A*,#	Main Steam	N/A
85	1MS021B*,#	Main Steam	N/A
86	1MS021C*,#	Main Steam	N/A
AL	1PR002E	Process Radiation	N/A
AL	1PR033A	Process Radiation	N/A
AL	1PR033B	Process Radiation	N/A
AL	1PR002F	Process Radiation	N/A
AL	1PR033C	Process Radiation	N/A
AL	1PR033D	Process Radiation	N/A

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

PENETRATION	VALVE NO.	FUNCTION	MAXIMUM ISOLATION TIME (SEC)
9. Manual (Continued)			
99	1FW015D*,#	Feedwater	N/A
100	1FW015A*,#	Feedwater	N/A
101	1FW015B*,#	Feedwater	N/A
102	1FW015C*,#	Feedwater	N/A
10. Check			
28	1CV8113	RCP Seal Water Return	N/A
37	1CV8348*	RCS Loop Fill	N/A
6	1WØ007A	Chilled Water	N/A
10	1WØ007B	Chilled Water	N/A
21	1CC9534	RCP Mtr Brng Return	N/A
24	1CC9518	RCP Thermal Barrier Return	N/A
25	1CC9486	RCP Cooling Wtr Supply	N/A
1	1CS008A	Containment Spray	N/A
16	1CS008B	Containment Spray	N/A
39	1IA091	Instrument Air	N/A
30	1WM191	Make-Up Demin	N/A
52	1PR032	Process Radiation	N/A
AL	1PR002G	Process Radiation	N/A
AL	1PR002H	Process Radiation	N/A
12	1PS231A	Hydrogen Monitor	N/A
12	1PS231B	Hydrogen Monitor	N/A
27	1RY8047	PRT Nitrogen	N/A
44	1RY8046	PRT Make-Up	N/A
26	1SI8815*	Safety Injection	N/A
50	1SI8818A*	Safety Injection	N/A
50	1SI8818D*	Safety Injection	N/A
51	1SI8818B*	Safety Injection	N/A
51	1SI8818C*	Safety Injection	N/A
59	1SI8905A*	Safety Injection	N/A
59	1SI8805D*	Safety Injection	N/A
60	1SI8819A*	Safety Injection	N/A
60	1SI8819B*	Safety Injection	N/A

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

PENETRATION	VALVE NO.	FUNCTION	MAXIMUM ISOLATION TIME (SEC)
10. Check (Continued)			
60	1SI8819C*	Safety Injection	N/A
60	1SI8819D*	Safety Injection	N/A
66	1SI8841A*	Safety Injection	N/A
66	1SI8841B*	Safety Injection	N/A
73	1SI8905B*	Safety Injection	N/A
73	1SI8905C*	Safety Injection	N/A
55	1SI8968*	Safety Injection	N/A
34	1FP345*	Fire Protection	N/A
33	1CV8368A*	RCP Seal Injection	N/A
33	1CV8368D*	RCP Seal Injection	N/A
53	1CV8368B*	RCP Seal Injection	N/A
53	1CV8368C*	RCP Seal Injection	N/A
11. S/G Safeties/PORVs			
77	1MS013D*	Main Steam	N/A
77	1MS014D*	Main Steam	N/A
77	1MS015D*	Main Steam	N/A
77	1MS016D*	Main Steam	N/A
77	1MS017D*	Main Steam	N/A
78	1MS013A*	Main Steam	N/A
78	1MS014A*	Main Steam	N/A
78	1MS015A*	Main Steam	N/A
78	1MS016A*	Main Steam	N/A
78	1MS017A*	Main Steam	N/A
85	1MS013B*	Main Steam	N/A
85	1MS014B*	Main Steam	N/A
85	1MS015B*	Main Steam	N/A
85	1MS016B*	Main Steam	N/A
85	1MS017B*	Main Steam	N/A
86	1MS013C*	Main Steam	N/A
86	1MS014C*	Main Steam	N/A
86	1MS015C*	Main Steam	N/A
86	1MS016C*	Main Steam	N/A
86	1MS017C*	Main Steam	N/A
77	1MS018D*	Main Steam	20
78	1MS018A*	Main Steam	20
85	1MS018B*	Main Steam	20
86	1MS018C*	Main Steam	20

*Not subject to Type C leakage tests.
**Proper valve operation will be demonstrated by verifying that the valve strokes to its required position.
#May be opened on an intermittent basis under administrative control.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 1 TO FACILITY OPERATING LICENSE NPF-23

COMMONWEALTH EDISON COMPANY

BYRON STATION, UNIT 1

DOCKET NO. STN 50-454

Introduction

By letter dated January 18, 1985 Commonwealth Edison Company (the licensee) proposed an amendment to the Technical Specifications (TSs) appended to Facility Operating License No. NPF-23 for Byron Station, Unit 1. The proposed amendment was telecopied to the NRC on January 18, 1985 and verbally approved by the Assistant Director for Licensing later that day. The amendment adds a footnote to Table 3.6-1, Containment Isolation Valves, that allows certain valves to be opened on an intermittent basis under administrative control.

Background and Evaluation of Emergency Circumstances

The Westinghouse Standard Technical Specifications (NUREG-0452) in the table of containment isolation valves contains a footnote that allows certain valves to be opened intermittently under administrative control. The footnote was inadvertently omitted from the Byron TSs.

On January 18, 1985, the licensee was going to control secondary water system chemistry by injecting hydrazine through valves IFW015A, B, C and D. The plant was in Mode 3 which requires that containment integrity be maintained; with the omission of the aforementioned footnote, these four valves could not be opened. Proper secondary water system chemistry is needed to limit the amount of corrosion in the steam generators. In order to open these valves under the TSs that existed, the licensee could have cooled the plant down to Mode 5 because containment integrity is not required in Mode 5. Instead, the licensee submitted the proposed amendment. Eight other valves (1RH8701A and B, 1RH8702A and B, and 1MS021A, B, C and D) were also included since they might also have to be opened in the near future.

Considering the potential corrosion of the steam generators, the delay that might have been incurred and the fact that the footnote was inadvertently omitted in the original TSs, emergency action was taken to approve the application. Therefore, although the State of Illinois was informed by telephone of the action prior to NRC approval, no prior notice of the action was published.

Discussion and Evaluation

The licensee proposed to change TS Table 3.6-1, Containment Isolation Valves, to add a footnote to twelve valves, which would state that these valves "may be opened on an intermittent basis under administrative control." These containment isolation valves are required by the current Technical Specifications to be closed during Modes 1 through 4. TS 3.6.1.1, Containment Integrity, requires containment integrity to be maintained during Modes 1 through 4, and TS 1.7, which reflects GDC 55, 56, and 57, defines containment integrity as existing in part, when all penetrations required to be closed during accident conditions are either: (1) capable of being closed by operable automatic containment isolation valves, or (2) closed by manual valves, blind flanges or deactivated automatic valves secured in their closed positions, except as provided in TS Table 3.6-1. Thus, the proposed footnote would permit the twelve valves to be opened on an intermittent basis under administrative control, during Modes 1 through 4. This is necessary for proper operation of the plant, and the Standard Technical Specifications (NUREG-0452) contain this footnote. The valves in question are:

1FW015A, B, C, D - chemical feedlines to steam generators

1M5021A, B, C, D - drain valves in the main steamlines

1RH8701A, B - RHR suction lines from the RCS
1RH8702A, B

The chemical feedlines are required to be opened to maintain steam generator chemistry within required limits in Modes 3 and 4 (hot standby and hot shutdown). The main steam line drain valves must be periodically opened to drain the main steam system. The RHR suction lines must be opened during reactor cooldown so that the plant may be shutdown. Thus, it is necessary and acceptable that the subject valves be permitted to be opened on an intermittent basis, under administrative control, and the staff finds the licensee's proposed TS change to be acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5 (d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal, need not be prepared in connection with issuance of this amendment.

Final No Significant Hazards Consideration Determination

The State was informed by telephone on January 18, 1985 of our proposed no significant hazards consideration and had no comments. Based on our review of the licensee's submittal as described in our above evaluation and for the reasons stated below, we have made a final determination that the licensee's amendment request does not involve a significant hazards consideration.

The Commission has provided guidance for the application of the criteria in 10 CFR 50.92 by providing examples of amendments that are considered not likely to involve significant hazards considerations (48 FR 14870); example (i) lists correction of an error. The omission of the footnote in the Byron TSs was clearly an error: the footnote is included in the standard TSs and reference to the exceptions allowed by the footnote are contained in the Byron TS Section 1.7.a.2). Therefore, the Commission has determined that the application does not involve a significant hazards consideration.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 28, 1985

The following NRC personnel have contributed to this Safety Evaluation:
J. Pulsipher
L. Olshan