

March 22, 2002

MEMORANDUM TO: Cynthia Carpenter, Program Director
Policy and Rulemaking Program
Division of Regulatory Improvement Programs, NRR

FROM: Peter C. Wen, Project Manager */RA/*
Policy and Rulemaking Program
Division of Regulatory Improvement Programs, NRR

SUBJECT: SUMMARY OF FEBRUARY 21, 2002, MEETING WITH NUCLEAR
ENERGY INSTITUTE ON STEAM GENERATOR TUBE DATABASE

On February 21, 2002, the Nuclear Regulatory Commission (NRC) staff met with representatives of the Nuclear Energy Institute (NEI) and the nuclear industry at the NRC's office in Rockville, Maryland to discuss issues related to the database used for assessing the significance of steam generator tube degradation at the tube support plate intersections as discussed in Generic Letter 95-05. The meeting attendees are listed in Attachment 1.

By letter dated September 17, 2001, NEI submitted Addendum 4 to the Steam Generator Degradation Specific Management Database. This addendum contains the most recent update of the databases for assessing the significance of outside diameter stress corrosion cracking (ODSCC) at the tube support plate elevations as discussed in Generic Letter 95-05. It also provides a new data exclusion criteria for NRC review and approval. The net effect of this new exclusion criteria is to remove the pulled tube data from France from the 7/8-inch diameter databases.

In early December, the NRC staff was informed by the licensee for Beaver Valley of the preliminary destructive examination results from a tube pull performed by the licensee during the fall. The preliminary evaluation of the results from the Beaver Valley testing indicated that the leakage database for 7/8-inch diameter tubes may no longer satisfy the criterion for demonstrating a correlation exists between voltage and leakage (i.e., leak rate is independent of the voltage). In a subsequent phone call in mid-December, the staff was informed by various industry representatives that the review of the new data exclusion criteria should be considered a high priority in its review of the Addendum 4 databases.

Based on a preliminary review of the new data exclusion criteria, the NRC provided questions and comments on the industry's proposal by letter dated January 3, 2002 (ADAMS Accession No. ML020090708). The purpose of the meeting was to discuss the responses to these questions and to discuss proposals for modifying the methodology/database used in assessing leakage under postulated accident conditions.

Louise Lund of the NRC started the meeting with an introduction. Jim Riley of NEI described the nuclear utilities' perspective and outlined the industry's plan for adding the Beaver Valley data point. He indicated that by April 2, 2002, the industry will issue a revision to the database which would include the Beaver Valley Unit 1 data and issue a report to document the refined analytical approach. His presentation slides are provided as Attachment 2.

Bob Keating of Westinghouse gave a presentation on "ODSCC ARC Leak Rate Correlation & Simulation by Monte Carlo Methods." His slides are provided as Attachment 3. Tom Pitterle of Westinghouse gave presentations on "Merging of 3/4" and 7/8" Steam-Line Break Leak Rate Data," and the industry's response to the questions and comments posed in the January 3, 2002 NRC letter. His presentation slides are provided as Attachments 4 and 5.

At the end of the meeting, the staff agreed to provide feedback within one week of the meeting regarding whether the changes to the methodology required NRC staff review and approval. Representatives of the NRC and the industry agreed that this meeting had been useful for the exchange of information on the discussed topics. Subsequent to the meeting, the NRC staff indicated to NEI that the proposals would require prior NRC review and approval in accordance with Generic Letter 95-05.

Project No. 689

Attachments: As stated

cc w/attn: See list

**Attendees for February 21, 2002
Meeting on ODSCC Database**

NAME	ORGANIZATION
L. Lund	NRC/NRR/DE/EMCB
K. Karwoski	NRC/NRR/DE/EMCB
E. Murphy	NRC/NRR/DE/EMCB
J. Tsao	NRC/NRR/DE/EMCB
J. Davis	NRC/NRR/DE/EMCB
J. Birmingham	NRC/NRR/DRIP/RPRP
J. Riley	NEI
H. Cothron	TVA
J. Arhar	PG&E
R. Keating	Westinghouse
T. Pitterle	Westinghouse
G. Elder	Westinghouse
G. Srikantiah	EPRI
G. Kammerdeiner	FENOC
B. Mays	TXU.
P. Heasler	PNNL

Bob Keating of Westinghouse gave a presentation on "ODSCC ARC Leak Rate Correlation & Simulation by Monte Carlo Methods." His slides are provided as Attachment 3. Tom Pitterle of Westinghouse gave presentations on "Merging of 3/4" and 7/8" Steam-Line Break Leak Rate Data," and the industry's response to the questions and comments posed in the January 3, 2002 NRC letter. His presentation slides are provided as Attachments 4 and 5.

At the end of the meeting, the staff agreed to provide feedback within one week of the meeting regarding whether the changes to the methodology required NRC staff review and approval. Representatives of the NRC and the industry agreed that this meeting had been useful for the exchange of information on the discussed topics. Subsequent to the meeting, the NRC staff indicated to NEI that the proposals would require prior NRC review and approval in accordance with Generic Letter 95-05.

Project No. 689
Attachments: As stated
cc w/attn: See list

DISTRIBUTION: (NRC-001)

ADAMS-PUBLIC	RPRP-Ref	SCollins/JJohnson	BSheron	WBorchardt
J Strosnider	W Bateman	L Lund	K Karwosk	
E Murphy	J Tsao	J Davis	DMatthews/FGillespie	
CCarpenter	SWest	JBirmingham		

ADAMS Accession Number: ML020850606 (package)

DOCUMENT: G:\RPRP\PCW\MSUM0221SG.WPD

OFFICE	RPRP/DRIP	EMCB/DE	SC:EMCB/DE	SC:RPRP/DRIP
NAME	PWen:kig *	K Karwoski*	L Lund*	S West*
DATE	03/18/02	03/19/02	03/21/02	03/22/02

Official Record Copy

Nuclear Energy Institute

Project No. 689

cc: Mr. Alex Marion, Director
Engineering
Nuclear Energy Institute
Suite 400
1776 I Street, NW
Washington, DC 20006-3708
am@nei.org

Mr. Jim Riley, Project Manager
Engineering
Nuclear Energy Institute
Suite 400
1776 I Street, NW
Washington, DC 20006-3708
Jhr@nei.org