

May 9, 1988

Docket Nos. STN 50-454, STN 50-455,
STN 50-456 AND STN 50-457

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Mr. Henry Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box
Chicago, Illinois 60690

Dear Mr. Bliss:

SUBJECT: CORRECTION TO BYRON AMENDMENT NO. 16 AND BRAIDWOOD AMENDMENT NO. 7

Amendment No. 16 to Facility Operating License Nos. NPF-37 and NPF-66 for Byron Station, Units 1 and 2, and Amendment No. 7 to Facility Operating License Nos. NPF-72 and NPF-75 for Braidwood Station, Units 1 and 2, were issued on April 8, 1988. Page 3/4 3-10 contained a typographical omission for Functional Unit 16. Enclosed are corrected copies of page 3/4 3-10 for both Byron and Braidwood.

We regret any inconvenience that this error may have caused.

Sincerely,

Original Signed by/

Leonard Olshan, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
As stated

cc:
See next page

*See previous concurrence

PDIII-2:PM
LOlshan:bj
5/9/88

PDIII-2:PM*
SSands
05/05/88

PDIII-2:LA*
LLuther
05/05/88

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LNorholm
5/9/88

~~OGC-Rockville~~
~~1/1/88~~

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PDR
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Nuclear Licensing Manager
Commonwealth Edison Company
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We regret any inconvenience that this error may have caused.

Sincerely,

Leonard Olshan, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
As stated

cc:
See next page

PDIII-2:PM
LOlshan:bj
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PDIII-2:PM
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PDIII-2:LA
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5 / 5 / 88

PDIII-2:PD
LNorrholm
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~~OGC-Rockville~~
~~1 / 88~~



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 9, 1988

Docket Nos. STN 50-454, STN 50-455,
STN 50-456 AND STN 50-457

Mr. Henry Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Bliss:

SUBJECT: CORRECTION TO BYRON AMENDMENT NO. 16 AND BRAIDWOOD AMENDMENT NO. 7

Amendment No. 16 to Facility Operating License Nos. NPF-37 and NPF-66 for Byron Station, Units 1 and 2, and Amendment No. 7 to Facility Operating License Nos. NPF-72 and NPF-75 for Braidwood Station, Units 1 and 2, were issued on April 8, 1988. Page 3/4 3-10 contained a typographical omission for Functional Unit 16. Enclosed are corrected copies of page 3/4 3-10 for both Byron and Braidwood.

We regret any inconvenience that this error may have caused.

Sincerely,

A handwritten signature in cursive script that reads "Leon Olshan".

Leonard Olshan, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
As stated

cc:
See next page

Mr. Henry Bliss
Commonwealth Edison Company

Byron/Braidwood

cc:

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Mr. Henry Bliss
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- 2 - Byron/Braidwood

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TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

BRAIDWOOD - UNITS 1 & 2

3/4 3-10

AMENDMENT NO. 7

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	ANALOG CHANNEL OPERATIONAL TEST	TRIP ACTUATING DEVICE OPERATIONAL TEST	ACTUATION LOGIC TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
12. Reactor Coolant Flow-Low	S	R#	Q	N.A.	N.A.	1
13. Steam Generator Water Level-Low-Low	S	R#	Q**	N.A.	N.A.	1, 2
14. Undervoltage-Reactor Coolant Pumps (Above P-7)	N.A.	R	N.A.	Q**(10)	N.A.	1
15. Underfrequency-Reactor Coolant Pumps (Above P-7)	N.A.	R	N.A.	Q(10)	N.A.	1
16. Turbine Trip (Above P-7 or P-8)***						
a. Emergency Trip Header Pressure	N.A.	R	N.A.	S/U(1, 10)	N.A.	1
b. Turbine Throttle Valve Closure	N.A.	R	N.A.	S/U(1, 10)	N.A.	1
17. Safety Injection Input from ESF	N.A.	N.A.	N.A.	R	N.A.	1, 2
18. Reactor Coolant Pump Breaker Position Trip (Above P-7)	N.A.	N.A.	N.A.	R	N.A.	1
19. Reactor Trip System Interlocks						
a. Intermediate Range Neutron Flux, P-6	N.A.	R(4)#	Q	N.A.	N.A.	2##
b. Low Power Reactor Trips Block, P-7	N.A.	R(4)#	Q (8)	N.A.	N.A.	1
c. Power Range Neutron Flux, P-8	N.A.	R(4)#	Q (8)	N.A.	N.A.	1

TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

BYRON - UNITS 1 & 2	FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	ANALOG CHANNEL OPERATIONAL TEST	TRIP ACTUATING DEVICE OPERATIONAL TEST	ACTUATION LOGIC TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
	12. Reactor Coolant Flow-Low	S	R [#]	Q	N.A.	N.A.	1
	13. Steam Generator Water Level-Low-Low	S	R [#]	Q**	N.A.	N.A.	1, 2
	14. Undervoltage-Reactor Coolant Pumps (Above P-7)	N.A.	R	N.A.	Q**(10)	N.A.	1
3/4 3-10	15. Underfrequency-Reactor Coolant Pumps (Above P-7)	N.A.	R	N.A.	Q(10)	N.A.	1
	16. Turbine Trip (Above P-7 or P-8)***						
	a. Emergency Trip Header Pressure	N.A.	R	N.A.	S/U(1, 10)	N.A.	1
	b. Turbine Throttle Valve Closure	N.A.	R	N.A.	S/U(1, 10)	N.A.	1
	17. Safety Injection Input from ESF	N.A.	N.A.	N.A.	R	N.A.	1, 2
	18. Reactor Coolant Pump Breaker Position Trip (Above P-7)	N.A.	N.A.	N.A.	R	N.A.	1
AMENDMENT NO. 16	19. Reactor Trip System Interlocks						
	a. Intermediate Range Neutron Flux, P-6	N.A.	R(4) [#]	Q	N.A.	N.A.	2##
	b. Low Power Reactor Trips Block, P-7	N.A.	R(4) [#]	Q (8)	N.A.	N.A.	1
	c. Power Range Neutron Flux, P-8	N.A.	R(4) [#]	Q (8)	N.A.	N.A.	1