

November 3, 1995

Mr. Richard W. Smedley
Manager Licensing
Palisades Plant
27780 Blue Star Memorial Highway
Covert, MI 49043

SUBJECT: PALISADES PLANT - ISSUANCE OF AMENDMENT RE: ADMINISTRATIVE CONTROLS
(TAC NO. M92899)

Dear Mr. Smedley:

The Commission has issued the enclosed Amendment No. 170 to Facility Operating License No. DPR-20 for the Palisades Plant. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated July 5, 1995.

The amendment revises Section 6.0 of the TS to incorporate several administrative control and editorial changes to the Training, Plant Review Committee, and Plant Safety and Licensing Staff sections.

A copy of our Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Marsha Gamberoni, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-255

Enclosures: 1. Amendment No. 170 to DPR-20
2. Safety Evaluation

cc w/encl: See next page

DOCUMENT NAME: G:\WPDOCS\PALISADE\PAL92899.AMD

*SEE PREVIOUS CONCURRENCE

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DATED: November 3, 1995

AMENDMENT NO. 170 TO FACILITY OPERATING LICENSE NO. DPR-20-PALISADES

~~Docket File~~

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PDIII-1 Reading

J. Roe

E. Adensam (e)

B. Holian

C. Jamerson

M. Gamberoni

OGC

G. Hill (2)

C. Grimes

ACRS

W. Kropp, RIII

SEDB

cc: Plant Service list

070163

DF01.1

Mr. Richard W. Smedley
Consumers Power Company

Palisades Plant

cc:

Mr. Thomas J. Palmisano
Plant General Manager
Palisades Plant
27780 Blue Star Memorial Highway
Covert, Michigan 49043

Nuclear Facilities and Environmental
Monitoring Section Office
Division of Radiological Health
Department of Public Health
3423 N. Logan Street
P. O. Box 30195
Lansing, Michigan 48909

Mr. Robert A. Fenech
Vice President, Nuclear Operations
Palisades Plant
27780 Blue Star Memorial Highway
Covert, Michigan 49043

Gerald Charnoff, Esquire
Shaw, Pittman, Potts and Trowbridge
2300 N Street, N. W.
Washington DC 20037

M. I. Miller, Esquire
Sidley & Austin
54th Floor
One First National Plaza
Chicago, Illinois 60603

Michigan Department of Attorney
General
Special Litigation Division
630 Law Building
P.O. Box 30212
Lansing, Michigan 48909

Mr. Thomas A. McNish
Vice President & Secretary
Consumers Power Company
212 West Michigan Avenue
Jackson, Michigan 49201

Judd L. Bacon, Esquire
Consumers Power Company
212 West Michigan Avenue
Jackson, Michigan 49201

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
801 Warrenville Road
Lisle, Illinois 60532-4351

Jerry Sarno
Township Supervisor
Covert Township
36197 M-140 Highway
Covert, Michigan 49043

Office of the Governor
Room 1 - Capitol Building
Lansing, Michigan 48913

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
Palisades Plant
27782 Blue Star Memorial Highway
Covert, Michigan 49043

September 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CONSUMERS POWER COMPANY

DOCKET NO. 50-255

PALISADES PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 170
License No. DPR-20

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consumers Power Company (the licensee) dated July 5, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to the license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-20 is hereby amended to read as follows:

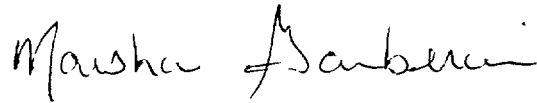
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Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 170 , and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "Marsha Gamberoni".

Marsha Gamberoni, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: November 3, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 170

FACILITY OPERATING LICENSE NO. DPR-20

DOCKET NO. 50-255

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

REMOVE

6-5
6-9

INSERT

6-5
6-9

6.4 (Deleted)

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW COMMITTEE (PRC)

6.5.1.1 FUNCTION

The Plant Review Committee (PRC) shall function to advise the Plant General Manager on all matters related to nuclear safety.

6.5.1.2 COMPOSITION

The PRC is composed of nine regular members. The qualification level for PRC members shall be at least equivalent to those described in Section 4.4 of ANSI N18.1-1971. The PRC shall include representatives from the Operations, Radiological Services, Maintenance and Engineering Departments. The Chairman, Alternate Chairmen, and members shall be designated in administrative procedures by the Plant General Manager.

6.5.1.3 ALTERNATES

Alternate members of the PRC shall be appointed in writing by the PRC Chairman to serve on a temporary basis. No more than two alternates shall participate as voting members at any one time in PRC activities.

6.5.2.6 RECORDS

Records of NPAD activities shall be maintained. Reports shall be prepared and distributed as indicated below:

- a. The results of reviews, performed pursuant to Section 6.5.2.4.1 and Section 6.5.2.4.2, shall be reported to the Vice President - NOD at least monthly.
- b. A report assessing the overall nuclear safety performance of Palisades shall be provided to senior Consumers Power Company management annually.

6.5.3 PLANT SAFETY AND LICENSING

6.5.3.1 FUNCTION

The Plant Safety and Licensing staff shall review proposed changes in design or operation and such other matters as the PRC may assign to identify issues significant to nuclear safety and recommend nuclear safety improvements.

6.5.3.2 COMPOSITION

The Plant Safety and Licensing staff responsible for the review function shall be an experienced technical staff meeting the qualifications of Section 6.3.

6.5.3.3 RESPONSIBILITIES

The Plant Safety and Licensing staff may provide nuclear safety review as delegated by PRC for:

- a. Procedures, programs and changes thereto identified in Specification 6.8 and any additional procedures and changes thereto identified by the Plant General Manager as significant to nuclear safety.
- b. All proposed tests or experiments.
- c. All proposed changes or modifications to plant systems or equipment.
- d. The Site Emergency Plan.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 170 TO FACILITY OPERATING LICENSE NO. DPR-20

CONSUMERS POWER COMPANY

PALISADES PLANT

DOCKET NO. 50-255

1.0 INTRODUCTION

By letter dated July 5, 1995, the Consumers Power Company (the licensee) requested an amendment to the Technical Specifications (TS) appended to Facility Operating License No. DPR-20 for the Palisades Plant. The proposed amendment would change the Administrative Controls section of the Palisades TS.

The requested changes and NRC staff's evaluation and findings pertaining to each item of change are discussed in Section 2.0.

2.0 BACKGROUND

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to state TS to be included as part of the license. The Commission's regulatory requirements related to the content of TS are set forth in 10 CFR 50.36. That regulation requires that the TS include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TS.

The Commission has provided guidance for the contents of TS in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" ("Final Policy Statement"), 58 FR 39132 (July 22, 1993), in which the Commission indicated that compliance with the Final Policy Statement satisfies Section 182a of the Act. In particular, the Commission indicated that certain items could be relocated from the TS to licensee-controlled documents, consistent with the standard enunciated in *Portland General Electric Co. (Trojan Nuclear Plant)*, ALAB-531, 9 NRC 263, 273 (1979). In that case, the Atomic Safety and Licensing Appeal Board indicated that "technical specifications are to be reserved for those matters as to which the imposition of rigid conditions or limitations upon reactor operation is deemed necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety."

Consistent with this approach, the Final Policy Statement identified four criteria to be used in determining whether a particular matter is required to be included in the TS, as follows: (1) Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a design-basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design-basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety.¹ As a result, existing TS requirements which fall within or satisfy any of the criteria in the Final Policy Statement must be retained in the TS, while those TS requirements which do not fall within or satisfy these criteria may be relocated to other, licensee-controlled documents.

3.0 EVALUATION

3.1 Deletion of Section 6.4: TRAINING

The licensee requested that Section 6.4 be deleted from the TS because

(i) the specification was outdated and referred to Appendix A of 10 CFR Part 55, which has been superseded.

(ii) the requested change would facilitate the conversion of the plant-specific TS to the more generic improved Combustion Engineering (CE) TS (NUREG-1432, April 7, 1995). NUREG-1432 was written with the objective to reduce redundancy, improve the clarity, and standardize the TS.

(iii) the training requirement and recommendation need not be specified in the TS since 10 CFR 55.59 (Subpart F) stipulates the operator training and requalification requirements.

(iv) the training programs at Palisades are accredited by the National Academy for Nuclear Training Accreditation Board, and these programs meet or exceed the requirements under Section 6.4 of the TS.

¹ The Commission recently adopted amendments to 10 CFR 50.36, pursuant to which the rule was revised to codify and incorporate these criteria. See Final Rule, "Technical Specifications," 60 FR 36953 (July 19, 1995). The Commission indicated that reactor core isolation cooling, isolation condenser, residual heat removal, standby liquid control, and recirculation pump trip systems are included in the TS under Criterion 4, although it recognized that other structures, systems, and components could also meet this criterion. (60 FR at 36956)

The NRC staff has evaluated the proposed deletion of Section 6.4 of the TS and the supporting information presented by the licensee. The staff finds deletion of Section 6.4. from the Palisades TS acceptable in part because 10 CFR 55.59 covers the licensed operator training requirements. However, we note that ANSI N18.1 1971, which is referenced in Section 6.4 addresses training of all plant staff and 10 CFR 55.59 is limited to training and requalification of licensed operators. Section 50.120 of 10 CFR provides the more general training requirements for nonlicensed staff.

Therefore, the NRC staff concludes that the deletion of Section 6.4 of the TS is acceptable as 10 CFR Part 55 and 10 CFR 10.120 stipulate the regulatory requirements.

3.2 Revision of Section 6.5.1.2: Composition of the Plant Review Committee (PRC)

The TS Section 6.5.1.2 explicitly states that the PRC "is composed of nine regular members from either the Palisades staff or the Nuclear Engineering and Construction Organization (NECO) staff." The proposed amendment maintains the number of regular members of the PRC but deletes reference to their department affiliation. The licensee requested that such plant-specific terms be edited out of the TS in order to avoid periodic amendments which would be required with any plant reorganization.

The licensee also requested that the Chairman and the Alternate Chairman of the PRC be designated through administrative procedures as opposed to the current "in writing" selection process.

The proposed removal of the reference to the PRC member's department affiliation when either NECO or Palisades staff can be selected is a non-substantive change. The requested change does streamline the TS by eliminating unnecessary organization-specific statements.

In addition, the designation of the Chairman, Alternate Chairman, and the members of the PRC through administrative procedures standardizes the selection process.

These changes are consistent with the acceptance criteria of Section 13 of NUREG-0800, the Standard Review Plan, and are administrative changes that do not alter the composition or the responsibilities of the PRC. Therefore, the NRC staff finds the proposed modification of Section 6.5.1.2 of the TS acceptable.

3.3 Revision of Sections 6.5.3.1 and 6.5.3.2: Function and Composition of the Plant Safety and Licensing Staff

In Sections 6.5.3.1 and 6.5.3.2 of the TS, the functions and the compositions of the Plant Safety and Licensing (PSL) organization are outlined. Initially, the licensee established the PSL organization in order to take precautionary

steps and partially institute the Three Mile Island (TMI) action plan, even though Palisades was not legally bound to comply with the post-TMI safeguards and guidelines.

In Section 6.5.3.1, the licensee seeks to rephrase the paragraph pertaining to the function of the PSL organization. The requested modification identifies the functions of the PSL staff as opposed to the PSL organization.

The proposed editorial change in Section 6.5.3.2 cancels any reference to the required PSL staff size of five and permits any desired number of staff to conduct the review function. Another proposed change requires that the PSL review staff "be an experienced technical staff" and meet the qualifications covered under Section 6.3. Further changes also strike out the requirement that the PSL staff report their review assessment to their independent PSL department director.

The licensee states that:

(i) the Plant Safety and Licensing department was responsible for several different functions. Therefore, only those PSL staff responsible for the review functions should be required to meet the Section 6.3 qualifications.

(ii) currently, only three out of the five qualified review staff conduct the actual plant review process. Hence, the number of review staff should be allowed to fluctuate with the workload.

(iii) The title of the director and the specific department the PSL review staff report to represent unnecessary organization-specific stipulations. The line of authority and the plant organizational plan should remain under the control of the licensee, and the quality review process does not necessarily require departmental independence.

The proposed changes to Sections 6.5.3.1 and 6.5.3.2 may lead to an actual change in the composition and the function of the Plant Safety and Licensing Organization. However, the establishment of a PSL review unit was a voluntary action taken by the licensee, and it is appropriate to allow the licensee flexibility over the unit's composition and function. Furthermore, the new improved CE TS does not contain specific guidelines for the PSL review unit.

The modifications to Sections 6.5.3.1 and 6.5.3.2, as proposed by the licensee, meet the objectives of removing redundancy, of reducing the unnecessary issuance of amendments due to internal plant reorganizations, and of improving the readability of the TS. The changes do not decrease the licensee's organizational effectiveness as required by Appendix B to 10 CFR Part 50. Therefore, the NRC staff finds these changes acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Michigan State official was notified of the proposed issuance of the amendment. The Michigan State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes administrative procedures and requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Zena Abdullahi

Date: November 3, 1995