

July 1, 1993

Docket No. 50-255

Mr. Gerald B. Slade
Plant General Manager
Palisades Plant
Consumers Power Company
27780 Blue Star Memorial Highway
Covert, Michigan 49043

Dear Mr. Slade:

SUBJECT: PALISADES PLANT - RADIOLOGICAL EFFLUENT TECHNICAL SPECIFICATIONS
(RETS) - ERRATA TO AMENDMENT NO. 154 TO FACILITY OPERATING LICENSE
NO. DPR-20 (TAC NO. M75060)

In response to your letter dated January 20, 1993, and as clarified in a conference call with Mr. Barry Young (Consumers Power) on June 30, 1993, we have confirmed that the changes to the plant Technical Specifications (TS) implemented by Amendment No. 154 to License No. DPR-20 for the Palisades Plant, which were transmitted to you by letter dated December 18, 1992, contained several minor typographical and pagination errors. In addition, page 3-50 was inadvertently omitted from the original issuance.

However, page 6-10 was not deleted in the original issuance of Amendment No. 54 and remains unchanged by this amendment.

The enclosed errata to Amendment No. 154 provides the corrected TS pages.

Sincerely,

Original signed by

Anthony H. Hsia, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:
Corrected TS Pages

cc w/enclosures:
See next page

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DATED: July 1, 1993

CORRECTION TO AMENDMENT NO. 154 TO FACILITY OPERATING LICENSE NO. DPR-20-PALISADES

[REDACTED]
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cc: Plant Service list

ERRATA

ATTACHMENT TO LICENSE AMENDMENT NO. 154

FACILITY OPERATING LICENSE NO. DPR-20

DOCKET NO. 50-255

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

CORRECTED PAGES

REMOVE

i
ii
iii
iv
v
3-50
3-113
4-10
6-12
6-35

INSERT

i
ii
iii
iv
v
3-50
3-113
4-10
6-12
6-35

PALISADES PLANT TECHNICAL SPECIFICATIONS
TABLE OF CONTENTS

<u>SECTION</u>	<u>DESCRIPTION</u>	<u>PAGE NO</u>
1.0	<u>DEFINITIONS</u>	1-1
1.1	REACTOR OPERATING CONDITIONS	1-1
1.2	PROTECTIVE SYSTEMS	1-3
1.3	INSTRUMENTATION SURVEILLANCE	1-3
1.4	MISCELLANEOUS DEFINITIONS	1-4
2.0	<u>SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS</u>	2-1
2.1	SAFETY LIMITS - REACTOR CORE	2-1
2.2	SAFETY LIMITS - PRIMARY COOLANT SYSTEM PRESSURE	2-1
2.3	LIMITING SAFETY SYSTEM SETTINGS - RPS	2-1
	Table 2.3.1 Reactor Protective System Trip Setting Limits	2-2
B2.1	Basis - Reactor Core Safety Limit	B2-1
B2.2	Basis - Primary Coolant System Safety Limit	B2-2
B2.3	Basis - Limiting Safety System Settings	B2-3
3.0	<u>LIMITING CONDITIONS FOR OPERATION</u>	3-1
3.0	APPLICABILITY	3-1
3.1	PRIMARY COOLANT SYSTEM	3-1b
3.1.1	Operable Components	3-1b
	Figure 3-0 Reactor Inlet Temperature vs Operating Pressure	3-3a
3.1.2	Heatup and Cooldown Rates	3-4
	Figure 3-1 Pressure - Temperature Limits for Heatup	3-9
	Figure 3-2 Pressure - Temperature Limits for Cooldown	3-10
	Figure 3-3 Pressure - Temperature Limits for Hydro	3-11
3.1.3	Minimum Conditions for Criticality	3-12
3.1.4	Maximum Primary Coolant Radioactivity	3-17
3.1.5	Primary Coolant System Leakage Limits	3-20
3.1.6	Maximum PCS Oxygen and Halogen Concentration	3-23
3.1.7	Primary and Secondary Safety Valves	3-25
3.1.8	Overpressure Protection Systems	3-25a
3.2	CHEMICAL AND VOLUME CONTROL SYSTEM	3-26
3.3	EMERGENCY CORE COOLING SYSTEM	3-29
3.4	CONTAINMENT COOLING	3-34
3.5	STEAM AND FEEDWATER SYSTEMS	3-38
3.6	CONTAINMENT SYSTEM	3-40
	Table 3.6.1 Containment Penetrations and Valves	3-40b
3.7	ELECTRICAL SYSTEMS	3-41
3.8	REFUELING OPERATIONS	3-46
3.9	Deleted	3-50

PALISADES PLANT TECHNICAL SPECIFICATIONS
TABLE OF CONTENTS

<u>SECTION</u>	<u>DESCRIPTION</u>	<u>PAGE NO</u>
3.0	<u>LIMITING CONDITIONS FOR OPERATION</u> (continued)	
3.10	CONTROL ROD AND POWER DISTRIBUTION LIMITS	3-58
3.10.1	Shutdown Margin Requirements	3-58
3.10.2	Deleted	3-59
3.10.3	Part-Length Control Rods	3-59
3.10.4	Misaligned or Inoperable Rod	3-60
3.10.5	Regulating Group Insertion Limits	3-60
3.10.6	Shutdown Rod Limits	3-61
3.10.7	Low Power Physics Testing	3-61
3.10.8	Center Control Rod Misalignment	3-61
	Figure 3-6 Control Rod Insertion Limits	3-62
3.11	POWER DISTRIBUTION INSTRUMENTATION	3-65
3.11.1	Incore Detectors	3-65
3.11.2	Excore Power Distribution Monitoring System	3-66a
	Figure 3.11-1 Axial Variation Bounding Condition	3-66d
3.12	MODERATOR TEMPERATURE COEFFICIENT OF REACTIVITY	3-67
3.13	Deleted	3-69
3.14	CONTROL ROOM VENTILATION	3-70
3.15	REACTOR PRIMARY SHIELD COOLING SYSTEM	3-70a
3.16	ESF SYSTEM INITIATION INSTRUMENTATION SETTINGS	3-71
	Table 3.16.1 ESF System Initiation Instrument Setting Limits	3-75
3.17	INSTRUMENTATION AND CONTROL SYSTEMS	3-76
	Table 3.17.1 Instrument Requirements for RPS	3-78
	Table 3.17.2 Instrument Requirements for ESF Systems	3-79
	Table 3.17.3 Instrument Conditions for Isolation Functions	3-80
	Table 3.17.4 Instrument Requirements for Other Safety Features	3-81
3.18	Deleted	3-82
3.19	IODINE REMOVAL SYSTEM	3-84
3.20	SHOCK SUPPRESSORS (Snubbers)	3-88
3.21	MOVEMENT HEAVY LOADS	3-92
3.22	Deleted	3-96
3.23	POWER DISTRIBUTION LIMITS	3-103
3.23.1	Linear Heat Rate	3-103
	Table 3.23.1 Linear Heat Rate Limits	3-107
	Table 3.23.2 Radial Peaking Factor Limits	3-107
	Table 3.23-3 Power Distribution Measurement Uncertainty	3-107
	Figure 3.23-1 Allowable LHR vs Peak Power Location	3-108
3.23.2	Radial Peaking Factors	3-111
3.23.3	Quadrant Power Tilt - Tq	3-112
3.24	Deleted	3-113
3.25	ALTERNATE SHUTDOWN SYSTEM	3-134
	Table 3.25.1 Alternate Shutdown Minimum Equipment	3-135

PALISADES PLANT TECHNICAL SPECIFICATIONS
TABLE OF CONTENTS

<u>SECTION</u>	<u>DESCRIPTION</u>	<u>PAGE NO</u>
4.0	<u>SURVEILLANCE REQUIREMENTS</u>	4-1
4.1	INSTRUMENTATION AND CONTROL	4-1e
4.1.1	Overpressure Protection Systems	4-1e
Table 4.1.1	Frequencies for Testing of RPS Instrumentation	4-3
Table 4.1.2	Frequencies for Testing of EFS Instrumentation	4-6
Table 4.1.3	Frequencies for Testing of Misc. Instrumentation	4-10
4.2	EQUIPMENT AND SAMPLING TESTS	4-13
Table 4.2.1	Minimum Frequencies for Sampling Tests	4-14
Table 4.2.2	Minimum Frequencies for Equipment Tests	4-15
Table 4.2.3	HEPA Filter and Charcoal Adsorber Systems	4-15c
4.3	SYSTEMS SURVEILLANCE	4-16
Table 4.3.1	Primary Coolant System Pressure Isolation Valves	4-19
Table 4.3.2	Miscellaneous Surveillance Items	4-23
4.4	Deleted	4-24
4.5	CONTAINMENT TESTS	4-25
4.5.1	Integrated Leakage Rate Tests	4-25
4.5.2	Local Leak Detection Tests	4-27
4.5.3	Recirculation Heat Removal Systems	4-28a
4.5.4	Surveillance for Prestressing System	4-29
4.5.5	End Anchorage Concrete Surveillance	4-32
4.5.6	Containment Isolation Valves	4-32
4.5.7	Deleted	4-32a
4.5.8	Dome Delamination Surveillance	4-32a
4.6	SAFETY INJECTION AND CONTAINMENT SPRAY SYSTEMS TESTS	4-39
4.6.1	Safety Injection System	4-39
4.6.2	Containment Spray System	4-39
4.6.3	Pumps	4-40
4.6.4	Deleted	4-40
4.6.5	Containment Air Cooling System	4-40
4.7	EMERGENCY POWER SYSTEM PERIODIC TESTS	4-42
4.7.1	Diesel Generators	4-42
4.7.2	Station Batteries	4-42
4.7.3	Emergency Lighting	4-43
4.8	MAIN STEAM STOP VALVES	4-44
4.9	AUXILIARY FEEDWATER SYSTEM	4-45
4.10	REACTIVITY ANOMALIES	4-46
4.11	Deleted	4-46
4.12	AUGMENTED ISI PROGRAM FOR HIGH ENERGY LINES	4-60
4.13	Deleted	4-65
4.14	AUGMENTED ISI PROGRAM FOR STEAM GENERATORS	4-66
4.15	PRIMARY SYSTEM FLOW MEASUREMENT	4-70
4.16	ISI PROGRAM FOR SHOCK SUPPRESSORS (Snubbers)	4-71
4.17	Deleted	4-75

PALISADES PLANT TECHNICAL SPECIFICATIONS
TABLE OF CONTENTS

<u>SECTION</u>	<u>DESCRIPTION</u>	<u>PAGE NO</u>
4.0	<u>SURVEILLANCE REQUIREMENTS</u> (Continued)	
4.18	POWER DISTRIBUTION INSTRUMENTATION	4-81
4.18.1	Incore Detectors	4-81
4.18.2	Excore Monitoring System	4-82
4.19	POWER DISTRIBUTION LIMITS	4-83
4.19.1	Linear Heat Rate	4-83
4.19.2	Radial Peaking Factors	4-84
4.20	MODERATOR TEMPERATURE COEFFICIENT (MTC)	4-85
4.21	ALTERNATE SHUTDOWN SYSTEM	4-86
	Table 4.21.1 Alternate Shutdown Monitoring System Surveillance Requirements	4-87
5.0	<u>DESIGN FEATURES</u>	5-1
5.1	SITE	5-1
5.2	CONTAINMENT DESIGN FEATURES	5-1
5.2.1	Containment Structures	5-1
5.2.2	Penetrations	5-2
5.2.3	Containment Structure Cooling Systems	5-2
5.3	NUCLEAR STEAM SUPPLY SYSTEM (NSSS)	5-2
5.3.1	Primary Coolant System	5-2
5.3.2	Reactor Core and Control	5-3
5.3.3	Emergency Core Cooling System	5-3
5.4	FUEL STORAGE	5-4
5.4.1	New Fuel Storage	5-4
5.4.2	Spent Fuel Storage	5-4a
	Figure 5-1 Site Environment TLD Stations	5-5
6.0	<u>ADMINISTRATIVE CONTROLS</u>	6-1
6.1	RESPONSIBILITY	6-1
6.2	ORGANIZATION	6-1
6.2.1	Offsite and Onsite Organizations	6-1
6.2.2	Plant Staff	6-2
6.3	PLANT STAFF QUALIFICATIONS	6-3
	Table 6.2-1 Minimum Shift Crew Composition	6-4
6.4	TRAINING	6-5
6.5	REVIEW AND AUDIT	6-5
6.5.1	Plant Review Committee	6-5
6.5.2	Nuclear Performance Assessment Department	6-6a
6.5.3	Plant Safety and Licensing	6-9

PALISADES PLANT TECHNICAL SPECIFICATIONS
TABLE OF CONTENTS

<u>SECTION</u>	<u>DESCRIPTION</u>	<u>PAGE NO</u>
6.0	<u>ADMINISTRATIVE CONTROLS</u> (Continued)	
6.6	Deleted	6-10
6.7	SAFETY LIMIT VIOLATION	6-10
6.8	PROCEDURES AND PROGRAMS	6-11
6.9	REPORTING REQUIREMENTS	6-14
6.9.1	Routine Reports	6-14
6.9.1.a	Start-up Report	6-14
6.9.1.b	Annual Report	6-14
6.9.1.c	Monthly Operating Report	6-15
6.9.1.d	Radioactive Effluent Release Report	6-15
6.9.1.e	Radiological Environmental Operating Report	6-15
6.9.2	Reportable Events	6-15
6.9.3	Nonroutine Reports	6-15
6.9.4	Special Reports	6-26
6.10	RECORD RETENTION	6-26
6.11	RADIATION PROTECTION PROGRAM	6-28
6.12	HIGH RADIATION AREA	6-28
6.13	Deleted	6-33
6.14	Deleted	6-33
6.15	SYSTEMS INTEGRITY	6-33
6.16	IODINE MONITORING	6-33
6.17	POST ACCIDENT SAMPLING	6-34
6.18	OFFSITE DOSE CALCULATION MANUAL	6-35
6.19	PROCESS CONTROL PROGRAM	6-35
6.20	Deleted	6-36
6.21	SEALED SOURCE CONTAMINATION	6-37
6.22	SECONDARY WATER CHEMISTRY	6-38

3.9 EFFLUENT RELEASE

DELETED IN ITS ENTIRETY

12 Pages

3-50
3-50a
3-51
3-51a
3-52
3-52a
3-53
3-53a
3-54
3-55
3-56
3-57

3-50

Amendment No. 83, 154

POWER DISTRIBUTION LIMITS

3.23.3 QUADRANT POWER TILT - T_q

LIMITING CONDITION FOR OPERATION

References

- (1) FSAR, Section 7.4.2.2
- (2) FSAR, Section 7.6.2.4

3.24 Deleted

(Next page is 3-134)

3-113

Amendment No. ~~144~~, 154
Corrected Page

Minimum Frequencies for Checks, Calibrations and Testing of Miscellaneous Instrumentation and Controls (Cont'd)

TABLE 4.1.3

<u>Channel Description</u>	<u>Surveillance Function</u>	<u>Frequency</u>	<u>Surveillance Method</u>
1. Source Range Neutron Monitors	a. Check	S	a. Comparison of both channel count rate indications when in service. b. Internal test signals. c. Channel alignment through measurement/adjustment of internal test points.
	b. Test	P	
	c. Calibrate	R	
2. Primary Rod Position Indication System	a. Check	S	a. Comparison of output data with secondary RPIS. b. Check of power dependent insertion limits monitoring system. c. Physically measured rod drive position used to verify system accuracy. Check rod position interlocks.
	b. Check	M	
	c. Calibrate	R	
3. Secondary Rod Position Indication System	a. Check	S	a. Comparison of output data with primary RPIS. b. Same as 2(b) above. c. Same as 2(c) above, including out-of-sequence alarm function.
	b. Check	M	
	c. Calibrate	R	
4. Area Monitors	a. Check	D	a. Normal readings observed and internal test signals used to verify instrument operation. b. Exposure to known external radiation source. c. Detector exposed to remote operated radiation check source or integral electronic check source.
	b. Calibrate	R	
	c. Test	M	
5. Emergency Plan Radiation Instruments	a. Calibrate	A	a. Exposure to known radiation source. b. Battery check.
	b. Test	M	
6. (Deleted)			
7. Pressurizer Level Instruments	a. Check	S	a. Comparison of two wide and two narrow range independent level readings. b. Known differential pressure applied to sensor. c. Signal to meter relay adjusted with test device.
	b. Calibrate	R	
	c. Test	M	

ADMINISTRATIVE CONTROLS

6.8.4 The following programs shall be established, implemented, and maintained:

a. Radioactive Effluent Controls Program

A program shall be provided conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in the ODCM, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- 1) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM,
- 2) Limitations on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 CFR 20, Appendix B, Table II, Column 2.
- 3) Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.106 and with the methodology and parameters in the ODCM,
- 4) Limitation on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from each unit to UNRESTRICTED AREAS conforming to Appendix I to 10 CFR 50,
- 5) Limitations on the dose rate resulting from radioactive material released in gaseous effluents to areas beyond the SITE BOUNDARY conforming to the doses associated with 10 CFR 20, Appendix B, Table II, Column 1.
- 6) Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR 50,
- 7) Limitations on the annual and quarterly doses to a MEMBER OF THE PUBLIC from Iodine-131, Iodine-133, tritium and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR 50,
- 8) Limitations on the annual doses or dose commitment to any MEMBER OF THE PUBLIC due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR 190.

ADMINISTRATIVE CONTROLS

6.18 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Changes to ODCM:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.20. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR 190, 10 CFR 50.36a, and Appendix I to 10 CFR 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.
- b. Shall become effective after the review and acceptance by the PRC and the approval of the Plant General Manager.
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

6.19 PROCESS CONTROL PROGRAM

Changes to the PROCESS CONTROL PROGRAM:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.20. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluation justifying the change(s) and
 - 2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
- b. Shall become effective after review and acceptance by the PRC and approval of the Plant General Manager.