

July 18, 1989

Docket No. 50-255
Serial No. PAL-89-028

Mr. Kenneth W. Berry
Director, Nuclear Licensing
Consumers Power Company
1945 West Parnall Road
Jackson, Michigan 49201

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OC/LFMB

Dear Mr. Berry:

SUBJECT: CORRECTION TO AMENDMENT NO. 126 (TAC NO. 64339)

On June 1, 1989, the Commission issued Amendment No. 126 to Provisional Operating License No. DPR-20) for the Palisades Plant. The amendment consisted of changes to the Appendix A Technical Specifications (TSs) in response to your application dated March 3, 1982, as supplemented by letters dated August 29, 1984, March 21, 1988, and April 20 and May 24, 1989 November 21, 1985.

The amendment revised the Palisades Plant Technical Specifications with regard to the requirements for leak-rate testing of the containment air lock door seals and the associated action to be taken if the allowable leak-rate is exceeded. However, several of the substitute TS pages issued with the amendment contained some errors with respect to revision notations and past amendments. Also, the amendment itself did not indicate the dated when the amendment must be implemented.

These errors have been corrected; compete copies of all license and TS pages affected by Amendment No. 126 are provided herein.

Sincerely,
original signed by

Albert W. De Agazio, Sr. Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc w/enclosures:
See next page

[CORRECTION TO AMEND 126]

LA/PD31:DRSP
PSHUTTLEWORTH
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PM/PD31:DRSP
ADEAGAZIO:cr
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Director, Nuclear Licensing
Consumers Power Company
1945 West Parnall Road
Jackson, Michigan 49201

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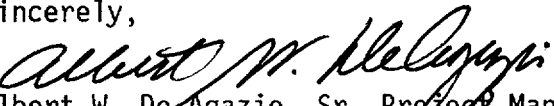
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Enclosures:
As stated

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Mr. Kenneth W. Berry
Consumers Power Company

Palisades Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSUMERS POWER COMPANY

PALISADES PLANT

DOCKET NO. 50-255

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 126
License No. DPR-20

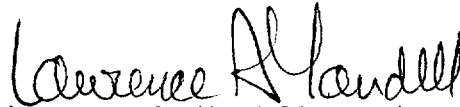
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consumers Power Company (the licensee) dated March 3, 1982 as supplemented by letters dated August 29, 1984, March 21, 1988, and April 20 and May 4, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Provisional Operating License No. DPR-20 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 126, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented not later than July 24, 1989.

FOR THE NUCLEAR REGULATORY COMMISSION



Lawrence A. Yandell, Acting Director
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 18, 1989

- (2) If any periodic integrated leak rate test fails to meet the acceptance criteria, the test schedule applicable to subsequent integrated leak rate tests will be reviewed and approved by the Commission.

e. Report of Test Results

Each integrated leak rate test will be the subject of a summary technical report which will include summaries of local leak detection tests and leak test of the recirculation heat removal systems.

4.5.2 Local Leak Detection Tests

a. Test

- (1) Local leak rate tests shall be performed at a pressure of not less than 55 psig.
- (2) Local leak rate tests for checking air lock door seals within 72 hours of each door opening shall be performed at a pressure of not less than 10 psig. /
- (3) Acceptable methods of testing are halogen gas detection, soap bubble, pressure decay, or equivalent. /
- (4) The local leak rate shall be measured for each of the following components: /
 - (a) Containment penetrations that employ resilient seal gaskets, sealant compounds, or bellows.
 - (b) Air lock and equipment door seals.
 - (c) Fuel transfer tube.
 - (d) Isolation valves on the testable fluid systems' lines penetrating the containment.
 - (e) Other containment components which require leak repair in order to meet the acceptance criterion for any integrated leak rate test.

b. Acceptance Criteria

- (1) The total leakage from all penetrations and isolation valves shall not exceed 0.60 La. /
- (2) The leakage for an air lock door seal test shall not exceed 0.023 La. /

c. Corrective Action

- (1) If at any time it is determined that 0.60 La is exceeded, repairs shall be initiated immediately.

4.5 CONTAINMENT TESTS (Continued)

4.5.2 Local Leak Detection Tests (Continued)

If repairs are not completed and conformance to the acceptance criterion of 4.5.2.b(1) is not demonstrated within 48 hours, the Plant shall be placed in at least hot shutdown within the next 6 hours and in at least cold shutdown within the following 30 hours.

- (2) If at any time it is determined that total containment leakage exceeds L_a , within one hour action shall be initiated to bring the Plant to hot shutdown within the next six (6) hours and cold shutdown within the following thirty (30) hours.

- (3) If air lock door seal leakage is greater than $0.023 L_a$, repairs shall be initiated immediately to restore the door to less than specification 4.5.2.b(2). In the event repairs cannot be completed within 7 days, the Plant shall be brought to a hot shutdown condition within the next six (6) hours and cold shutdown within the following thirty (30) hours.

If air lock door seal leakage results in one (1) door causing total containment leakage to exceed $0.60 L_a$, the door shall be declared inoperable and the remaining operable door shall be immediately locked closed and tested within four (4) hours. As long as the remaining door is found to be operable, the provisions of 4.5.2.c(2) do not apply. Repairs shall be initiated immediately to establish conformance with specification 4.5.2.b(1). In the event conformance to this specification cannot be established within 48 hours the Plant shall be brought to a hot shutdown within the next 6 hours and cold shutdown within the following 30 hours.

d. Test Frequency

- (1) Individual penetrations and containment isolation valves shall be leak rate tested at a frequency of at least every six months prior to the first postoperational integrated leak rate test and at a frequency of at least every refueling thereafter, not exceeding a two-year interval, except as specified in (a) and (b) below:

- (a) The containment equipment hatch and the fuel transfer tube shall be tested at each refueling shutdown or after each time used, if that be sooner.

4.5 CONTAINMENT TESTS (Continued)

/

4.5.2 Local Leak Detection Tests (Continued)

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- (b) A full air lock penetration test shall be performed at six-month intervals. During the period between the six-month tests when containment integrity is required, a reduced pressure test for the door seals or a full air lock penetration test shall be performed within 72 hours after either each air lock door opening or the first of a series of openings.
- (2) Each three months the isolation valves must be stroked to the position required to fulfill their safety function unless it is established that such operation is not practical during plant operation. The latter valves shall be full-stroked during each cold shutdown.

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4.5.3 Recirculation Heat Removal Systems

a. Test

- (1) The portion of the shutdown cooling system that is outside the containment shall be tested either by use in normal operation or hydrostatically tested at 255 psig at the interval specified in 6.15.
- (2) Piping from valves CV-3029 and CV-3030 to the discharge of the safety injection pumps and containment spray pumps shall be hydrostatically tested at no less than 100 psig at the interval specified in 6.15.

an important part of the structural integrity of the containment is maintained.

The basis for specification of a total leakage rate of 0.60 La from penetrations and isolation valves is specified to provide assurance that the integrated leak rate would remain within the specified limits during the intervals between integrated leak rate tests. This value allows for possible deterioration in the intervals between tests.

The basis for specification of an airlock door seal leakage rate of 0.023 La is to provide assurance that the failure of a single airlock door will not result in the total containment leakage exceeding 0.6 La. The seven (7) day LCO specified for exceeding the airlock door leakage limit is acceptable since it requires that the total containment leakage limit is not exceeded.

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The limiting leakage rates from the shutdown cooling system are judgment values based primarily on assuring that the components could operate without mechanical failure for a period on the order of 200 days after a DBA. The test pressure (270 psig) achieved either by normal system operation or by hydrostatically testing gives an adequate margin over the highest pressure within the system after a DBA. Similarly, the hydrostatic test pressure for the return lines from the containment to the shutdown cooling system (100 psig) gives an adequate margin over the highest pressure within the lines after a DBA.⁽⁵⁾

A shutdown cooling system leakage of 1/5 gpm will limit off-site exposures due to leakage to insignificant levels relative to those calculated for leakage directly from the containment in the DBA. The engineered safeguards room ventilation system is equipped with isolation valves which close upon a high radiation signal from a local radiation detector. These monitors shall be set at

2.2×10^5 cpm, which is well below the expected level, following a loss-of-coolant accident (LOCA), even without clad failure. The 1/5 gpm leak rate is sufficiently high to permit prompt detection and to allow for reasonable leakage through the pump seals and valve packings, and yet small enough to be readily handled by the sumps and radioactive waste system. Leakage to the engineered safeguards room sumps will be returned to the containment clean water receiver following a LOCA, via the equipment drain tank and pumps. Additional makeup water to the containment sump inventory can be readily accommodated via the charging pumps from either the SIRW tank or the concentrated boric acid storage tanks.