

December 12, 1988

Docket No. 50-255

Mr. Kenneth W. Berry
Director, Nuclear Licensing
Consumers Power Company
1945 West Parnall Road
Jackson, Michigan 49201

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Dear Mr. Berry:

SUBJECT: AMENDMENT NO.119 TO PROVISIONAL OPERATING LICENSE NO. DPR-20:
STEAM GENERATOR OPERATING DIFFERENTIAL PRESSURE (TAC NO. 66901)

The Commission has issued the enclosed Amendment No. 119 to Provisional Operating License No. DPR-20 for the Palisades Plant. This amendment consists of changes to the Technical Specifications in response to your application dated October 7, 1988, which supersedes a portion of your application dated March 25, 1988.

This amendment deletes the steam generator differential pressure limit from the Technical Specifications.

A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

original signed by

Thomas V. Wambach, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosures:

1. Amendment No. 119 to License No. DPR-20
2. Safety Evaluation

cc w/enclosures:
See next page

<i>JW</i> LA/PD31: DRSP RIngram 11/23/88	<i>W</i> PM/PD31: DRSP Twambach: cr 11/23/88	<i>OGC</i> NRR/EMTB CCheng 12/1/88	<i>TW</i> D/PD31: DRSP TQuay 11/25/88	<i>OGC</i> OGC 12/1/88
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
December 12, 1988

Docket No. 50-255

Mr. Kenneth W. Berry
Director, Nuclear Licensing
Consumers Power Company
1945 West Parnall Road
Jackson, Michigan 49201

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Sincerely,

A handwritten signature in cursive script that reads "Thomas V. Wambach".

Thomas V. Wambach, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosures:

1. Amendment No. 119 to License No. DPR-20
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Kenneth W. Berry
Consumers Power Company

Palisades Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSUMERS POWER COMPANY

PALISADES PLANT

DOCKET NO. 50-255

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 119
License No. DPR-20

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consumers Power Company (the licensee) dated October 7, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Provisional Operating License No. DPR-20 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 119, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Acting Director
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 12, 1988

ATTACHMENT TO LICENSE AMENDMENT NO.119

PROVISIONAL OPERATING LICENSE NO. DPR-20

DOCKET NO. 50-255

Revise Appendix A Technical Specifications by removing the page identified below and inserting the attached page. The revised page is identified by the captioned amendment number and contains a marginal line indicating the area of change.

REMOVE

3-1b

INSERT

3-1b

3.1 PRIMARY COOLANT SYSTEM

Applicability

Applies to the operable status of the primary coolant system.

Objective

To specify certain conditions of the primary coolant system which must be met to assure safe reactor operation.

Specifications

3.1.1 Operable Components

- a. At least one primary coolant pump or one shutdown cooling pump with a flow rate greater than or equal to 2810 gpm shall be in operation whenever a change is being made in the boron concentration of the primary coolant and the plant is operating in cold shutdown or above, except during an emergency loss of coolant flow situation. Under these circumstances, the boron concentration may be increased with no primary coolant pumps or shutdown cooling pumps running.
- b. Four primary coolant pumps shall be in operation whenever the reactor is operated above hot shutdown, with the following exception:

Before removing a pump from service, thermal power shall be reduced as specified in Table 2.3.1 and appropriate corrective action implemented. With one pump out of service, return the pump to service within 12 hours (return to four-pump operation) or be in hot shutdown (or below) with the reactor tripped (from the C-06 panel, opening the 42-01 and 42-02 circuit breakers) within the next 12 hours. Start-up (above hot shutdown) with less than four pumps is not permitted and power operation with less than three pumps is not permitted.
- c. The measured four primary coolant pumps operating reactor vessel flow shall be 124.3×10^6 lb/hr or greater, when corrected to 532°F.
- d. Both steam generators shall be capable of performing their heat transfer function whenever the average temperature of the primary coolant is above 325°F.
- e. Maximum primary system pressure differentials shall not exceed the following:
 - (1) Deleted



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 119 TO PROVISIONAL OPERATING LICENSE NO. DPR-20

CONSUMERS POWER COMPANY

PALISADES PLANT

DOCKET NO. 50-255

INTRODUCTION

Consumers Power Company (the licensee) made application for a license amendment on October 7, 1988, to revise the Technical Specifications for the Palisades Plant to reflect the deletion of the steam generator differential pressure limit. The existing limit was used by Combustion Engineering to establish the current steam generator tube repair criteria in Technical Specification 4.14.4. Structural integrity of the steam generator tubes is assured by appropriate application of the specified repair criteria and limiting the primary system pressure in accordance with Technical Specification 3.1.1.f.

EVALUATION

The Palisades steam generator differential pressure limits of 1380 psid (operating) and 1530 psid (transient) were used by Combustion Engineering to establish steam generator tube repair criteria. The differential pressure is controlled by maintaining the primary system pressure at the nominal operating pressure of 2100 psia as specified in Technical Specification 3.1.1.f. The 1380 psid value continues to be maintained in plant procedure SOP-1, "Primary Coolant System", under plant requirements Section 4.0 so that operators are cognizant of this limit during system hydrostatic testing as specified in Specifications 3.1.1.e. (2) and (3). The transient pressure has been shown by ANF reanalysis of the loss of external load event, using the PTSPWR2 computer code, to be a calculated maximum of 1604.4 psi. Although the reanalyzed value of the transient differential pressure has increased, a large margin to tube failure exists. In addition, structural integrity of the steam generator tubes is assured by the implementation of the tube plugging or repair criteria contained in Technical Specification 4.14.4. It is further noted that the CE Standard Technical Specifications do not contain a steam generator differential pressure limit. Therefore, the NRC finds the deletion of the differential pressure limit as well as the recalculated maximum transient differential pressure acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously

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issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date:

Principal Contributors: H. F. Conrad
Eric R. Swanson
L. Kopp