

December 1, 1982

Docket No. 50-255
LS05-82-12-003

Mr. David J. Vandewalle
Nuclear Licensing Administrator
Consumers Power Company
1945 West Parnall Road
Jackson, Michigan 49201

Dear Mr. Vandewalle:

SUBJECT: PALISADES PLANT - RECIRCULATION HEAT REMOVAL SYSTEM TESTING

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The Commission has issued the enclosed Amendment No. 71 to Provisional Operating License No. DPR-20 for the Palisades Plant. This amendment consists of changes to the Technical Specifications in response to your application dated July 6, 1981, and is also in reference to your proposal dated November 12, 1981.

By submittal dated July 6, 1981, Consumers Power Company requested a change to Technical Specification 4.5.3.d relating to the frequency of performing leakage and hydrostatic testing of the Recirculation Heat Removal System. Presently, Specification 4.5.3.d requires this testing to be performed annually. Testing of this system for leakage also falls within the scope of Technical Specification 6.15 which requires testing of the system for leakage at a frequency not to exceed refueling cycle intervals. Hydrostatic testing of the system at 10 year intervals is the present requirement for this type system in Section XI of the ASME B&PV Code which is incorporated by reference in Technical Specification 4.3.

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You have also proposed a Technical Specification change by submittal dated November 12, 1981 which would also require leakage testing of this system at refueling intervals.

The existing requirement for annual testing given in Specification 4.5.3.d predates the requirements of Section XI of the ASME B&PV Code as well as the requirements of Specification 6.15 and your proposal of November 12, 1981, both of which are responsive to the NRC position given in NUREG-0737, Item III.D.1.1. The purpose of this position is to ensure as-low-as-practical levels of leakage from systems outside containment that could contain highly radioactive fluids during a serious transient or accident condition.

We have discussed with your representative modifications to your proposed Technical Specification change. The purpose of our modifications is to

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December 1, 1982

make the leakage test requirements consistent with those in Specification 6.15, NUREG-0737, and your proposed change of November 12, 1981. Requiring the leakage tests at refueling outage frequency is consistent with the present NRC requirements reflecting the post TMI lessons learned (NUREG-0737) and is acceptable.

With regard to the remaining parts of Specification 4.5.3 and Specification 6.15, we request that you review your submittal of November 12, 1981 and revise it as necessary so that it will replace the Specification 4.5.3 as changed by this amendment and Specification 6.15. This will consolidate the testing requirements for this system in one specification and reduce the likelihood of confusion or conflict.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

Original signed by/

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosures:

1. Amendment No. 71 to
License No. DPR-20

2. Notice of Issuance

OFFICE	DL:ORB#5	DL:ORB#5	DE:MTEB	OELO	DL:ORB#5
SURNAME	cc w/enclosures: See next page	H. M. Chais	J. Wambach	G. Johnson	D. Crutchfield
DATE	11/29/82	11/24/82	11/24/82	11/24/82	11/30/82

cc

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Consumers Power Company
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Jackson, Michigan 49201

Judd L. Bacon, Esquire
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Glen Ellyn, Illinois 60137

Township Supervisor
Covert Townshi
Route 1, Box 10
Van Buren County, Michigan 49043

Office of the Governor (2)
Room 1 - Capitol Building
Lansing, Michigan 48913

Palisades Plant
ATTN: Mr. Robert Montross
Plant Manager
Covert, Michigan 49043

U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: Regional Radiation Representative
230 South Dearborn Street
Chicago, Illinois 60604

Resident Inspector
c/o U. S. NRC
Palisades Plant
Route 2, P. O. Box 155
Covert, Michigan 49043



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSUMERS POWER COMPANY

DOCKET NO. 50-255

PALISADES PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 71
License No. DPR-20

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consumers Power Company (the licensee) dated July 6, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.


2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Provisional Operating License No. DPR-20 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B (Environmental Protection Plan), as revised through Amendment No. 71 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 1, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 71
PROVISIONAL OPERATING LICENSE NO. DPR-20
DOCKET NO. 50-255

Revise Appendix A Technical Specifications by removing the following pages and by inserting the enclosed pages. The revised pages contain the captioned amendment number and marginal lines indicating the area of change.

PAGES

4-28

4-29

4.5 CONTAINMENT TESTS (Contd)

- (2) If repairs are not completed and conformance to the acceptance criterion of 4.5.2.b is not demonstrated within 48 hours, the reactor shall be shut down and depressurized until repairs are effected and the local leakage meets this acceptance criterion.

d. Test Frequency

- (1) Individual penetrations and containment isolation valves shall be leak rate tested at a frequency of at least every six months prior to the first postoperational integrated leak rate test and at a frequency of at least every refueling thereafter, not exceeding a 2-year interval, except as specified in (a) and (b) below:
 - (a) The containment equipment hatch and the fuel transfer tube shall be tested at each refueling shutdown or after each time used, if that be sooner.
 - (b) The personnel air lock seals shall be tested at six-month intervals, except when the air locks are not opened during the interval. In that case, the test is to be performed after each opening, except that no test interval is to exceed twelve months.
- (2) Each three months the isolation valves must be stroked to the position required to fulfill their safety function unless it is established that such operation is not practical during plant operation. The latter valves shall be full-stroked during each cold shutdown.

4.5.3 Recirculation Heat Removal Systems

a. Test

- (1) The portion of the shutdown cooling system that is outside the containment shall be tested either by use in normal operation or hydrostatically tested at 255 psig at the interval specified in 6.15.
- (2) Piping from valves CV-3029 and CV-3030 to the discharge of the safety injection pumps and containment spray pumps shall be hydrostatically tested at no less than 100 psig at the interval specified in 6.15.

CONTAINMENT TESTS (Contd)

(3) Visual inspection shall be made for excessive leakage from components of the system at the interval specified in 6.15. Any significant leakage shall be measured by collection and weighing or by another equivalent method.

b. Acceptance Criterion

The maximum allowable leakage from the recirculation heat removal systems' components (which include valve stems, flanges and pump seals) shall not exceed 0.2 gallon per minute under the normal hydrostatic head from the SIRW tank (approximately 44 psig).

c. Corrective Action

Repairs shall be made as required to maintain leakage within the acceptance criterion of 4.5.3.b.

4.5.4 Surveillance for Prestressing System

- a. Tendon inspection shall be accomplished in accordance with the following schedule:
 1. One year after initial structural integrity test.
 2. Three years after initial structural integrity test.
 3. Five years after initial structural integrity test.
 4. At five-year intervals thereafter for the life of the plant.
- b. Surveillance tendons for the one-year inspection shall be the nine designated surveillance tendons plus V-104 and V-200. In addition, 15 vertical tendons shall be tested for lift-off forces only.
- c. For the three-year inspection, the surveillance tendons shall consist of the 11 tendons inspected during the one-year test plus an additional 10 vertical tendons to be tested for lift-off force only. The additional 10 tendons shall be selected from tendons other than those tendons tested for lift-off force during the one-year inspection.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-255CONSUMERS POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 71 to Provisional Operating License No. DPR-20, issued to Consumers Power Company (the licensee), which revised the Technical Specifications for operation of the Palisades Plant (the facility) located in Van Buren County, Michigan. This amendment is effective as of its date of issuance.

The amendment approves changes to the Technical Specifications to allow revised testing intervals for the Recirculation Heat Removal System.

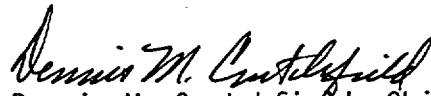
The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated July 6, 1981 and November 12, 1981, (2) Amendment No. 71 to License No. DPR-20, and (3) the Commission's letter of transmittal which contains its evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Kalamazoo Public Library, 315 South Rose Street, Kalamazoo, Michigan 49006. A single copy of items (2) and (3) may be obtained by request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 1st day of December, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing