

CP 51

December 21, 1982

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Docket No. 50-255
LS05-82-12-050

Mr. David J. Vandewalle
Nuclear Licensing Administrator
Consumers Power Company
1945 W. Parnall Road
Jackson, Michigan 49201

Dear Mr. Vandewalle:

SUBJECT: PALISADES PLANT - APPROVAL OF TECHNICAL SPECIFICATION CHANGES
SEP TOPICS III-7.C, V-10.B AND V-11.A

The Commission has issued the enclosed Amendment No. 72 to DPR-20 for the Palisades Plant. This amendment consists of changes to the Technical Specifications in response to your three separate applications dated July 29, 1982.

Our review related to your proposed technical specification changes is documented in the evaluations completed within the scope of the Systematic Evaluation Program (SEP) for dome delamination (SEP Topic III-7.C); RHR system reliability (SEP Topic V-10.B); and requirements for isolation of high and low pressure systems (SEP Topic V-11.A). These evaluations are dated May 21 and October 19, 1981; October 27 and December 23, 1981; and November 9, 1981, respectively. We have found that your proposed changes are consistent with these evaluations and are, therefore, acceptable.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

*SEP
DSA use(04)*

We have concluded, based on the considerations discussed in the SEP Topic evaluations identified above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an

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accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

The related Notice of Issuance is also enclosed.

Sincerely,

Original signed by/

Dennis M. Crutchfield, Chief
 Operating Reactors Branch #5
 Division of Licensing

Enclosures:

1. Amendment No. 72 to License No. DPR-20
2. Notice of Issuance

cc w/enclosures:
 See next page

*FR. R. NOTICE
 AMENDMENT
 12-14-82*

OFFICE	DL:ORB#5	DL:ORB#5	DL:ORB#5	OELD	DL:AO-SA		
SURNAME	HSmith:ajs	TWambach	DCrutchfield		FMTraglia		
DATE	12/9/82	12/10/82	12/10/82	12/11/82	12/21/82		

Mr. David J. Vandewalle

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December 21, 1982

cc

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Consumers Power Company
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Township Supervisor
Covert Townshi
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Van Buren County, Michigan 49043

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Lansing, Michigan 48913

Palisades Plant
ATTN: Mr. Robert Montross
Plant Manager
Covert, Michigan 49043

U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: Regional Radiation Representative
230 South Dearborn Street
Chicago, Illinois 60604

Resident Inspector
c/o U. S. NRC
Palisades Plant
Route 2, P. O. Box 155
Covert, Michigan 49043



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSUMERS POWER COMPANY

DOCKET NO. 50-255

PALISADES PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 72
License No. DPR-20

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications (3) for amendment by Consumers Power Company (the licensee) dated July 29, 1982 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Provisional Operating License No. DPR-20 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B (Environmental Protection Plan), as revised through Amendment No. 72, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 21, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 72

PROVISIONAL OPERATING LICENSE NO. DPR-20

DOCKET NO. 50-255

Revised Appendix A Technical Specifications by removing the following pages and by inserting the enclosed pages. The revised pages contain the captioned amendment number and marginal lines indicating the area of change.

Remove Pages

3-25a

4-17

4-18

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4-38

Insert Pages

3-25a

4-17*

4-18

4-18a

4-32a

4-38

*Included for pagination purposes only.

Overpressure Protection Systems Specifications

- a. When the temperature of one or more of the primary coolant system cold legs is $\leq 250^{\circ}\text{F}$, or whenever the shutdown cooling isolation valves (MOV-3015 and MOV-3016) are open, two power operated relief valves (PORVs) with a lift setting of ≤ 400 psia, or a reactor coolant system vent of ≥ 1.3 square inches shall be operable except as specified below:
- (1) With one PORV inoperable, either restore the inoperable PORV to operable status within 7 days or depressurize and vent the PCS through a ≥ 1.3 square inch vent(s) within the next 8 hours; maintain the PCS in a vented condition until both PORVs have been restored to operable status.
 - (2) With both PORVs inoperable, depressurize and vent the PCS through a ≥ 1.3 square inch vent(s) within 8 hours; maintain the PCS in a vented condition until both PORVs have been restored to operable status.
- b. In the event either the PORVs or the PCS vent(s) are used to mitigate a PCS pressure transient, a Special Report shall be prepared and submitted to the Commission within 30 days. The report shall describe the circumstances initiating the transient, the effect of the PORVs or vent(s) on the transient and any corrective action necessary to prevent recurrence.

Basis

The OPERABILITY of two PORVs or an PCS vent opening of greater than 1.4 square inches ensures that the PCS will be protected from pressure transients which could exceed the limits of Appendix G to 10 CFR Part 50 when one or more of the PCS cold legs are $\leq 250^{\circ}\text{F}$. Either PORV has adequate relieving capability to protect the PCS from overpressurization when the transient is limited to either (1) the start of an idle PCP with the secondard water temperature of the steam generator $\leq 70^{\circ}\text{F}$ above the PCS cold leg temperatures or (2) the start of a HPSI pump and its injection into a water solid PCS. (1)

Whenever the SCS is not isolated from the reactor coolant system, the PCS will be vented or the PORVs will be in service. This requirement will ensure that the overpressurization of the SCS that could lead to a loss-of-coolant accident outside containment is prevented.

References

- (1) "Palisades Plant Overpressurization Analysis," June, 1977 and "Palisades Plant Primary Coolant System Overpressurization Sub-system Description," October, 1977.
- (2) Systematic Evaluation Program Topic V-10.B, NRC letters to the licensee transmitting the final topic evaluation dated October 27 and December 23, 1981.

inspection techniques that have been proven practical, and the conclusions of the evaluation shall be used as appropriate to update the inspection program.

- f. Surveillance of the regenerative heat exchanger and primary coolant pump flywheels shall be performed as indicated in Table 4.3.2.
- g. A surveillance program to monitor radiation induced changes in the mechanical and impact properties of the reactor vessel materials shall be maintained as described in Section 4.5.3 of the FSAR. The specimen removal schedule shall be as indicated in Table 4.3.3.
- h. Periodic leakage testing (a), (b) on each check valve listed in Table 4.3.1 shall be accomplished prior to returning to the Power Operation Condition after every time the plant has been placed in the Refueling Shutdown Condition, or the Cold Shutdown Condition for more than 72 hours if such testing has not been accomplished within the previous 9 months, and prior to returning the check valves to service after maintenance, repair or replacement work is performed on the valves.
- i. Whenever integrity of a pressure isolation valve listed in Table 4.3.1 cannot be demonstrated and credit is being taken for

(NRC Order dated
April 20, 1981)

4-17

Amendment No. ~~53~~, 72

(a) To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria.

(b) Reduced pressure testing is acceptable (see footnote 5 to Table 4.3.1). Minimum test differential pressure shall not be less than 150 psid.

compliance with Specification 3.3.3.b, the integrity of the remaining check valve in each high-pressure line having a leaking valve shall be determined and recorded daily and the position of the other closed valve located in that pressure line shall be recorded daily.

- 4.3.j. Following each use of the LPSI system for shutdown cooling, the reactor shall not be made critical until the LPSI check valves (CK-3103, CK-3118, CK-3133 and CK-3148) have been verified closed.

Basis

The inspection program specified places major emphasis on the areas of highest stress concentration as determined by general design evaluation and experience with similar systems.⁽¹⁾ In addition, that portion of the reactor vessel shell welds which will be subjected to a fast neutron dose sufficient to change ductility properties will be inspected. The inspections will rely primarily on ultrasonic methods utilizing up-to-date analyzing equipment and trained personnel. Pre-operational inspections will establish base conditions by determining indications that might occur from geometrical or metallurgical sources and from discontinuities in weldments or plates which might cause undue concern on a postservice inspection. To the extent applicable, based upon the existing design and construction of the plant, the requirements of Section XI of the Code shall be complied with. Significant exceptions are detailed in the requests for relief which have received NRC approval and are contained in the Class 1, Class 2 and Class 3 Long-Term Inspection Plans.

Reactor Vessel Surveillance Specimens

Table 4.3.3 is consistent with the surveillance program as presented in the FSAR. (2) However, the withdrawal schedule has been modified to reflect the slightly different wall fluence values resulting from removal of the thermal shield.

Valve Testing

To ensure the continued integrity of selected check valves which are relied upon to preclude a potential LOCA outside containment, special requirements for periodic leak tests are specified. In addition a valve disk position check for the LPSI check valves is specified following each use of the LPSI system for shutdown cooling. This position check ensures that the four LPSI check valves have reclosed upon cessation of shutdown cooling flow.

References

- (1) FSAR, Section 4.5.6
- (2) FSAR, Section 4.5.3
- (3) Systematic Evaluation Program Topic V-11.A, NRC letter to the licensee transmitting the final topic evaluation dated November 9, 1981

4.5 CONTAINMENT TESTS (Con't)

4.5.8 Dome Delamination Surveillance

If, as a result of a prestressing system inspection under Section 4.5.4, corrective retensioning of five percent (8) or more of the total number of dome tendons is necessary to restore their liftoff forces to within the limits of Specification 4.5.4, a dome delamination inspection shall be performed within 90 days following such corrective retensioning. The results of this inspection shall be reported to the NRC.

CONTAINMENT TESTS (Con't)

two weeks (in extreme weather) during this two-year period, a visual inspection shall be made for stress indications. After this time, if no unexpected behavior of the liner plate or penetration assemblies is observed, the surveillance program will be extended at that time.

Containment dome delamination inspections performed in 1970 and 1982 have confirmed that no concrete delamination has occurred. The possibility that delamination might occur in the future is remote because dome tendon prestress forces gradually diminish through normal tendon relaxation and concrete strength normally increases over time. To account for this remote possibility, however, an additional delamination inspection will be performed in the event that 5% or more of the installed tendons must be retensioned to compensate for excessive loss of prestress. This inspection would be to confirm that any systematic excessive prestress loss did not result from delamination and that the retensioning process did not result in delamination.

References

- (1) FSAR, Section 5.1.2.
- (2) FSAR, Section 5.1.8.
- (3) FSAR, Section 14.22.
- (4) FSAR, Section 8.5.4.
- (5) FSAR, Section 6.2.3.
- (6) FSAR, Section 5.1.8.4.
- (7) FSAR, Amendment No. 14, Question 5.37.
- (8) 10 CFR Part 50, Appendix J.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-255CONSUMERS POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 72 to Provisional Operating License No. DPR-20, to Consumers Power Company (the licensee), which revised the Technical Specifications for operation of the Palisades Plant (the facility) located in Covert Township, Van Buren County, Michigan. The amendment is effective as of its date of issuance.

The amendment approves Technical Specification provisions pertaining to dome delamination (SEP Topic III-7.C); RHR system reliability (SEP Topic V-10.B); and requirements for isolation of high and low pressure systems (SEP Topic V-11.A).

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this action was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant

to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) three separate applications for amendment dated July 29, 1982, (2) Amendment No. 72 to License No. DPR-20, and (3) the Commission's Systematic Evaluation Program (SEP) Topic evaluations transmitted by letters to the licensee dated May 21 and October 19, 1981 (SEP Topic III-7.C); October 27 and December 23, 1981 (SEP Topic V-10.B); and November 9, 1981 (SEP Topic V-11.A). All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. 20555, and at the Kalamazoo Public Library, 315 South Rose Street, Kalamazoo, Michigan 49006. A copy of item (2) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 21st day of December, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing