

June 12, 1981

Docket No. 50-255
LS05-81-06-034

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Mr. David P. Hoffman
Nuclear Licensing Administrator
Consumers Power Company
1945 W. Parnall Road
Jackson, Michigan 49201

Dear Mr. Hoffman:



The Commission has issued the enclosed Amendment No. 66 to Provisional Operating License No. DPR-20 for the Palisades Plant. This amendment consists of changes to the Technical Specifications in response to your request dated May 26, 1981.

The amendment extends the surveillance interval for seven surveillance tests for varying periods up to 63 days. The maximum extension represents an increase of less than 15% in the period between surveillance tests.

During our review of your application, we found it necessary to modify your proposed license amendment by deleting 3 proposed extensions that were determined not to be necessary. We have discussed these changes with your representative and have mutually agreed upon them.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosures:

1. Amendment No. 66 to License No. DPR-20
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

8106260066

REGULATORY DOCKET FILE COPY

Notified Palisades Resident Inspector B. Jorgensen & CPCo Representative R. Houston that this amendment was signed. Pwot 10:17 6/12/81

Pwot to T.W.W.

6-10-81 DCrutch

[Handwritten signature]

| | | | | | | | |
|---------|--------------|------------|------|--------------|----------|--|--|
| OFFICE | DL: ORB #5 | DL: ORB #5 | OELD | DL: OR B#5 | DL: D/SA | | |
| SURNAME | PO'Connor:cd | HSmith | | DCrutchfield | GLathas | | |
| DATE | 6/10/81 | | | 6/12/81 | 6/12/81 | | |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 12, 1981

Docket No. 50-255
LS05-81-06-034

Mr. David P. Hoffman
Nuclear Licensing Administrator
Consumers Power Company
1945 W. Parnall Road
Jackson, Michigan 49201

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The amendment extends the surveillance interval for seven surveillance tests for varying periods up to 63 days. The maximum extension represents an increase of less than 15% in the period between surveillance tests.

During our review of your application, we found it necessary to modify your proposed license amendment by deleting 3 proposed extensions that were determined not to be necessary. We have discussed these changes with your representative and have mutually agreed upon them.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in black ink, appearing to read "Dennis M. Crutchfield".

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosures:

1. Amendment No. 66 to License No. DPR-20
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

Mr. David P. Hoffman

- 2 -

June 12, 1981

cc

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Van Buren County, Michigan 49043

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Room 1 - Capitol Building
Lansing, Michigan 48913

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(ANR-460)
U. S. Environmental Protection
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Washington, D. C. 20460

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Agency
Federal Activities Branch
Region V Office
ATTN: EIS COORDINATOR
230 South Dearborn Street
Chicago, Illinois 60604

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Atomic Safety and Licensing Board
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U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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Palisades Plant
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Plant Manager
Covert, Michigan 49043

William J. Scanlon, Esquire
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Ann Arbor, Michigan 48103



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSUMERS POWER COMPANY

DOCKET NO. 50-255

PALISADES PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 66
License No. DPR-20

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consumers Power Company (the licensee) dated May 26, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Provisional Operating License No. DPR-20 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B (Environmental Protection Plan), as revised through Amendment No. 66, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachments:
Changes to the Technical
Specifications

Date of Issuance: June 12, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 66

PROVISIONAL OPERATING LICENSE NO. DPR-20

DOCKET NO. 50-255

Revise Appendix A Technical Specifications by removing the following pages and by inserting the enclosed pages. The revised pages contain the captioned amendment number and marginal lines indicating the area of change.

REMOVE

4.3

4.4

4.6

4.11

4.29

INSERT

4.3

4.4

4.6

4.11

4.29

TABLE 4.1.1
Minimum Frequencies for Checks, Calibrations and Testing of Reactor Protective System⁽⁵⁾

| Channel Description | Surveillance Function | Frequency | Surveillance Method |
|--|-----------------------------|--|--|
| 1. Power Range Safety Channels | a. Check ⁽³⁾ | S | a. Comparison of four-power channel readings. |
| | b. Check | D | b. Channel adjustment to agree with heat balance calculation. Repeat whenever flux-ΔT power comparator alarms. |
| | c. Test | M ⁽²⁾ | c. Internal test signal. ⁽⁴⁾ |
| | d. Calibrate | R | d. Channel alignment through measurement/adjustment of internal test points. |
| 2. Wide-Range Logarithmic Neutron Monitors | a. Check | S | a. Comparison of both wide-range readings. |
| | b. Test | P | b. Internal test signal. |
| 3. Reactor Coolant Flow | a. Check | S | a. Comparison of four separate total flow indications. |
| | b. Calibrate ⁽⁶⁾ | R | b. Known differential pressure applied to sensors. |
| | c. Test | M ⁽²⁾ | c. Bistable trip tester. ⁽¹⁾⁽⁴⁾ |
| 4. Thermal Margin/Low Pressurizer Pressure | a. Check: | S | a. Check: |
| | (1) Temperature Input | | (1) Comparison of four separate calculated trip pressure set point indications. |
| | (2) Pressure Input | | (2) Comparison of four pressurizer pressure indications. (Same as 5(a) below.) |
| | b. Calibrate | R | b. Calibrate: |
| | (1) Temperature Input | | (1) Known resistance substituted for RTD coincident with known pressure input. |
| | (2) Pressure Input | | (2) Part of 5(b) below. |
| c. Test | M ⁽²⁾ | c. Bistable trip tester. ⁽¹⁾⁽⁴⁾ | |
| 5. High-Pressurizer Pressure | a. Check | S | a. Comparison of four separate pressure indications. |
| | b. Calibrate | R | b. Known pressure applied to sensors. |
| | c. Test | M ⁽²⁾ | c. Bistable trip tester. ⁽¹⁾ |

TABLE 4.1.1
Minimum Frequencies for Checks, Calibrations and Testing of Reactor Protective System⁽⁵⁾ (Contd)

| Channel Description | Surveillance Function | Frequency | Surveillance Method |
|---|-----------------------|------------------|---|
| 6. Steam Generator Level | a. Check | S | a. Comparison of four level indications per generator. |
| | b. Calibrate | R | b. Known differential pressure applied to sensors. |
| | c. Test | M ⁽²⁾ | c. Bistable trip tester. ⁽¹⁾ |
| 7. Steam Generator Pressure | a. Check | S. | a. Comparisons of four pressure indications (per generator. |
| | b. Calibrate | R ⁽²⁾ | b. Known pressure applied to sensors. |
| | c. Test | M ⁽²⁾ | c. Bistable trip tester. ⁽¹⁾ |
| 8. Containment Pressure | a. Calibrate | R ⁽²⁾ | a. Known pressure applied to sensors. |
| | b. Test | M ⁽²⁾ | b. Simulate pressure switch action. |
| 9. Loss of Load | a. Test | P | a. Manually trip turbine auto stop oil relays. |
| 10. Manual Trips | a. Test | P | a. Manually test both circuits. |
| 11. Reactor Protection System Logic Units | a. Test | M ⁽²⁾ | a. Internal test circuits. |

- Notes:
- (1) The bistable trip tester injects a signal into the bistable and provides a precision readout of the trip set point.
 - (2) All monthly tests will be done on only one of four channels at a time to prevent reactor trip.
 - (3) Adjust the nuclear gain pot on the ΔT cabinet until readout agrees with heat balance calculations.
 - (4) Trip setting for operating pump combination only. Settings for other than operating pump combinations must be tested during routine monthly testing performed when shut down and within four hours after resuming operation with a different pump combination if the setting for that combination has not been tested within the previous month.
 - (5) It is not necessary to perform the specified testing during prolonged periods in the regueling shutdown condition. If this occurs, omitted testing will be performed prior to returning the plant to service.
 - (6) The 1981 surveillance function may be deferred until the end of the 1981 refueling outage.

A-4

TABLE 4.1.2
Minimum Frequencies for Checks, Calibrations and Testing of
Engineered Safety Feature Instrumentation Controls

| Channel Description | Surveillance Function | Frequency | Surveillance Method |
|--|-----------------------|------------------|--|
| 1. Low-Pressure SIS Initiation Channels | a. Check | S | a. Comparison of four separate pressure indications. |
| | b. Test (1)(4) | R | b. Signal to meter relay adjust with test device to verify SIS actuation logic. |
| | c. Test | M ⁽²⁾ | c. Signal to meter relay adjusted with test device. |
| 2. Low-Pressure SIS Signal Block Permissive and Auto Reset | a. Test (1)(4) | R | a. Part of 1(b) above. |
| 3. SIS Actuation Relays | a. Test | Q | a. Simulation of SIS 2/4 logic trip using built-in testing system. Both "standby power" and "no standby power" circuits will be tested for left and right channels. Test will verify functioning of initiation circuits of all equipment normally operated by SIS signals. |
| | b. Test | R | b. Complete automatic test initiated by same method as Item 1(b) and including all normal automatic operations. |
| 4. Containment High-Pressure Channels | a. Calibrate | R | a. Known pressure applied to sensors. |
| | b. Test | R | b. Simulation of CHP 2/4 logic trip to verify actuation logic for SIS, containment isolation and containment spray |
| | c. Test | M ⁽²⁾ | c. Pressure switch operation simulated by opening or shorting terminals or pressure applied to the switch. |
| 5. Containment High Radiation Channels | a. Check | D | a. Comparison of four separate radiation level indications. |
| | b. Calibrate | R | b. Exposure to known external radiation source. |

TABLE 4.1.2
Minimum Frequencies for Checks, Calibrations and Testing of
Engineered Safety Feature Instrumentation Controls (Contd)

| Channel Description | Surveillance Function | Frequency | Surveillance Method |
|--|--------------------------|-----------|---|
| 13. Safety Injection Tank Level and Pressure Instruments | a. Check | S | a. Verify that level and pressure indication is between independent high high/low alarms for level and pressure. |
| | b. Calibrate | R | b. Known pressure and differential pressure applied to pressure and level sensors. |
| 14. Boric Acid Tank Level Switches | a. Test | R | a. Pump tank below low-level alarm point to verify switch operation. |
| 15. Boric Acid Heat Tracing System | a. Check | D | a. Observe temperature recorders for proper readings. |
| 16. Main Steam Isolation Valve Circuits | a. Check | S | a. Compare four independent pressure indications. |
| | b. Test ⁽³⁾ | R | b. Signal to meter relay adjusted with test device to verify MSIV circuit logic. |
| 17. SIRW Tank Temperature Indication and Alarms | a. Check | M | a. Compare independent temperature readouts. |
| | b. Calibrate | R | b. Known resistance applied to indicating loop. |
| 18. Low-Pressure Safety Injection Flow Control Valve CV-3006 | a. Check | P | a. Observe valve is open with air supply isolated. |
| 19. Safety Injection Bottle Isolation Valves | a. Check | P | a. Ensure each valve open by observing valve position indication and valve itself. Then lock open breakers (at MCC-9) and control power (key switch in control room). |
| 20. Safety Injection Miniflow Valves CV-3027, 3056 | a. Check | P | a. Verify valves open and HS-3027 and 3056 positioned to maintain them open. |

- Notes: (1) Calibration of the sensors is performed during calibration of Item 5(b), Table 4.1.1.
(2) All monthly tests will be done on only one channel at a time to prevent protection system actuation.
(3) Calibration of the sensors is performed during calibration of Item 7(b), Table 4.1.1.
(4) The 1981 surveillance function may be deferred until the end of the 1981 refueling outage.

TABLE 4.1.3

Minimum Frequencies for Checks, Calibrations and Testing of Miscellaneous Instrumentation and Controls (Contd)

| Channel Description | Surveillance Function | Frequency | Surveillance Method |
|--|-----------------------|-----------|---|
| 8. Control Rod Drive System Interlocks | a. Test | R | a. Verify proper operation of all rod drive control system interlocks, using simulated signals where necessary. |
| | b. Test | P | b. Same as 8(a) above, if not done within three months. |
| 9. Flux-AT Power Comparator | a. Calibrate | R | a. Use simulated signals. |
| | b. Test | M | b. Internal test signal. |
| 10. Calorimetric Instrumentation | a. Calibrate (2) | R | a. Known differential pressure applied to feed-water flow sensors. |
| 11. Containment Building Humidity Detectors | a. Test | R | a. Expose sensor to high humidity atmosphere. |
| 12. Interlocks - Isolation Valves on Shutdown Cooling Line | a. Calibrate | R | a. Known pressure applied to sensor. |
| 13. Service Water Break Detector in Containment | a. Test | R | a. Known differential pressure applied to sensors. |
| 14. Control Room Ventilation | a. Test | R | a. Check damper operation for DBA mode with HS-1801 and isolation signal. |
| | b. Test | R | b. Check control room for positive pressure. |

(1) During the 1978 refueling outage, Item 2.c will only be performed on 7 rods (1 per bank). The secondary rod position surveillance (Item 3.c) will be performed in entirety. Additionally, a 20" rod position check (comparing primary indication to secondary indication) will be conducted on each rod. If the primary and secondary indications vary more than 2" from each other, corrective action will be taken to restore the proper tolerances.

(2) The 1981 surveillance function may be deferred until the end of the 1981 refueling outage.

CONTAINMENT TESTS (Contd)

- (3) Visual inspection shall be made for excessive leakage from components of the system. Any significant leakage shall be measured by collection and weighing or by another equivalent method.
- b. Acceptance Criterion
The maximum allowable leakage from the recirculation heat removal systems' components (which include valve stems, flanges and pump seals) shall not exceed 0.2 gallon per minute under the normal hydrostatic head from the SIRW tank (approximately 44 psig).
- c. Corrective Action
Repairs shall be made as required to maintain leakage within the acceptance criterion of 4.5.3.b.
- d. Test Frequency
Tests of the recirculation heat removal system shall be conducted at intervals not to exceed twelve months.
- e. The 1981 recirculation heat removal system test may be deferred until the end of the 1981 refueling outage.

4.5.4 Surveillance for Prestressing System

- a. Tendon inspection shall be accomplished in accordance with the following schedule:
1. One year after initial structural integrity test.
 2. Three years after initial structural integrity test.
 3. Five years after initial structural integrity test.
 4. At five-year intervals thereafter for the life of the plant.
- b. Surveillance tendons for the one-year inspection shall be the nine designated surveillance tendons plus V-104 and V-200. In addition, 15 vertical tendons shall be tested for lift-off forces only.
- c. For the three-year inspection, the surveillance tendons shall consist of the 11 tendons inspected during the one-year test plus an additional 10 vertical tendons to be tested for lift-off force only. The additional 10 tendons shall be selected from tendons other than those tendons tested for lift-off force during the one-year inspection.

4.7 EMERGENCY POWER SYSTEM PERIODIC TESTS (Contd)

b. Every three months, the specific gravity of each cell, the temperature reading of every fifth cell, the height of electrolyte, and the amount of water added shall be measured and recorded.

4.7.3 Emergency Lighting

The correct functioning of the emergency lighting system shall be verified at least once each year. The 1981 verification may be deferred until the end of the 1981 refueling outage.

Basis

The emergency power system provides power requirements for the engineered safety features in the event of a DBA. Each of the two diesel generators is capable of supplying minimum required safeguards equipment from independent buses. (1, 2) This redundancy is a factor in establishing testing intervals. The monthly tests specified above will demonstrate operability and load capacity of the diesel generator. The fuel supply and various controls are continuously monitored and alarmed for abnormal conditions. Starting on complete loss of off-site power will be verified by simulated loss-of-power tests at approximately yearly intervals (during refueling shutdowns). Considering system redundancy, the specified testing intervals for the station batteries should be adequate to detect and correct any malfunction before it can result in system malfunction. Batteries will deteriorate with time, but precipitous failure is extremely unlikely. The surveillance specified is that which has been demonstrated over the years to provide an indication of a cell becoming unserviceable long before it fails.

References

- (1) FSAR, Section 8.4.1.
- (2) FSAR, Section 8.5.2.2.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 66 TO PROVISIONAL OPERATING LICENSE NO. DPR-20

CONSUMERS POWER COMPANY

PALISADES PLANT

DOCKET NO. 50-255

1.0 INTRODUCTION

By letter dated May 26, 1981 Consumers Power Company (CPC) (the licensee) requested that the surveillance interval between required surveillance tests on certain systems and instruments be extended. The extension was requested to permit the reactor to complete the 57 days remaining between the first scheduled surveillance test and the beginning of the scheduled 1981 refueling outage.

2.0 BACKGROUND

A large number of surveillance tests are required to assure the operational readiness of reactor instruments and components. These tests are generally scheduled for surveillance at intervals determined by the expected performance of the system and the ability of the instruments to remain within specified tolerances.

Many tests, particularly those on systems with proven reliability that cannot be tested with the reactor in operation, are scheduled to be carried out annually or during the periodic refueling outages.

In order to bring about consistency, standardization and enforceability in the specifications issued by the NRC, the surveillance interval for these tests has customarily been specified as 12 months and 18 months for the annual and refueling tests.

3.0 EVALUATION

The 1979-1980 refueling outage at Palisades was extended by approximately 6 months to complete modifications required by NRC. Many of the refueling and annual surveillance requirements were satisfied near the beginning of that outage. Because of the length of the outage some of the surveillance requirements have come due as early as June 1981 although less than 12 months of operation has elapsed. These surveillance requirements cannot be satisfied without shutting the reactor down to gain access to the components.

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We have reviewed the requested extensions and have determined that Test RO-28 Control Room Ventilation and Test RO-13 Normal Shut Down Sequencer can be carried out without shutting the reactor down and without unnecessary radiation exposure to operating personnel. Therefore, there is no basis for deferral of these tests. The licensee has agreed to conduct these two surveillance tests on schedule. We have further determined that Test RR-10 Area Monitor Calibration is not required prior to the scheduled August 8, 1981 shutdown for refueling and therefore does not require an extension.

We have reviewed the licensee's request for extending the interval for performing the other seven surveillance tests and have concluded that the requested extension results in an insignificant increase of less than 15% in the allowed time between testing. We have reviewed the performance of this equipment in the previous surveillance test and determined that none of the systems or components proposed for extension failed the previous test.

The Palisades Technical Specifications require that if a surveillance interval is lengthened beyond the nominal value specified, that the next two surveillance intervals be shortened to assure that the combined interval time for any three consecutive surveillance intervals not exceed 3.25 times the specified surveillance interval.

Based upon the above evaluation we have concluded that the extension of the surveillance interval for these seven tests will have an insignificant impact on the safe operation of the plant and the subsequent shortening of future surveillance intervals will assure that an equivalent level of safety will be maintained in the future. Therefore, the proposed extension is acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluents types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

We have concluded, based on the consideration discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accident previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: June 12, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-255CONSUMERS POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 66 to Provisional Operating License No. DPR-20, issued to Consumers Power Company (the licensee), which revised the Technical Specifications for operation of the Palisades Plant (the facility) located in Covert Township, Van Buren County, Michigan. The amendment is effective as of its date of issuance.

The amendment extends the surveillance interval for seven surveillance tests for varying periods up to 63 days. The maximum extension represents an increase of less than 15% in the period between surveillance tests.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this action was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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- 2 -

For further details with respect to this action, see (1) the application for amendment dated May 26, 1981, (2) Amendment No. 66 to License No. DPR-20, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room 1717 H Street, N. W., Washington, D. C. 20555, and at the Kalamazoo Public Library, 315 South Rose Street, Kalamazoo, Michigan 49006. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C., 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 12th day of June, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard P. Snaider, Acting Chief
Operating Reactors Branch #5
Division of Licensing