

MAR 28 1975

Docket No. 50-255

Consumers Power Company  
ATTN: Mr. R. C. Youngdahl  
Senior Vice President  
212 West Michigan Avenue  
Jackson, Michigan 49201

Gentlemen:

This is in response to your letter of March 10, 1975, in which you have reported the results of the Palisades Plant steam generator tube inspection conducted in February 1975, as required by our Order for Modification of License dated February 6, 1975. This report by you indicates compliance with the provisions of this Order. As provided therein, you are hereby authorized to resume operation of the Palisades Plant provided that the plant is otherwise ready to resume operation within the limits of the license and associated Technical Specifications, including Technical Specification Change No. 17 (Amendment No. 13 to Facility License No. DPR-20) enclosed with this letter.

The report you provided dated March 10, 1975, indicates that additional tubes in both steam generators will be plugged, with a consequent reduction in reactor coolant vessel flow. Accordingly, the enclosed amendment relates to the allowable reactor power level as a function of measured vessel flow, as well as to your request for change to the schedule for the next steam generator tube inspection. The details of these changes and the bases therefore are contained in the enclosed amendment and related Safety Evaluation. A copy of the Federal Register Notice is also enclosed.

The enclosed amendment limits operation to power levels not exceeding 2100 Mwt. Prior to proposed operation at any power level in excess of 2100 Mwt, we would require the following information for our review:

- (a) Flow measurement data ~~resulting~~ from the startup and operation at the 2100 Mwt level.
- (b) A revised ECCS LOCA analysis based on operation at the power level and minimum flow rates which would be requested. This analysis should include an assessment of the effects of the unbalanced and additional flow resistance due to plugged tubes in the two steam generators.

*J.P.*  
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MAR 28 1975

- (c) Demonstration of the continued validity of the existing safety analysis for all anticipated transients and postulated accidents at the requested power/flow operating limits or the submittal of a revised analysis.

We consider this amendment and the events that have transpired since December 1974 to have made moot your requests for amendment dated December 13, 1974, and January 23, 1975, wherein at that time you had requested different steam generator tube inspection intervals and procedures. Accordingly, we consider these requests to be no longer pending before us for action.

Sincerely,

Original Signed by  
A. Giambusso

A. Giambusso, Director  
Division of Reactor Licensing  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 13
- 2. Safety Evaluation
- 3. Federal Register Notice

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- (b) A revised ECCS LOCA analysis based on operation at the power level and minimum flow rates which would be requested. This analysis should include an assessment of the effects of the unbalanced and additional flow resistance due to plugged tubes in the two steam generators.
- (c) Justification of the existing safety analysis for all anticipated transients and postulated accidents at the requested power/flow operating limits or the submittal of a revised analysis.

We consider this amendment and the events that have transpired since December 1974 to have made moot your requests for amendment dated December 13, 1974, and January 23, 1975, wherein at that time you had requested different steam generator tube inspection intervals and procedures. Accordingly, we consider these requests to be no longer pending before us for action.

Sincerely,

A. Giambusso, Director  
 Division of Reactor Licensing  
 Office of Nuclear Reactor Regulation

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cc w/enclosures:  
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- (b) A revised ECCS LOCA analysis based on operation at the power level and minimum flow rates which would be requested. This analysis should include an assessment of the effects of the unbalanced and additional flow resistance due to plugged tubes in the two steam generators.
- (c) Justification of the existing safety analysis for all anticipated transients and postulated accidents at the requested power/flow operating limits or the submittal of a revised analysis.

We consider this amendment and the events that have transpired since December 1974 to have mooted your requests for amendment dated December 13, 1974, and January 23, 1975, wherein at that time you had requested different steam generator tube inspection intervals and procedures. Accordingly, we consider these requests to be no longer pending before us for action.

Sincerely,

A. Giambusso, Director  
 Division of Reactor Licensing  
 Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 13
- 2. Safety Evaluation
- 3. Federal Register Notice

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- Docket File ACRS (14)
- OR-1 Reading
- NRC PDR
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- Attorney, OELD
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- TJCarter
- PColins
- SVarga
- CHebron

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 See next page

bcc: J. R. Buchanan  
 T. B. Abernathy

*NO LEGAL OBJECTION  
 DEC*

*VS.*  
*[Signature]*  
 TR:RSB  
*[Signature]*  
 3/26/75

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SURNAME →	CTrammell:lb	RAPurple	Dkatalic	KRGoller	AGiambusso	[Signature]
DATE →	3/25/75	3/25/75	3/28/75	3/28/75	3/ /75	3/26/75

MAR 28 1975

cc w/enclosures:

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J. L. Bacon, Esquire  
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Jackson, Michigan 49201

Paul A. Perry, Secretary  
Consumers Power Company  
212 West Michigan Avenue  
Jackson, Michigan 49201

Kalamazoo Public Library  
315 South Rose Street  
Kalamazoo, Michigan 49006

Mr. Jerry Sarno  
Township Supervisor  
Covert County  
Route 1, Box 10  
Van Buren County, Michigan 49043

cc w/enclosures & incoming:  
Mr. John D. Beck (2 cys)  
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Executive Office of the Governor  
Lewis Cass Building, 2nd Floor  
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Mr. Gary Williams  
Environmental Protection Agency  
Federal Activities Branch  
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Chicago, Illinois 60606

Myron M. Cherry, Esquire  
Suite 4501  
One IBM Plaza  
Chicago, Illinois 60611

CONSUMERS POWER COMPANY

DOCKET NO. 50-255

PALISADES PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 13  
License No. DPR-20

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Consumers Power Company (the licensee) dated March 10, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended, (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Facility License No. DPR-20 is hereby amended to read as follows:

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**"B. Technical Specifications**

The Technical Specifications contained in Appendices A, B, and C, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 17."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by  
A. Giambusso

A. Giambusso, Director  
Division of Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Change No. 17 to Technical  
Specifications

Date of Issuance: MAR 28 1975

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ATTACHMENT TO LICENSE AMENDMENT NO. 13  
CHANGE NO. 17 TO THE TECHNICAL SPECIFICATIONS  
PROVISIONAL OPERATING LICENSE NO. DPR-20  
DOCKET NO. 50-255

Revise Appendices A and C as follows:

Remove pages 4-65 and C-1 and insert identically numbered new pages. Add new page C-2.

4.14 Augmented In-Service Inspection Program for Steam Generators

Applicability

Applies to the tubes within both steam generators.

Objective

To provide assurance of continued integrity of the steam generator tubes over their service lifetime.

Specification

4.14.1 At the end of no more than 90 effective full power days or no more than nine calendar months from the date of initial criticality after February 1975, whichever occurs first, an inspection of the steam generator tubes shall be conducted in accordance with the requirements of Regulatory Guide 1.83, "In-Service Inspection of Pressurized Water Reactor Steam Generator Tubes," (issued June 1974) as it applies to inspections after the baseline inspection. | 17

4.14.2 Subsequent inspections will be made thereafter at a frequency no less than the inspection intervals specified in Regulatory Guide 1.83 (issued June 1974). Tube sampling requirements will be as identified in Regulatory Guide 1.83 (issued June 1974). | 17

4.14.3 Any steam generator tubes with eddy current indications of 50% or more wall thinning shall be removed from service by plugging. Such indications may be confirmed by averaging during a given inspection, but such average shall be based on not less than three readings, in which case an average indication of 50% or more wall thinning shall result in tube plugging. | Order  
Feb.6,  
1975

INTERIM TECHNICAL SPECIFICATIONS  
FOR OPERATION OF THE PALISADES PLANT

DOCKET NO. 50-255

The specifications of this Appendix have precedence over requirements in Appendices A and B.

1. Maximum thermal power level of the reactor shall be as follows:<sup>(1)</sup>
  - (a) For a measured reactor coolant vessel flow<sup>(2)</sup> of  $125.0 \times 10^6$  lb/hr or greater (4 reactor coolant pumps operating), the reactor may be operated at power levels up to 2100 MWt.
  - (b) In the event that measured reactor vessel flow is less than  $125.0 \times 10^6$  lb/hr, the reactor shall not be operated pending analysis and justification for resuming power operation.
  - (c) For three-pump operation, the high power trip set point shall be 25% (550 MWt) of rated power.
  - (d) Reactor operation with less than three pumps operating is prohibited.
  
2. The following specifications apply to facility operations for the purpose of removing chemical impurities from the steam generators to maintain the steam generators in a serviceable condition.
  - (a) The facility shall be operated in keeping with the requirements of the steam generator flushing program proposed by the licensee's letter and attachments dated August 20, 1974, from R. C. Youngdahl to the Division of Reactor Licensing. At the conclusion of the steam generator flushing program, the plant may return to normal operation.
  - (b) Monthly reports on the steam generator water chemistry shall be submitted to the Regional Director, Office of Inspection and Enforcement and the Division of Reactor Licensing. These reports shall contain a graph or plot showing the following as a function of time for both steam generators:

<sup>(1)</sup> These interim restrictions, particularly 1(c) and (d), are subject to revision following actual flow measurement with various pump combinations and further supportive safety analysis, including calculation of flow and high power trip settings.

<sup>(2)</sup> As determined by reactor coolant pump differential pressure and pump performance curves.

MAR 28 1975

- (1) Phosphate and sulfate concentration,
- (2) Conductivity ( $\mu$ mhos/cm),
- (3) pH, and
- (4) Weight of phosphate and sulfate removed.

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These reports shall be submitted until the next scheduled steam generator tube inspection following that conducted in February 1975.

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 13 TO LICENSE NO. DPR-20

(CHANGE NO. 17 TO TECHNICAL SPECIFICATIONS)

CONSUMERS POWER COMPANY

PALISADES PLANT

DOCKET NO. 50-255

Introduction

By letter dated March 10, 1975, Consumers Power Company (the licensee) reported the results of a steam generator tube inspection program which was conducted pursuant to an Order for Modification of License issued by us on February 6, 1975 (40 FR 6241). These results indicate that a total of 281 steam generator tubes in both steam generators have localized thinning wherein 50% or less of the original tube material remains, and therefore these tubes must be removed from service by plugging. The plugging of the steam generator tubes results in a reduced primary side flow area through the steam generators and hence a slight reduction in reactor coolant flow rate.

The Order further required the licensee to furnish a proposal for the conduct of future operations, including a recommended schedule for the next steam generator tube inspection. Proposed technical specifications relating to this subject were included with the licensee's report of March 10, 1975.

This Safety Evaluation, therefore, addresses the effect of the reduced primary side flow rate as it relates to allowable reactor power level, and the proposed steam generator tube inservice inspection program requested by the licensee.

Evaluation

1. Effect of Plugging of Steam Generator Tubes on Primary Side Flow

The reactor vessel flow used in the current Palisades Fuel Densification Analysis for rated power (2200 MWt) is  $127.3 \times 10^6$  lb/hr. Allowing 3% for possible instrument error, the measured vessel flow must equal or exceed  $131.2 \times 10^6$  lb/hr in order to permit the reactor to be operated at rated power.



By letter of March 10, 1975, the licensee states that reactor vessel flow was determined to be  $131.3 \times 10^6$  lb/hr in August 1974. Plugging 281 tubes (out of a total of approximately 13,000 unplugged tubes in both steam generators) will reduce this value slightly. The licensee estimates this reduction will be  $1.1 \times 10^6$  lb/hr (0.84%) with a resulting reactor vessel coolant flow of  $130.2 \times 10^6$  lb/hr. This predicted flow value therefore falls slightly below the indicated flow necessary for rated power operation, and a corresponding slight power reduction would be necessary to maintain the safety margins previously existing.

The licensee has stated that, based on an analysis furnished on July 22, 1974, the presently existing overpower thermal margin of 122% can be preserved for a vessel flow of  $130.2 \times 10^6$  lb/hr by restricting thermal power to 2183 MWt or 99.2% of rated power. To provide for added safety margin pending completion of analyses that would support full power operation, the licensee has proposed to limit power to 2100 MWt (95.5% of rated power).

We have independently assessed the impact of the flow reduction on permissible reactor power level as it is related to safe operation of the facility, and conclude that limiting reactor power level to 2100 MWt is conservative and will preserve the 122% overpower thermal margin previously existing and approved by us, provided that measured reactor vessel flow is equal to or greater than  $125.0 \times 10^6$  lb/hr. Accordingly, the Technical Specifications associated with this amendment would allow reactor operation at power levels up to 2100 MWt provided that measured flow equals or exceeds this value.

To ensure that thermal margins are not reduced below those previously evaluated and approved, we have prohibited power operation if the measured vessel flow is less than  $125.0 \times 10^6$  lb/hr (4 reactor coolant pumps operating). In that unlikely event, the Tech. Specs. accompanying this amendment require the licensee to submit further analysis and justification for resuming power operation.

As a result of previously discovered discrepancies concerning the calculation of trip settings for the 2 and 3 pump combinations,<sup>(1)</sup> the licensee is currently using the 2-pump high power trip settings for 3-pump operation, and not conducting reactor operation with only 2 reactor coolant pumps running. Operation with three reactor coolant pumps at the 2-pump high power trip settings limits reactor power level to a maximum of

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<sup>(1)</sup> Abnormal Occurrence Report AO-22-74, October 18, 1974

25% with a reactor coolant flow of about 75%. We conclude that operation in this conservative manner more than adequately accounts for both the trip setting discrepancies and the slight flow reduction that is expected as a result of the tube plugging operation previously described, and have included these limitations in the Tech. Specs. as an interim measure, pending measurement of flow for these pump combinations and receipt of additional information from the licensee concerning this mode of operation.

We have reviewed all anticipated transients and postulated accidents to determine those that are affected by changes in primary system coolant flow rate. We find that by reducing maximum power so as to preserve the same overpower thermal margin, there is no significant reduction in the margin of safety and no significant increase in the probability or consequences of anticipated transients or postulated accidents, and that therefore this change does not involve a significant hazards consideration.

## 2. Proposed Steam Generator Tube Inspection

In February 1975, the licensee conducted a 100% eddy-current inspection of the inlet-side tubes of both steam generators; a 10% sample of the outlet-side tubes of the "A" steam generator; and a 3% sample of the outlet-side tubes of the "B" steam generator. This inspection involved approximately 14,000 separate measurements.

The results reported by the licensee in the March 10, 1975 letter showed 281 tubes in both steam generators with eddy current indications (localized thinning) of 50% or greater; all but two of these occurred in the steam generator inlet-side tubes.

Prior to August 1974, the licensee had conducted a similar steam generator tube inspection, and plugged all tubes with indications of 50% thinning or more. A conversion to all-volatile chemistry was proposed, combined with special procedures to flush the corrodant phosphate and sulfur compounds previously used for secondary water chemistry control from the steam generators. These special procedures included power changes and various power levels to produce mixing and fluid agitation in order to remove the chemical corrodants.

Since criticality on September 5, 1974, the plant has experienced only limited power operation due to turbine and condenser problems. Only seven effective full power days (EFPD) has been achieved, with brief periods of operation at the 50 and 60% power levels. The licensee has been able to control the concentration of these chemicals at low levels in the secondary water by blowdown while at elevated temperatures. However, the extensive flushing program which includes power operation for an extended period with its attendant mixing and agitation is not complete. In fact, the amount of chemicals and sludge deposits still remaining may be considerable. This is evidenced by an increasing concentration of these chemicals while in the cold condition when it is not possible to flush at high rates.

Thus, although the licensee is adding all-volatile chemicals to the secondary water, it would be premature to call the conversion to all-volatile chemistry complete since there is evidence that a residual of the other treatment chemicals remain at this time. Therefore, occasional phosphate and/or sulfate excursions may occur during the next period of operation, with attendant tube degradation potential. Furthermore, the results of this recent inspection indicate that some additional tube wall thinning has occurred in localized areas, and that tube degradation has not as yet been arrested during the water chemistry transition from phosphate to all-volatile treatment.

Thus, the conditions which exist at this time are similar to those existing in August 1974: all tubes with indications of 50% thinning or over have been plugged; a flushing program will be conducted to remove the corrodant chemicals which may still remain.

On August 30, 1974, we issued a license amendment (No. 10) for the Palisades Plant which required that the steam generators be inspected at the end of not more than 90 EFPD or 6 calendar months following the startup in September 1974. As discussed previously, the plant operated for only a brief period following the September 1974 startup and therefore the calendar time provision governed the inspection schedule. In recognition of this limited operating time, the licensee proposed (and conducted) a selective examination of about 500 tubes in the "A" steam generator on December 13, 1974, concentrating the inspection effort on the tubes which previous inspections (the latest at that time was June 1974) had disclosed the greatest wall thinning. The "A" steam generator was selected since it had experienced more extensive tube wall thinning than the (other) "B" steam generator. Based on the results of this inspection, which disclosed 27 tubes with localized wall thinning of 50% or greater from the relatively limited sample taken, we concluded that a more extensive inspection was warranted. Our Order for Modification of License of February 6, 1975 (mentioned previously) implemented these requirements.

The existing Tech Specs would require that the next steam generator tube inspection be conducted at the next refueling (approximately September 1975). The licensee now proposes a change to the Tech Specs which would require the next inspection be conducted not at the next refueling but at the end of not more than 90 EFPD following the February 1975 inspection.

We agree that reinspection should occur after not more than 90 EFPD. This inspection interval is the same as that incorporated into the license by Amendment No. 10 on August 30, 1974. Since the licensee has achieved only 7 EFPD of operation since that time, this change is therefore a continuation of that requirement, rather than a revision of the inspection schedule. Because of this limited operation, there is no substantial operating experience to support any revision to this schedule, and our original bases for this relatively short inspection interval remain unchanged at this time.

The volatile water chemistry treatment should substantially reduce, and may completely arrest, the tube wastage phenomena previously experienced. The next inspection could therefore serve to confirm the efficacy of this water treatment method.

In addition to the 90 EFPD (maximum) inspection interval, we conclude that a calendar time restriction for the next inspection should be required. This is based on the fact that only limited power operation has taken place between the last two major inspections, (approximately 9 months apart) and the latest inspection results indicate that wastage may be continuing during lay-up periods.

We therefore have required the next inspection be made at the end of not more than 90 EFPD or 9 calendar months, whichever occurs first. This inspection interval is shorter than that (12-20 months) recommended in Regulatory Guide 1.83 (issued June 1974); the inspection methods will fully conform to this Guide.

In view of the December 1974 and February 1975 eddy-current test results and the possibility that further tube degradation may occur until the new all-volatile chemistry is stabilized, we have continued the requirement that the licensee submit monthly water chemistry reports to us, and added the requirement that they contain a graph or plot of the following as a function of time for both steam generators:

- a. Phosphate and sulfate concentration,
- b. conductivity ( $\mu$ mhos/cm),
- c. pH, and
- d. Weight of phosphate and sulfate removed.

These reports will be required until the next scheduled steam generator tube inspection.

In addition to the above changes, several editorial changes to Section 4.14 of the Technical Specifications have been made. Paragraph 4.14.1, relating to a requirement for a primary system leak test prior to initial criticality after August 1, 1974, has been deleted since this test was satisfactorily performed and therefore now obsolete. Paragraph 4.14.4 was added by our Order for Modification of License of February 6, 1975. This paragraph required additional steam generator tube inspections which have since been completed. Therefore this paragraph has been similarly deleted since it is now also obsolete. The tube plugging criterion promulgated by this Order as paragraph 4.14.5 has been included as new paragraph 4.14.3. Other paragraphs of this section have been appropriately renumbered.

We have concluded that this inspection program will provide an acceptable means to verify that steam generator tube corrosion has been arrested, or to identify any additional corrosion, should it occur, in a timely manner, and that the Palisades Plant can be safely operated in the 90 EFPD or 9 calendar month interval before the next inspection.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 28, 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-255

CONSUMERS POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL  
OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 13 to Provisional Operating License No. DPR-20 issued to Consumers Power Company which revised Technical Specifications for operation of the Palisades Plant, located in Covert Township, Van Buren County, Michigan. The amendment is effective as of its date of issuance.

This amendment requires the next steam generator tube inspection to be conducted at the end of not more than 90 effective full power days or more than nine calendar months from the date of initial criticality after February 1975, whichever occurs first. The amendment also imposes limits on reactor power level and reactor coolant minimum flow for various reactor coolant pump combinations, with an overall power limitation of 2100 MWt. In addition, the amendment revises Section 4.14 of the Technical Specifications by deleting paragraphs 4.14.1 and 4.14.4, two one-time requirements which have been satisfied. The remaining paragraphs of Specification 4.14 are renumbered.

The amendment relates to two prior FEDERAL REGISTER notices. On February 3, 1975, 40 FR 4968, the Commission published in the FEDERAL REGISTER a notice of consideration of a license amendment proposed by the license which would have eliminated the requirement, in the license at that time,

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for a steam generator tube inspection within six months from the date of initial criticality after the August 1974 shutdown. The proposed amendment would have preserved, however, the alternative requirement, also in the license at that time, for such an inspection at the end of no more than 90 effective full power days following the August 1974 shutdown. No request for hearing or petition for leave to intervene was filed in response to the February 3, 1975 notice.

On February 6, 1975, the Commission issued an Order for Modification of License which was published in the FEDERAL REGISTER on February 10, 1975, 40 FR 6241. This Order required Consumers Power Company to follow certain specified tube inspection and plugging procedures prior to further reactor operation, and to submit the results thereof together with a proposal for the conduct of future operations, including a recommended schedule for the next steam generator tube inspection. No request for a hearing or petition for leave to intervene was filed in response to the February 6, 1975 Order.

As discussed in the Commission's Safety Evaluation related to this action, the licensee has complied with the provisions of the February 6 Order. Extensive steam generator tube inspections have been conducted and approximately 280 additional tubes have been plugged. The additional tube plugging is expected to result in a reduction of reactor coolant flow of approximately 1%. The new limits on reactor power level and reactor coolant minimum flow have been specified in order to take into account the anticipated reduction of coolant flow.

The amendment will allow a maximum interval of nine months prior to the next steam generator tube inspection. This is about the same interval

effectively allowed between the last two inspections, which occurred in May-June 1974 and February-March 1975, respectively.

The Commission has found that the amendment involves no significant hazards consideration not described or implicit in the FEDERAL REGISTER notice of February 3, 1975, the Order published in the FEDERAL REGISTER on February 10, 1975, or both.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

For further details with respect to this action, see (1) the application for amendment dated March 10, 1975, (2) Amendment No. 13 to License No. DPR-20, with Change No. 17, (3) the Commission's related Safety Evaluation, and (4) the February 3 and February 10, 1975 FEDERAL REGISTER notices and documents referenced therein. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C., and at the Kalamazoo Public Library, 315 South Rose Street, Kalamazoo, Michigan.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this **MAR 28 1975**

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:  
Robert A. Purple

Robert A. Purple, Chief  
Operating Reactors Branch #1  
Division of Reactor Licensing

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SURNAME						
DATE						

CHECKLIST FOR ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

APPLICANT Consumers Power Company DOCKET NO. 50-255

FACILITY Palisades Plant

PROJECT MANAGER Charles Trammell

LICENSING ASSISTANT Shirley Sheppard

DATE

Notice of Proposed Issuance Published  
In FEDERAL REGISTER  
~~Action Date~~ ORDER for Modification

2/3/75 (40-FR-4968)  
2/10/75 (40-FR-6241)

Issuance Package: ELD Concurrence

1. License Amendment

3/28/75

2. FEDERAL REGISTER Notice

"

3. Staff Evaluation

"

4. Letter to applicant

"

NEPA Determination:

Required/Not Required

not required

For Amendments Affecting Power Level:

IE Notification and/or Concurrence

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OAI Notification and/or Concurrence 1/

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ADM Ofc. Notification and/or  
Concurrence

\_\_\_\_\_

PA Notification

\_\_\_\_\_

1/ or name change, transfer of facility ownership

FEB 7 1975