

December 30, 1998

Mr. J. A. Scalice  
Chief Nuclear Officer and  
Executive Vice President  
Tennessee Valley Authority  
6A Lookout Place  
1101 Market Street  
Chattanooga, Tennessee 37402-2801

SUBJECT: ISSUANCE OF AMENDMENT ON SLAVE RELAY SURVEILLANCE INTERVAL  
(TAC NO. M94425)

Dear Mr. Scalice:

The Commission has issued the enclosed Amendment No. 17 to Facility Operating License No. NPF-90 for Watts Bar Nuclear Plant, Unit 1. This amendment is in response to your application dated February 28, 1996, as supplemented October 2 and December 12, 1997, March 30 and December 11, 1998. The February 28, 1996 letter proposed to extend the surveillance interval for Westinghouse type AR relays with alternating current and direct current coils from quarterly to an 18 month interval. The letter of December 11, 1998 revised the scope of the application such that it now applies only to Westinghouse type AR relays which use alternating current coils. Accordingly, this amendment approves the extension of the surveillance interval only for Westinghouse type AR relays which use alternating current coils.

A copy of the safety evaluation is also enclosed. Notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Robert E. Martin, Senior Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

9901050170 981230  
PDR ADOCK 05000390  
P PDR

Docket No. 50-390

Enclosures: 1. Amendment No. 17 to NPF-90  
2. Safety Evaluation

cc w/enclosures: See next page

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Original signed by:

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Division of Reactor Projects - I/II  
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DATE	12/18/98		12/30/98		12/18/98		12/23/98		12/30/98	

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 30, 1998

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A copy of the safety evaluation is also enclosed. Notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Robert Martin".

Robert E. Martin, Senior Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-390

Enclosures: 1. Amendment No. 17 to NPF-90  
2. Safety Evaluation

cc w/enclosures: See next page

Mr. J. A. Scalice  
Tennessee Valley Authority

cc:  
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6A Lookout Place  
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## WATTS BAR NUCLEAR PLANT

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**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-390

WATTS BAR NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 17  
License No. NPF-90

1. The Nuclear Regulator Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated February 28, 1996, as supplemented October 2 and December 12, 1997, March 30 and December 11, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-90 is hereby amended to read as follows:

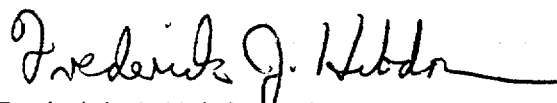
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 17, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented prior to startup following the second refueling outage. The licensee shall perform, as discussed in the licensee's submittal dated October 2, 1997, a plant specific aging assessment for all normally energized and periodically energized Type AR slave relays to determine a service life which satisfies the recommendations and guidance set forth in the topical report, WCAP-13877, prior to completion of the second refueling outage. The licensee shall revise, as discussed in the licensee's submittal dated March 30, 1998, the Watts Bar Nuclear Plant Technical Instruction TI-119, "Maintenance Rule Performance Indicator Monitoring, Trending, and Reporting," Attachment 30, Reactor Protection System (099), to require that the surveillance interval be evaluated and reduced, when needed, if two or more Westinghouse Type AR relays fail within a 12-month interval, prior to completion of the second refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdorn, Director  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: ~~December~~ 30, 1998

ATTACHMENT TO AMENDMENT NO. 17

FACILITY OPERATING LICENSE NO. NPF-90

DOCKET NO. 50-390

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

Remove Pages

3.3-32  
3.3-55  
B 3.3-116  
B 3.3-120  
B 3.3 162

Insert Pages

3.3-32  
3.3-55  
B 3.3-116  
B 3.3-120  
B 3.3-162

**SURVEILLANCE REQUIREMENTS (continued)**

<b>SURVEILLANCE</b>		<b>FREQUENCY</b>
SR 3.3.2.5	<p>-----NOTE----- Slave relays tested by SR 3.3.2.7 are excluded from this surveillance. -----</p> <p>Perform SLAVE RELAY TEST.</p>	<p>92 days</p> <p><u>OR</u></p> <p>18 months for Westinghouse type AR relays</p>
SR 3.3.2.6	<p>-----NOTE----- Verification of relay setpoints not required. -----</p> <p>Perform TADOT.</p>	<p>92 days</p>
SR 3.3.2.7	<p>Perform SLAVE RELAY TEST on slave relays K603A, K603B, K604A, K604B, K607A, K607B, K609A, K609B, K612A, K625A, and K625B.</p>	<p>18 months</p>
SR 3.3.2.8	<p>-----NOTE----- Verification of setpoint not required. -----</p> <p>Perform TADOT.</p>	<p>18 months</p>
SR 3.3.2.9	<p>-----NOTE----- This Surveillance shall include verification that the time constants are adjusted to the prescribed values. -----</p> <p>Perform CHANNEL CALIBRATION.</p>	<p>18 months</p>

(continued)



**SURVEILLANCE REQUIREMENTS**

-----NOTE-----  
Refer to Table 3.3.6-1 to determine which SRs apply for each Containment Vent Isolation Function.  
-----

SURVEILLANCE	FREQUENCY
SR 3.3.6.1 Perform CHANNEL CHECK.	12 hours
SR 3.3.6.2 Perform ACTUATION LOGIC TEST.	31 days on a STAGGERED TEST BASIS
SR 3.3.6.3 Perform MASTER RELAY TEST.	31 days on a STAGGERED TEST BASIS
SR 3.3.6.4 Perform COT.	92 days
SR 3.3.6.5 Perform SLAVE RELAY TEST.	92 days  OR 18 months for Westinghouse type AR relays
SR 3.3.6.6 -----NOTE----- Verification of setpoint is not required.	
Perform TADOT.	18 months
SR 3.3.6.7 Perform CHANNEL CALIBRATION.	18 months

BASES

SURVEILLANCE  
REQUIREMENTS

SR 3.3.2.4 (continued)

The Frequency of 92 days is justified in Reference 7, except for Function 7. The Frequency for Function 7 is justified in Reference 10.

SR 3.3.2.5

SR 3.3.2.5 is the performance of a SLAVE RELAY TEST. The SLAVE RELAY TEST is the energizing of the slave relays. Contact operation is verified in one of two ways. Actuation equipment that may be operated in the design mitigation MODE is either allowed to function, or is placed in a condition where the relay contact operation can be verified without operation of the equipment. Actuation equipment that may not be operated in the design mitigation MODE is prevented from operation by the SLAVE RELAY TEST circuit. For this latter case, contact operation is verified by a continuity check of the circuit containing the slave relay. This test is performed every 92 days. The Frequency is adequate, based on industry operating experience, considering instrument reliability and operating history data.

For ESFAS slave relays which are Westinghouse type AR relays, the SLAVE RELAY TEST is performed every 18 months. The frequency is based on the relay reliability assessment presented in Reference 13. This reliability assessment is relay specific and applies only to Westinghouse type AR relays with AC coils. Note that, for normally energized applications, the relays may require periodic replacement in accordance with the guidance given in Reference 13.

This SR is modified by a Note, which states that performance of this test is not required for those relays tested by SR 3.3.2.7.

SR 3.3.2.6

SR 3.3.2.6 is the performance of a TADOT every 92 days. This test is a check of the Loss of Offsite Power (Function 6.d), AFW Pump Suction Transfer on Suction Pressure-Low for motor driven and turbine driven pumps (Functions 6.f and 6.g respectively), and Turbine Trip and Feedwater Isolation - Main Steam Valve Vault Rooms Water Level - High (Function 5.d).

The SR is modified by a Note that excludes verification of setpoints for relays. Relay setpoints require elaborate bench calibration and are verified during CHANNEL CALIBRATION. The Frequency is adequate. It is based on industry operating experience, considering instrument reliability and operating history data.

(continued)

BASES

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REFERENCES  
(continued)

6. WCAP-12096, Rev. 7. "Westinghouse Setpoint Methodology for Protection System, Watts Bar 1 and 2." March 1997.
  7. WCAP-10271-P-A, Supplement 1 and Supplement 2, Rev. 1. "Evaluation of Surveillance Frequencies and Out of Service Times for the Reactor Protection Instrumentation System." and "Evaluation of Surveillance Frequencies and Out of Service Times for the Engineered Safety Features Actuation System." May 1986 and June 1990.
  8. Watts Bar Technical Requirements Manual, Section 3.3.2. "Engineered Safety Feature Response Times."
  9. TVA Letter to NRC, November 9, 1984. "Request for Exemption of Quarterly Slave Relay Testing. (L44 841109 808)."
  10. Evaluation of the applicability of WCAP-10271-P-A, Supplement 1, and Supplement 2, Revision 1, to Watts Bar.
  11. Westinghouse letter to TVA (WAT-D-8347), September 25, 1990. "Charging/Letdown Isolation Transients" (T33 911231 810).
  12. Design Change Notice W-38238 associated documentation.
  13. WCAP-13877, Rev. 1. "Reliability Assessment of Westinghouse Type AR Relays Used As SSPS Slave Relays." August 1998.
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**BASES**

**SURVEILLANCE  
REQUIREMENTS**

**SR 3.3.6.5 (continued)**

For ESFAS slave relays which are Westinghouse type AR relays, the SLAVE RELAY TEST is performed every 18 months. The frequency is based on the relay reliability assessment presented in Reference 3. This reliability assessment is relay specific and applies only to Westinghouse type AR relays with AC coils. Note that, for normally energized applications, the relays may require periodic replacement in accordance with the guidance given in Reference 3.

**SR 3.3.6.6**

SR 3.3.6.6 is the performance of a TADOT. This test is a check of the Manual Actuation Functions and is performed every 18 months. Each Manual Actuation Function is tested up to, and including, the master relay coils. In some instances, the test includes actuation of the end device (i.e., pump starts, valve cycles, etc.).

For these tests, the relay trip setpoints are verified and adjusted as necessary. The Frequency is based on the known reliability of the Function and the redundancy available, and has been shown to be acceptable through operating experience.

The SR is modified by a Note that excludes verification of setpoints during the TADOT. The Functions tested have no setpoints associated with them.

**SR 3.3.6.7**

A CHANNEL CALIBRATION is performed every 18 months, or approximately at every refueling. CHANNEL CALIBRATION is a complete check of the instrument loop, including the sensor. The test verifies that the channel responds to a measured parameter within the necessary range and accuracy.

The Frequency is based on operating experience and is consistent with the typical industry refueling cycle.

**REFERENCES**

1. Title 10, Code of Federal Regulations, Part 100.11, "Determination of Exclusion Area, Low Population Zone, and Population Center Distance."
2. NUREG-1366, "Improvement to Technical Specification Surveillance Requirements," December 1992.
3. WCAP-13877, Rev. 1, "Reliability Assessment of Westinghouse Type AR Relays Used As SSPS Slave Relays," August 1998.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 17 TO FACILITY OPERATING LICENSE NO. NPF-90

TENNESSEE VALLEY AUTHORITY

WATTS BAR NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-390

1.0 INTRODUCTION

By letter dated February 28, 1996, as supplemented October 2 and December 12, 1997, March 30 and December 11, 1998, the Tennessee Valley Authority (TVA or the licensee) submitted a request for changes to the Watts Bar Nuclear Plant (WBN), Unit 1, Technical Specifications (TSs). The February 28, 1996 letter proposed to extend the surveillance interval for Westinghouse type AR relays with alternating current (ac) and direct current (dc) coils from quarterly to an 18 month interval. The letter of December 11, 1998 revised the scope of the application such that it now applies only to Westinghouse type AR relays which use ac coils. Accordingly, this amendment approves the extension of the surveillance interval only for Westinghouse type AR relays which use ac coils. The October 2 and December 12, 1997, March 30 and December 11, 1998, letters provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 BACKGROUND

TVA's letter of February 28, 1996, submitted proposed TS changes for WBN, Unit 1, as a lead plant, based on generic Westinghouse Owners Group (WOG) topical reports, as discussed below. The proposal, as modified by TVA's letter of December 11, 1998 would allow a test interval extension for Westinghouse Type AR relays used as slave relays which use ac coils. Currently at WBN and other Westinghouse plants, slave relays for the Engineered Safety Features Actuation System (ESFAS) are tested quarterly with the exception of some relays which were previously approved by the U.S. Nuclear Regulatory Commission (NRC or Commission) to be tested every 18 months. The proposed changes to the TSs would extend the test interval for all Westinghouse Type AR slave relays in WBN's ESFAS to 18 months. In order to justify these changes, TVA provided generic

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ENCLOSURE

Westinghouse Topical Report WCAP-13877, Rev. 0, "Reliability Assessment of Westinghouse Type AR Relays Used as SSPS Slave Relays," dated June 1994. In addition to this Topical Report the WOG submitted Westinghouse Topical Report WCAP-13900, "Extension of Slave Relay Surveillance Test Interval," dated April 1994.

Following review of the above topical reports, the NRC staff, by letter dated September 3, 1996 requested additional information and TVA responded by letters dated October 2, 1997 and December 12, 1997. A further request for additional information (RAI) was submitted to TVA by letter dated January 27, 1998, and TVA responded by letter dated March 30, 1998, which included revised pages to WCAP-13877. The WOG by letter dated September 1, 1998, submitted Rev. 1 to WCAP-13877 incorporating these revisions. The staff, by letter dated October 26, 1998, accepted and issued a Safety Evaluation (SE) to the WOG finding the above topical reports acceptable with the requirement that each licensee address the following plant specific items:

1. Confirm the applicability of the WCAP-13877, analyses to their plant.
2. Ensure that the contact loading analysis for the Type AR relays has been performed to determine the acceptability of these relays.
3. Determine the qualified life for the Type AR relays based on plant specific environmental conditions.
4. Establish a program to evaluate the adequacy of the proposed test interval if two or more AR relays fail in a 12-month period.

### 3.0 EVALUATION

The licensee, in submittals dated October 2, 1997, December 12, 1997, and March 30, 1998, addressed each of the above plant specific items and the staff evaluation is discussed below:

1. TVA, in the letter dated October 2, 1997, confirmed the applicability of the topical report to WBN for the Type AR relays with ac coils and further proposed to extend the applicability of the topical report to Type AR relays with dc coils, used as interposing relays at WBN. The staff, in a telephone conference with TVA and Westinghouse on November 7, 1998, identified that additional information would be necessary in order to determine applicability of the topical report to relays with dc coils. Subsequently, by letter dated December 11, 1998, TVA revised the scope of its proposed license amendment request such that it is only applicable to Westinghouse type AR relays which use ac coils. Based on this, the staff considers that the licensee submittal has adequately addressed the applicability of the topical report to WBN.

2. TVA's letter dated December 12, 1997 provided the results of contact loading analysis which concluded that the ESFAS slave relay contacts are adequate for their applications and will not be subjected to long term degradation. Based on this, the staff finds that TVA has adequately addressed the staff's concern regarding contact loading of AR relays.
3. TVA's letter dated October 2, 1997, committed, in response to question 11, to perform a plant specific aging assessment for all normally energized and periodically energized Type AR slave relays to determine a service life which satisfies the recommendations and guidance set forth in the topical report, WCAP-13877. TVA has committed to complete this assessment prior to completion of the second refueling outage. The staff considers TVA's commitment acceptable based on the fact that the generic service life in the topical report for these relays has been established as 19 years and all relays should be replaced if any relay fails after 14 years.
4. TVA's letter dated March 30, 1998, committed, in its response to RAI question 4, to revise the WBN Technical Instruction TI-119, "Maintenance Rule Performance Indicator Monitoring, Trending, and Reporting," Attachment 30, Reactor Protection System (099), prior to implementation of the approved TS changes, to require that the surveillance interval be evaluated and reduced, when needed, if two or more Westinghouse Type AR relays fail within a 12-month interval. As discussed with TVA representatives on December 15 and 16, 1998, this implementation date will be consistent with the implementation of the license amendment, which is prior to completion of the second refueling outage. The staff finds TVA's commitment acceptable.

#### Summary

Based on the review of the WCAP-13877, Rev. 1, WCAP-13900, Rev. 0, and the licensee's submittals referencing these topical reports, the staff concludes that the proposed test interval extension to 18 months for Westinghouse Type AR relays with ac coils used in ESFAS slave relays applications is justified for WBN. This test interval extension does not apply to Westinghouse Type AR relays with dc coil relays. Therefore, the licensee's TS changes are acceptable for Westinghouse Type AR relays with ac coil.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Tennessee State official was notified of the proposed issuance of the amendment. The State official had no comments.

## 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (61 FR 15998 dated April 10, 1996). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: H. Garg

Date: December 30, 1998