

April 7, 1994

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Docket No. 50-305

Mr. C. A. Schrock
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Wisconsin Public Service Corporation
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Dear Mr. Schrock:

SUBJECT: AMENDMENT NO. 107 TO FACILITY OPERATING LICENSE NO. DPR-43
(TAC NO. M88693)

The Commission has issued the enclosed Amendment No. 107 to Facility Operating License No. DPR-43 for the Kewaunee Nuclear Power Plant (KNPP). This amendment revises the Technical Specifications (TS) in response to your application dated February 1, 1994.

The amendment revises the TS by removing the review of the Emergency Plan and its implementing procedures from the list of responsibilities of the Plant Operations Review Committee (PORC). Guidance for this change was provided in Generic Letter 93-07, "Modification of the Technical Specification Administrative Control Requirements for Emergency and Security Plans," dated December 28, 1993. Several other administrative TS changes were also made including removing specific titles from the list of PORC members in TS 6.5.a.2 and deleting TS 6.5.b which describes the Corporate Support Staff.

A copy of the Safety Evaluation is also enclosed. Notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by Richard J. Laufer
Richard J. Laufer, Acting Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 107 to License No. DPR-43
 2. Safety Evaluation
- cc w/enclosures:

OFC LA:PDIII-3	PE:PDIII-3	PM:PDIII-3	LPM:MPA D034	BC:RPEB	BC:OTSB	PD:PDIII-3	OGC
NAME MRUSHBROOK	RLAUFER	AHANSEN	WRECKLEY	GZECH	CGRIMES	JHANNON	OPW
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Wisconsin Public Service Corporation

Kewaunee Nuclear Power Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

DOCKET NO. 50-305

KEWAUNEE NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 107
License No. DPR-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Public Service Corporation, Wisconsin Power and Light Company, and Madison Gas and Electric Company (the licensees) dated February 1, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-43 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 107 are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, and is to be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Douglas V. Pickett for

John N. Hannon, Director
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: April 7, 1994

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6.5.a.3	Alternates	6.5-1
6.5.a.4	Meeting Frequency	6.5-1
6.5.a.5	Quorum	6.5-1
6.5.a.6	Responsibilities	6.5-1
6.5.a.7	Authority	6.5-2
6.5.a.8	Records	6.5-2
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6.5.c.9	Authority	6.5-5
6.5.c.10	Records	6.5-5
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6.12	System Integrity	6.12-1
6.13	High Radiation Area	6.13-1
6.14	Post-Accident Sampling and Monitoring	6.14-1
6.15	Secondary Water Chemistry	6.15-1
6.16	Radiological Effluents	6.16-1
6.17	Process Control Program (PCP)	6.17-1
6.18	Offsite Dose Calculation Manual (ODCM)	6.18-1
6.19	Major Changes to Radioactive Liquid, Gaseous and Solid Waste Treatment Systems	6.19-1

7/8.0 Deleted

ATTACHMENT TO LICENSE AMENDMENT NO. 107

FACILITY OPERATING LICENSE NO. DPR-43

DOCKET NO. 50-305

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

TS iv

TABLE TS 3.5-1
(PAGE 1 OF 2)

TS 4.2-1

TS 6.5-1 through
TS 6.5-7 (7 pages)

INSERT

TS iv

TABLE TS 3.5-1
(PAGE 1 OF 2)

TS 4.2-1

TS 6.5-1 through
TS 6.5-6 (6 pages)

TABLE TS 3.5-1

ENGINEERED SAFETY FEATURES INITIATION INSTRUMENT SETTING LIMITS

NO.	FUNCTIONAL UNIT	CHANNEL	SETTING LIMIT
1	High Containment Pressure (Hi)	Safety injection ⁽¹⁾	≤ 4 psig
2	High Containment Pressure (Hi-Hi)	a. Containment spray b. Steam line isolation of both lines	≤ 23 psig ≤ 17 psig
3	Pressurizer Low Pressure	Safety injection ⁽¹⁾	≥ 1815 psig
4	Low Steam Line Pressure	Safety injection ⁽¹⁾ Lead time constant Lag time constant	≥ 500 psig ≥ 12 seconds ≤ 2 seconds
5	High Steam Flow in a Steam Line Coincident with Safety Injection and "Lo-Lo" T_{avg}	Steam line isolation of affected line ⁽²⁾	\leq d/p corresponding to 0.745×10^6 lb/hr at 1005 psig $\geq 540^\circ\text{F}$
6	High-High Steam Flow in a Steam Line Coincident with Safety Injection	Steam line isolation of affected line ⁽²⁾	\leq d/p corresponding to 4.5×10^6 lb/hr at 735 psig
7	Forebay Level	Trip circ. water pumps	

⁽¹⁾Initiates containment isolation, feedwater line isolation, shield building ventilation, auxiliary building special vent, and starting of all containment fans. In addition, the signal overrides any bypass on the accumulator valves.

⁽²⁾Confirm main steam isolation valves closure within 5 seconds when tested. d/p = differential pressure

4.2 ASME CODE CLASS IN-SERVICE INSPECTION AND TESTING

APPLICABILITY

Applies to in-service structural surveillance of the ASME Code Class components and supports and functional testing of pumps and valves.

OBJECTIVE

To assure the continued integrity and operational readiness of ASME Code Class 1, 2 and 3 components.

SPECIFICATION

a. ASME Code Class 1, 2 and 3 Components and Supports

1. In-service inspection of ASME Code Class 1, Class 2 and Class 3 components and supports shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR 50.55a(g)(6)(i). The testing and surveillance of shock suppressors (snubbers) is detailed in TS 3.14 and TS 4.14.
2. In-service testing of ASME Code Class 1, Class 2 and Class 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50.55a(f), except where relief has been granted by the Commission pursuant to 10 CFR 50.55a(f)(6)(i).
3. Surveillance testing of pressure isolation valves:
 - a. Periodic leakage testing⁽¹⁾ on each valve listed in Table TS 3.1-2 shall be accomplished prior to entering the OPERATING mode after every time the plant is placed in the COLD SHUTDOWN condition for refueling, after each time the plant is placed in a COLD SHUTDOWN condition for 72 hours if testing has not been accomplished in the preceding 9 months, and prior to returning the valve to service after maintenance, repair, or replacement work is performed.

⁽¹⁾To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria.

6.5 REVIEW AND AUDIT

a. Plant Operations Review Committee (PORC)

1. Function

The PORC shall function to advise the Manager - Kewaunee Plant on matters related to nuclear safety.

2. Composition

The PORC shall be composed of key supervisors of the on-site staff from the following disciplines: Operations, Maintenance, Instrument and Control, Reactor Engineering, Quality, and Radiological.

3. Alternates

Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates for required members shall participate in PORC meetings at any one time.

4. Meeting Frequency

The PORC shall meet at least once per calendar month and as convened by the Chairman.

5. Quorum

A quorum of the PORC shall consist of the chairman (or his designated alternate as stated in TS 6.1) and a majority of the required members including temporary alternates.

6. Responsibilities

The PORC shall be responsible for:

- A. Review of operating, maintenance and other procedures including emergency operating procedures which affect nuclear safety as determined by the Manager - Kewaunee Plant. Changes to those procedures are made in accordance with the provisions of TS 6.8.a.
- B. Review of all proposed tests and experiments that affect nuclear safety.
- C. Review of all proposed changes to the Technical Specifications.
- D. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

- E. Review of all proposed changes to the Security Plan, Fire Plan, and their respective implementing procedures.
- F. Review all reports covering the investigation of all violations of the Technical Specifications and the recommendations to prevent recurrence.
- G. Review plant operations to detect potential safety hazards.
- H. Performance of special reviews and investigations and prepare reports thereon as requested by the Chairman of the Nuclear Safety Review and Audit Committee.
- I. Review of all REPORTABLE EVENTS
- J. Review of changes to the PROCESS CONTROL PROGRAM, the OFF-SITE DOSE CALCULATION MANUAL, and the RADIOLOGICAL ENVIRONMENTAL MONITORING MANUAL.

7. Authority

The PORC shall:

- A. Recommend to the Manager - Kewaunee Plant approval or disapproval of items considered under TS 6.5.a.6.A through TS 6.5.a.6.E.
- B. Make determinations with regard to whether or not each item considered under TS 6.5.a.6 constitutes an unreviewed safety question.
- C. Provide immediate notification in the form of draft meeting minutes to the Senior Vice President - Nuclear Power and the Chairman-Nuclear Safety Review and Audit Committee of disagreement between the PORC and the Manager - Kewaunee Plant. The Manager - Kewaunee Plant shall have responsibility for resolution of such disagreements.

8. Records

Minutes shall be kept of all meetings of the PORC and copies shall be sent to the Senior Vice President - Nuclear Power and the Chairman - Nuclear Safety Review and Audit Committee.

b. Deleted

c. Nuclear Safety Review and Audit Committee (NSRAC)

1. Function

The NSRAC shall function to provide independent review and audit of designated activities in the areas of:

- A. Nuclear Power Plant Operations
- B. Nuclear Engineering
- C. Chemistry and Radio-Chemistry
- D. Metallurgy
- E. Instrumentation
- F. Radiological Safety
- G. Mechanical and Electrical Engineering
- H. Quality Assurance Practices
- I. Other appropriate fields as determined by the Committee, to be associated with the unique characteristics of the nuclear power plant.

2. Composition

The NSRAC shall be composed of, but not necessarily limited to:

- A. At least three technically qualified persons who are not members of the plant staff.
- B. One member from the supervisory staff of the plant.
- C. At least two qualified non-company affiliated technical consultants.
- D. In-house staff management advisors as required.

The Committee membership and its Chairman and Vice Chairman shall be appointed by the Senior Company Officer to whom the NSRAC reports. Each member of the NSRAC shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of five years technical experience, of which a minimum shall be in one or more areas given in TS 6.5.c.1.

3. Alternates

Alternate members shall be appointed by the NSRAC Chairman, upon approval by the Senior Vice President - Nuclear Power, to serve on a temporary basis; however, no more than two alternates shall participate in NSRAC activities at any one time.

4. Consultants

Consultants may be utilized as determined by the Chairman - NSRAC to provide expert advice to the NSRAC.

5. Meeting Frequency

The NSRAC shall meet at least once every six months.

6. Quorum

A quorum of the NSRAC shall consist of the Chairman or Vice Chairman and four members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the plant.

7. Review

The NSRAC shall review:

- A. Safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question.
- B. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59.
- C. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59.
- D. Proposed changes in Technical Specifications or licenses.
- E. Reports covering violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- F. Reports covering significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- G. Reports covering all REPORTABLE EVENTS.
- H. Reports covering any indication of an unanticipated deficiency in some aspect of design or operation of safety-related structures, systems, or components.
- I. Reports and meeting minutes of the PORC.

8. Audits

Audits of plant activities shall be performed under the cognizance of the NSRAC. These audits shall include:

- A. Conformance of plant operation to the provisions contained within the Technical Specifications and applicable license conditions.
- B. Performance, training, and qualifications of the entire plant staff.
- C. Results of all actions taken to correct deficiencies occurring in plant equipment, structures, systems, or method of operation that affect nuclear safety.
- D. Performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR Part 50.
- E. The Plant Fire Protection Program, implementing procedures and the fire protection and loss prevention program.
- F. Any other area of plant operation considered appropriate by the NSRAC or the Senior Company Officer to whom the NSRAC reports.
- G. The Radiological Environmental Monitoring Program and the results thereof.
- H. The OFF-SITE DOSE CALCULATION MANUAL and implementing procedures.
- I. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes.

9. Authority

The NSRAC shall report to a Senior Company Officer and shall advise the Officer on those areas of responsibility specified in TS 6.5.c.7 and TS 6.5.c.8.

10. Records

Records of NSRAC activities shall be prepared, approved and distributed as follows:

- A. Minutes of each NSRAC meeting forwarded to the Senior Company Officer to whom the NSRAC reports within 14 days following each meeting.
- B. Reports of reviews required by TS 6.5.c.7.E through TS 6.5.c.7.H, forwarded to the Senior Company Officer to whom the NSRAC reports within 14 days following completion of the review.

- C. Reports of audits performed by NSRAC shall be forwarded to the Senior Company Officer to whom the NSRAC reports and to the management positions responsible for the areas audited within 30 days after completion of the audit.

TS 6.5-6

Amendment No. 58, 64, 77, 81, 86, 99, 103, 107



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATING TO AMENDMENT NO. 107 TO FACILITY OPERATING LICENSE NO. DPR-43

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

KEWAUNEE NUCLEAR POWER PLANT

DOCKET NO. 50-305

1.0 INTRODUCTION

By letter dated February 1, 1994, the Wisconsin Public Service Corporation (WPSC), the licensee, submitted a request for revision to the Kewaunee Nuclear Power Plant (KNPP) Technical Specifications (TS). The proposed amendment would revise the TS by removing the review of the Emergency Plan and its implementing procedures from the list of responsibilities of the Plant Operations Review Committee (PORC). Guidance for this change was provided in Generic Letter (GL) 93-07, "Modification of the Technical Specification Administrative Control Requirements for Emergency and Security Plans," dated December 28, 1993. Several other administrative TS changes are also proposed including removing specific titles from the list of PORC members in TS 6.5.a.2 and deleting TS 6.5.b which describes the Corporate Support Staff.

2.0 EVALUATION

TS 6.5.a.2

The licensee's proposal is to remove the specific titles of the PORC Chairman and required members. Instead of listing the required PORC members by title as is currently done, TS 6.5.a.2 would be revised to state that "The PORC shall be composed of key supervisors of the onsite staff from the following disciplines: Operations, Maintenance, Instrument and Control, Reactor Engineering, Quality, and Radiological."

Removing these specific titles does not represent a safety concern and may prove to be a cost savings for the licensee, as a TS amendment will no longer be required to change this section, in the event of title or organizational changes. This proposed change will provide increased flexibility regarding the composition of the PORC while maintaining the same level of qualifications. The staff, therefore, finds this proposed TS change acceptable.

TS 6.5.a.6.E

The licensee's proposal is to remove the review of the Emergency Plan and implementing procedures from the list of responsibilities of the PORC. This requirement is redundant as it is already contained in the Emergency Plan. This TS change is consistent with the guidance provided in GL 93-07 and is therefore acceptable.

TS 6.5.b

The licensee's proposal is to delete TS 6.5.b, "Corporate Support Staff (CSS)." The offsite organization for plant management and technical support shall be as described in the Operational Quality Assurance Program Description (OQAPD) as stated in TS 6.2.a.

Removing the description of the CSS from the TS does not represent a safety concern and may prove to be a cost savings for the licensee, as a TS amendment will no longer be required to change this section in the event of title or organizational changes. In addition, this change is consistent with the requirements of NUREG-0452, "Standard Technical Specifications for Westinghouse Pressurized Water Reactors," which do not include a description of the CSS. The staff, therefore, finds this proposed change acceptable.

Administrative Changes

In addition to the changes described above, the licensee is proposing the following administrative changes.

- 1) Table TS 3.5-1 (Page 1 of 2): An administrative change is being proposed to insert a comma in Footnote 1 after the words "feedwater line isolation." This comma was inadvertently deleted with the submittal of TS Proposed Amendment 76 on August 1, 1986.
- 2) TS 4.2.a.2: An administrative change is being proposed to revise the reference to the federal requirement for inservice testing from 10 CFR 50.55a(g) to 10 CFR 50.55a(f). Effective August 31, 1992, the NRC amended 10 CFR 50.55a to separate the requirements for inservice testing from those for inservice inspection by moving the requirements for inservice testing to a separate paragraph. The requirement for inservice testing was moved from 10 CFR 50.55a(g) to 10 CFR 50.55a(f).
- 3) TS 6.5.c.8.E: The proposed change would remove the word "independent" from the TS for clarification purposes. TS 6.5.c.8.E is a requirement for the Nuclear Safety Review and Audit Committee (NSRAC) to conduct an independent audit of the fire protection program. The TS currently states NSRAC will conduct an audit of the independent fire protection program. The current sentence structure is the result of an administrative error which occurred when this TS was previously revised.

The staff has reviewed the changes discussed above and, since they are administrative in nature, provide clarification, and do not alter the intent or interpretation of the TS, the staff finds them acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: R. Laufer

Date: April 7, 1994