Facility: CPSES			Date	of Ex	am: (	05/20	01	E	xam	Leve	I: RC	)	
Tion	0				K/A	A Cat	egor	y Poi	nts				
Tier	Group	К 1	K 2	K 3	K 4	K 5	К 6	A 1	A 2	A 3	A 4	G *	Point Total
1.	1	2	2	5				2	3			2	16
Emergency & Abnormal	2	2	1	4				4	5			1	17
Plant Evolutions	3	3	0	0				0	0			0	3
	Tier Totals	7	3	9				6	8			3	36
	1     3     2     3     3     4     0     1     3     2     1     1       2     3     1     3     3     1     1     2     3     1     1												
2. Plant	2	3	1	3	3	1	1	2	3	1	1	1	20
Systems	3	1	1	1	0	0	1	2	0	2	0	0	8
	Tier Totals												51
3. Generic K	nowledge ar	nd Ab	oilities	5	Ca	at 1	Ca	at 2	Ca	at 3	Ca	at 4	
						3		3		4		3	13
ea tw 2. A 3. S 4. S 5. T 6.* T 6.* T 7. O to to	nsure that af ach tier (i.e., vo). ctual point to elect topics f opics from a ystems/evolu- he shaded a he generic k atalog, but t on the followi- opic, the topi- otals for each ne basis of p ne table above	the " from giver utions reas (/As i he to ng pa cs' im n syst lant-s	Tier must many syst s with are r n Tie pics i ages, nportatem a	Total mate y syst em u nin ea not ap rs 1 a must ente ance and c	s" in the the terms; and search g oplication be re be rethe rating atego	each ose s avoi s they roup ble to shal eleva K/A gs for ory. h	K/A pecifi d sel / rela are id o the l be s nt to numb r the K/As	categ ed in ecting te to dentif categ select the a pers, RO li belov	gory s the f g mo plant fied o gory/ ted fr pplic a brid cens v 2.5	shall re that -spector tier. om S able e lev shou	not b an tw cific p asse Sectic evolu script el, ar uld be	o or priorit ociate on 2 c ition c tion c ad the just	s than three K/A ies. ed outline. of the K/A or system. of each e point ified on

33 of 45

ES-401			Emerge	ency an	PWR R d Abnor	O Examina mal Plant	ation Outline Evolutions - Tier 1/Group 1	Form	1 ES-40
E/APE # / Name / Safety Function	К1	К2	КЗ	A1	A2	G	K/A Topic(s)	Imp.	Poin
000005 Inoperable/Stuck Control Rod / 1						2.1.12	A/T apply technical specifications for a system	2.9	1
000015/17 RCP Malfunctions / 4			A03				Sequence of events for manually tripping RX & RCP	3.7	1
BW/E09; CE/A13; W/E09&E10 Natural Circ./4	E 1						Components, capacity, and function of emergency systems	3.3	1
000024 Emergency Boration / 1									
000026 Loss of Component Cooling Water / 8					A 06		Length of time after loss of CCW flow to a component before component may be damaged	2.8	1
000027 Pressurizer Pressure Control System Malfunction / 3				:	A16		Actions to be taken if PZR instrument fails low	3.6	1
<del>000040 (BW/E05; CE/E05;</del> W/E12) Steam Line Rupture - Excessive Heat Transfer / 4			E 4			-	Control room proc. Are adhered to and limitations of the license and amendments are not violated	3.5	1
CE/A11; W/E08 RCS Overcooling - PTS / 4		E 1		·			Components including instrumentation, signals, interlocks, failure modes, and automatic, and manual features	3.4	1
000051 Loss of Condenser Vacuum / 4			A 01				Loss of steam dump capability upon loss of condenser vacuum	2.8	1
000055 Station Blackout / 6				E 04			Ability to operate and monitor the reduction of loads on the battery	3.5	1
000057 Loss of Vital AC Elec. Inst. Bus / 6				A 06			Manual control of components for which auto control is lost	3.5	1
000062 Loss of Nuclear Service Water / 4			A 01				Conditions that will initiate the automatic opening & closing of the SWS insolation valves to the nuclear service water coolers	3.2	1
000067 Plant Fire On-site / 9					A 11		Time limit for use of respirators	3.3	1
000068 (BW/A06) Control Room Evac. / 8			A13				Performing SDM calculation including boron needed and time	3.3	1
000069 (W/E14) Loss of CTMT Integrity / 5						2.4.47	Diagnose/recognize trends in an accurate and timely manner	3.4	1
000074 ( <del>W/E06&amp;E07</del> ) Inad. Core Cooling / 4	E 07						Definition of saturated steam	2.8	1
BW/E03 Inadequate Subcooling Margin / 4									
000076 High Reactor Coolant Activity / 9		A 01					Knowledge of the interactions between high RCS activity and process radiation monitors	2.6	1
BW/A02&A03 Loss of NNI-X/Y / 7									
K/A Category Totals:	2	2	5	2	3	2	Group Point Total:		16

ES-401		E	mergen	PW cy and Al	R RO Ex phormal I	amination Plant Evolu	Outline utions - Tier 1/Group 2	Form	1 ES-401
E/APE # / Name / Safety Function	К1	К2	КЗ	A1	A2	G	K/A Topic(s)	Imp.	Point
000001 Continuous Rod Withdrawal / 1			A02				Tech spec limits on rod operability	3.2	1
000003 Dropped Control Rod / 1						2.1.20	Ability to execute procedure steps	4.3	1
000007 (BW/E02&E10 CE/E02) Reactor Trip - Stabilization - Recovery / 1					E 05		Reactor trip first-out indication	3.4	1
BW/A01 Plant Runback / 1									
BW/A04 Turbine Trip / 4									
000008 Pressurizer Vapor Space Accident / 3					A 09		Pzr spray block valve controls and indicators	3.6	1
000009 Small Break LOCA / 3				E12			Ability to operate and monitor the RPS as applies to a SBLOCA	4.2	1
000011 Large Break LOCA / 3					E 08		Cond necessary for recovery when acc reaches a stable phase	3.4	1
W/E04 LOCA Outside Containment / 3	E2						Normal, abn, and emerg operating proc with LOCA outside cont.	3.5	1
BW/E08; W/E03 LOCA Cooldown/Depress. / 4									
W/E11 Loss of Emergency Coolant Recirc. / 4			E2				Normal, abn, & emerg oper proc associated with loss of recirc	3.5	1
W/EO1 & E02 Rediagnosis & SI Termination / 3					,				
000022 Loss of Reactor Coolant Makeup / 2							· · · · · · · · · · · · · · · · · · ·		
000025 Loss of RHR System / 4			A 03				Immediate actions contained in EOP for loss of RHR	3.9	1
000029 Anticipated Transient w/o Scram / 1					E 09		Determine/interpret the occurrence of a turb/Rx trip with ATWS	4.4	1
000032 Loss of Source Range NI / 7			A 01				Startup termination on source range loss	3.2	1
000033 Loss of Intermediate Range NI / 7									
000037 Steam Generator Tube Leak / 3									
000038 Steam Generator Tube Rupture / 3				E 09		•	Pzr tank level/press indicators, gauges, and recorders	3.2	1
000054 (CE/E06) Loss of Main Feedwater / 4				A 04			HPI, under total feedwater loss conditions	4.4	1
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4					E 1		Facility conditions and selection of app proc during abn & emerg ops	3.4	1
000058 Loss of DC Power / 6	A 01						Battery charger equipment and instrumentation	2.8	1
000059 Accidental Liquid RadWaste Rel. / 9									
000060 Accidental Gaseous Radwaste Rel. / 9		A 02					Interrelations between the accident gaseous radwaste release and Aux building ventilation	2.7	1
000061 ARM System Alarms / 7				A 01			Automatic actuation	3.6	1
W/E16 High Containment Radiation / 9									
CE/E09 Functional Recovery									
K/A Category Point Totals:	2	1	4	4	5	1	Group Point Total:		17

ES-401		E	merge	ncy ar	PWR Id Abn	RO E> ormal	amination Outline Plant Evolutions - Tier 1/Group 3	Form	ES-401-4
E/APE # / Name / Safety Function	К1	К2	КЗ	A1	A2	G	K/A Topic(s)	Imp.	Points
000028 Pressurizer Level Malfunction / 2	A 01						PZR reference leak abnormalities	2.8	1
000036 (BW/A08) Fuel Handling Accident / 8	A 01						Fuel handling incident radiation exposure hazards	3.5	1
000056 Loss of Off-site Power / 6									
000065 Loss of Instrument Air / 8									
BW/E13&E14 EOP Rules and Enclosures									
BW/A05 Emergency Diesel Actuation / 6									
BW/A07 Flooding / 8									
GE/A16 Excess RCS Leakage / 2									
W/E13 Steam Generator Over-pressure / 4	E 1.3						Annuciators and conditions indicating signals, and remedial actions associated with the steam generator overpressure	3.0	1
W/E15 Containment Flooding / 5									
K/A Category Point Totals:	3	0	0	0	0	0	Group Point Total:		3

ES-401						PWR F Plant S	RO Exa ystems	minatio - Tier 2	n Outlir 2/Group	ie 1			Fo	rm ES-401-4
System # / Name	K1	К2	КЗ	К4	К5	К6	A1	A2	A3	A4	G	K/A Topic(s)	Imp.	Points
001 Control Rod Drive		02										One-line diagram of pwr supply to trip bkrs	3.6	1
001 Control Rod Drive					96							Sign change in react when react is added	3.2	1
003 Reactor Coolant Pump							09					Seal flow and D/P	2.8	1
003 Reactor Coolant Pump											2.1 .19	Use plant computer to obtain & evaluate parametric info on system/component status	3,0	1
004 Chemical and Volume Control			06									Effect that loss/malfunction of CVCS on RCS	3.4	1
004 Chemical and Volume Control				01								Design features/interlocks for RCS O <sub>2</sub> control	2.8	1
013 Engineered Safety Features Actuation				15								Continuous testing	2.6	1
013 Engineered Safety Features Actuation					02							Safety system logic and reliability	2.9	1
015 Nuclear Instrumentation	08											RCS (pump start)	2.6	1
015 Nuclear Instrumentation			02									Effect that loss of NIS will have on CRDS	3.3	1
017 In-core Temperature Monitor								01				Thermocouple open and short circuits	3.1	1
022 Containment Cooling										01		Manually operate/monitor fans in Control Rm.	3.6	1
025 Ice Condenser														
056 Condensate	03											Relationship between cond and MFW	2.6	1
059 Main Feedwater			02									Effect of loss/malfunction of MFW on AFW	3.6	1
059 Main Feedwater	02											Relationship between MFW and AFW	3.4	1
061 Auxiliary/Emergency Feedwater		01										Bus pwr supplies to AFW MOVs	3.2	1
061 Auxiliary/Emergency Feedwater								06				Predict the impact of MFW leakage of AFW	2.7	1
068 Liquid Radwaste					04							Biological hazards of rad & goal of ALARA	3.2	1
068 Liquid Radwaste									02			Ability to monitor automatic isolation	3.6	1
071 Waste Gas Disposal				04								Isolation of waste gas release tanks	2.9	1
071 Waste Gas Disposal					04							Relationship of [H <sub>2</sub> /O <sub>2</sub> ] to flammability	2.5	1
072 Area Radiation Monitoring								02				Detector failure	2.8	1
072 Area Radiation Monitoring									01			Monitor changes in ARM vent alignment	2.9	1
K/A Category Point Totals:	3	2	3	3	4	0	1	3	2	1	1	Group Point Total:		23

ES-401					P	WR R lant Sy	O Exa stems	minatio - Tier	on Out 2/Grou	line ıp 2			Forn	n ES-401-4
System # / Name	К1	К2	КЗ	K4	K5	K6	A1	A2	A3	A4	G	K/A Topic(s)	Imp.	Points
002 Reactor Coolant						12						Effect of loss/malfunction of code SRVs	3.0	1
006 Emergency Core Cooling	09											Relationship between ECCS and nitrogen	2.6	1
010 Pressurizer Pressure Control							03					Predict/monitor change in PRT press/temp	2.9	1
011 Pressurizer Level Control							01					Predict/monitor change in PZR level/press	3.5	1
012 Reactor Protection	03											Relationship between RPS and CRDS	3.7	1
014 Rod Position Indication											2. 4. 21	Parameters and logic used to assess the status of safety functions	3.7	1
016 Non-nuclear Instrumentation			12									Effect loss/malfunction of NNIS on S/G	3.4	1
026 Containment Spray		01										Bus pwr supplies to CS pumps	3.4	1
029 Containment Purge										04		Manually operate cont evacuation signal	3.5	1
033 Spent Fuel Pool Cooling				05								Design feature for adequate SDM [boron]	3.1	1
035 Steam Generator								05				Impact of unbalanced flows to S/Gs	3.2	1
039 Main and Reheat Steam								04				Impact of malfunctioning stm dumps	3.4	1
055 Condenser Air Removal									03		-	Monitor automatic diversion of CARS exhaust	2.5	1
062 AC Electrical Distribution			01									Major system loads	3.5	1
063 DC Electrical Distribution				04				-				Trips	2.6	1
064 Emergency Diesel Generator	04											Relationship between EDG & DC dist system	3.6	1
073 Process Radiation Monitoring			01									Effect/loss of PRM on rad effluent releases	3.6	1
075 Circulating Water								03				Safety features/relationship between cond vacuum, turbine trip, & stm dump	2,5	1
079 Station Air				01								Cross connect with IAS	2.9	1
086 Fire Protection					04							Hazard to personnel as a result of fire type and methods of protection	2.9	1
K/A Category Point Totals:	3	1	3	3	1	1	2	3	1	1	1	Group Point Total:		20

ES-401					Pi Pi	WR R( ant Sys	D Exan stems	ninatio - Tier 2	n Outli 2/Grou	ne p 3			Forn	n ES-401-4
System # / Name	К1	К2	КЗ	К4	К5	K6	A1	A2	A3	A4	G	K/A Topic(s)	Imp.	Points
005 Residual Heat Removal		03										Pwr supplies to RCS press boundary MOVs	2.7	1
007 Pressurizer Relief/Quench Tank														
008 Component Cooling Water							01					Predict changes in CCW flow rate during ops	2.8	1
027 Containment Iodine Removal														
028 Hydrogen Recombiner and Purge Control						01						Effect of loss/malf of H2 recomb on HRPS	2.6	1
034 Fuel Handling Equipment	ļ								02			Monitor load limits associated with FHES	2.5	1
041 Steam Dump/Turbine Bypass Control														
045 Main Turbine Generator	20											Protection system	3.4	1
076 Service Water									02			Monitor auto ops of the SWS including emergency heat loads	3.7	1
078 Instrument Air			02									Effect of loss/malf of IAS will have on pneumatic valves and controls	3.4	1
103 Containment							01					Predict/monitor changes in containment press, temp, and humididty	3.7	1
K/A Category Point Totals:	1	1	1			1	2		2			Group Point Total:		8
						Plant	-Speci	fic Pric	orities					
System / Topic						Reco	mmen	ided R	eplace	ment f	or	Reason		Points
						1. <u>.</u>								
Plant-Specific Priority Total: (limit 10)					I							n na		

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39 of 45

<u>ES-401</u>

Facility:	$ \begin{array}{c ccccccccccccccccccccccccccccccccccc$												
Tion	0				K//	A Cat	egory	y Poi	nts				
Tier	Group					1			A2			G *	
1.	1	3	3	5				3	6			4	24
Emergency & Abnormal	2	1	1	3				4	5			2	16
Plant Evolutions	3	2										1	3
		6	4	8				7	11			7	43
	1		19										
2. Plant	2												17
Systems	3												4
													40
3. Generic K	nowledge ar	nd Ab	oilities	i	Ca	<u>t 1</u>	Са	t 2	Cat	3	Ca	nt 4	17
ea tw 2. Au 3. So 4. So 5. Ti 6.* Ti 6.* Ti 7. O to to	ach tier (i.e., /o). ctual point to elect topics f pics from a g ystems/evolu he shaded a	the " ptals r given utions reas /As in ne top ng pa cs' im a syst ant-s	Tier T must many syste with are n Tier bics n iges, porta em a	Fotals matc syst em ui in ea ot ap ot ap rs 1 a nust enter ince i nd ca	s" in e h tho ems; nless ch gr plica nd 2 be re r the r ating atego	each se sr avoi they oup ble to shall levar K/A r s for ry. k	K/A c becifie d seleving relat are id to the f to the f the f (/As t	ed in ecting e to lentificateg elect he aj ers, a RO lio pelow	the tag more plant plant plant plical a brief cense v 2.5 s	able that spece the er. ble e f des leve	an tw cific p asso ectio evolu script el, an Id be	o or t prioriti ciate ion 2 o ion o id the justi	s than hree K/A es. ed outline. f the K/A or system. f each point fied on

NUREG-1021, Revision 8

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ES-401			Emerge	PW ncy and /	/R SRO I Abnorma	Examinatio I Plant Evo	on Outline Ilutions - Tier 1/Group 1	Forr	m ES-401-3
E/APE # / Name / Safety Function	К1	К2	КЗ	A1	A2	G	K/A Topic(s)	Imp.	Points
000001 Continuous Rod Withdrawal / 1						2.1.33	Recognize entry level conditions for TS	4.0	1
000003 Dropped Control Rod / 1						2.1.20	A/T execute procedure steps	4.2	1
000005 Inoperable/Stuck Control Rod / 1				A 05			Operate/monitor the RPI as it applies to a stuck/inop rod	3.4	1
000011 Large Break LOCA / 3					E 08		Conditions for recovery when accident reaches a stable phase	4.3	1
W/E04 LOCA Outside Containment / 3	E2						Normal, abn, and EOP with LOCA outside containment	4.2	1
W/EO1 & E02 Rediagnosis & SI Termination / 3					E2		Adh to proc & ops within limitation in license & amendments	3.9	1
000015/17 RCP Malfunctions / 4			A 03				Seq of event for manually tripping RX and RCP	4.0	1
BW/E09; CE/A13; W/E09&E10 Natural Circ. / 4	E1						Components, capacity, and function of emergency systems	3.6	1
000024 Emergency Boration / 1						2.4.18	K/O the specific basis for EOPs	3.6	1
000026 Loss of Component Cooling Water / 8					A 06		Length of time after the loss of CCW flow until damage equip	3.1	1
000029 Anticipated Transient w/o Scram / 1					E 09		Determine/interpret the occ. of a Turb/Rx trip with ATWS	4.5	1
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4			E 4				Control room proc. Are adhered to and limitations of the license and amendments are not violated	3.8	1
CE/A11; W/E08 RCS Overcooling - PTS / 4		E1					Components including instrumentation, signals, interlocks, failure modes, and automatic, and manual features	3.7	1
000051 Loss of Condenser Vacuum / 4			A01				Loss of steam dump capability upon loss of condenser vacuum	3.1	1
000055 Station Blackout / 6				E 04			Ability to operate and monitor the red. of loads on the battery	3.9	1
000057 Loss of Vital AC Elec. Inst. Bus / 6				A 06			Manual control of components for which auto control is lost	3.5	1
000059 Accidental Liquid RadWaste Rel. / 9		A 01					Interrelations between acc liq radwaste release and rad-liq monitors	2.8	1
000062 Loss of Nuclear Service Water / 4			A01				Conditions that will initiate the automatic opening & closing of the SWS insolation valves to the nuclear service water coolers	3.5	1
000067 Plant Fire On-site / 9					A 11		Time limit for use of respirators	3.5	1
000068 (BW/A06) Control Room Evac. / 8			A13				Performing SDM calculation including boron needed and time	3.9	1
000069 (W/E14) Loss of CTMT Integrity / 5						2.4.47	Diagnose/recognize trends in an accurate and timely manner	3.7	1
000074 <del>(W/E06&amp;E07)</del> Inad. Core Cooling / 4	E07						Definition of saturated steam	3.2	1
000076 High Reactor Coolant Activity / 9		A01					Knowledge of the interactions between high RCS activity and process radiation monitors	3.0	1
000026 Loss of Component Cooling Water / 8					A 03		Determine valve lineup to restart CCW while bypassing comp.	2.9	1
K/A Category Totals:	3	3	5	3	6	4	Group Point Total:	ĺ	24

ES-401		E	Emergen	PWI cy and A	R SRO E bnormal	xaminatior Plant Evol	n Outline utions - Tier 1/Group 2	Form	n ES-401-3
E/APE # / Name / Safety Function	К1	К2	КЗ	A1	A2	G	K/A Topic(s)	Imp.	Points
000007 (BW/E02&E10 CE/E02) Reactor Trip - Stabilization - Recovery / 1					E 05		Rx trip first-out indication	3.9	1
000008 Pressurizer Vapor Space Accident / 3					A 09		Pzr spray block valve controls and indicators	3.7	1
000009 Small Break LOCA / 3				E 12			Operate and monitor the RPS as applies to a SBLOCA	4.2	1
BW/E08; W/E03 LOCA Cooldown - Depress. / 4						2.4.41	K/O emergency action level thresholds and classifications	4.1	1
W/E11 Loss of Emergency Coolant Recirc. / 4			E2				Normal, abn, & emerg oper proc associated with loss of recirc	4.0	1
000022 Loss of Reactor Coolant Makeup / 2									
000025 Loss of RHR System / 4			A 03				Immediate actions contained in EOP for loss of RHR	4.1	1
000027 Pressurizer Pressure Control System Malfunction / 3					A 06		Determine/interpret conditions requiring plant shutdown	3.9	4
000032 Loss of Source Range NI / 7			A 01				Startup termination on source range loss	3.6	1
000033 Loss of Intermediate Range NI / 7					A 09		Determine/Interpret conditions which allow bypass of inter- range level trip switch	3.7	1
000037 Steam Generator Tube Leak / 3									
000038 Steam Generator Tube Rupture / 3				E 09			Pzr tank level/press indicators, gauges, and recorders	3.3	1
000054 (CE/E06) Loss of Main Feedwater / 4				A 04			Operate HPI, under total feedwater loss conditions	4.5	1
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4	-				E 1		Facility conditions and selection of app proc during abn & emerg ops	4.4	1
000058 Loss of DC Power / 6	A 01						Battery charger equipment and instrumentation	3.1	1
000060 Accidental Gaseous Radwaste Rel. / 9		A 02					Interrelations between the accident gaseous radwaste release and Aux building ventilation	3.1	1
000061 ARM System Alarms / 7				A 01			Automatic actuation	3.6	1
W/E16 High Containment Radiation / 9									
000065 Loss of Instrument Air / 8						2.1.7	Evaluate plant perf and make oper judgements based on oper characteristics, reactor behavior, and instrument interpretation	4.4	1
K/A Category Point Totals:	1	1	3	4	5	2	Group Point Total:		16

ES-401		E	imerger	PV ncy and	VR SR( Abnorm	D Examina nal Plant E	ation Outline volutions - Tier 1/Group 3	Form	n ES-401-3
E/APE # / Name / Safety Function	К1	К2	КЗ	A1	A2	G	K/A Topic(s)	Imp.	Points
000028 Pressurizer Level Malfunction / 2	A 01						PZR reference leak abnormalities	3.1	1
000036 (BW/A08) Fuel Handling Accident / 8						2.3.10	A/T perform procedures to reduce excessive levels of radiation and guard against personnel exposure	3.3	1
000056 Loss of Off-site Power / 6							· · ·		
BW/E13&E14 EOP Rules and Enclosures									
BW/A05 Emergency Diesel Actuation / 6									
BW/A07 Flooding / 8									
CE/A16 Excess RCS Leakage / 2									
W/E13 Steam Generator Over-pressure / 4									
W/E15 Containment Flooding / 5	E 3						Annuciators and conditions indicating signals, and remedial actions associated with the steam generator overpressure	3.2	1
·									
							· · · · · · · · · · · · · · · · · · ·		
K/A Category Point Totals:	2					1	Group Point Total:		3

29 of 45

ES-401			100					aminati - Tier					Form	n ES-401-3
System # / Name	K1	К2	КЗ	K4	K 5	K6	A1	A2	A3	A4	G	K/A Topic(s)	Imp.	Points
001 Control Rod Drive					96							Sign change in react when react is added	3.4	1
003 Reactor Coolant Pump							09					Seal flow and D/P	2.8	1
004 Chemical and Volume Control			06									Effect that loss/malfunction of CVCS on RCS	3.6	1
013 Engineered Safety Features Actuation				15		<u> </u>		<u> </u>				Design/interlocks for continuous testing	3.2	1
014 Rod Position Indication											2.4.21	Parameters and logic used to assess the status of safety functions	4.3	1
015 Nuclear Instrumentation	08											Relationship - NIS and RCS (pump start)	2.9	1
017 In-core Temperature Monitor						ļ		01				Thermocouple open and short circuits	3.5	1
022 Containment Cooling										01		Manually operate/monitor fans in Control Rm	3.6	1
026 Containment Spray		01										Bus pwr supplies to CS pumps	3.6	1
056 Condensate	03											Relationship between cond and MFW	2.6	1
059 Main Feedwater			02									Effect of loss/malfunction of MFW on AFW	3.7	1
061 Auxiliary/Emergency Feedwater		01										Bus pwr supplies to AFW MOVs	3.3	1
063 DC Electrical Distribution				04								Design/interlocks that provide for trips	2.9	1
068 Liquid Radwaste					04							Bio hazards of rad & goal of ALARA	3.5	1
071 Waste Gas Disposal				04								Isolation of waste gas release tanks	3.4	1
072 Area Radiation Monitoring									01			Monitor changes in ARM vent alignment	3.1	1
026 Containment Spray								05				Predict impact failure of chemical add tanks to inject	4.1	1
061 Auxiliary/Emergency Feedwater											2.2.25	K/O basis - TS for LCO and safety limits	3.7	1
015 Nuclear Instrumentation								04				Impact on NIS amd mitigate consequences	3.8	1
K/A Category Point Totals:	2	2	2	3	2	0	1	3	1	1	2	Group Point Total:		19

ES-401 PWR SRO Examination Outline Plant Systems - Tier 2/Group 2										Form ES-401-3				
System # / Name	К1	К2	КЗ	К4	К5	K6	A1	A2	A3	A4	G	K/A Topic(s)		Points
002 Reactor Coolant						12						Effect of loss/malfunction of code SRVs		1
006 Emergency Core Cooling	09	<u> </u>	ļ									Relationship between ECCS and N <sub>2</sub>		1
010 Pressurizer Pressure Control							03					Predict/monitor change in PRT press/temp		1
011 Pressurizer Level Control							01					Predict/monitor change in PZR level/press	3.6	1
012 Reactor Protection	03				<b>_</b>							Relationship between RPS and CRDS	3.8	1
016 Non-nuclear Instrumentation			12									Effect loss/malfunction of NNIS on S/G	3.6	1
027 Containment Iodine Removal								<u> </u>						
028 Hydrogen Recombiner and Purge Control						01						Effect of loss/malf of H2 recomb on HRPS		1
029 Containment Purge										04		Manually operate cont evacuation signal	3.6	1
033 Spent Fuel Pool Cooling														
034 Fuel Handling Equipment											2.2.32	K/O the effects of alterations of core configuration	3.3	1
035 Steam Generator								05				Impact of unbalanced flows to S/Gs	3.4	1
039 Main and Reheat Steam								04				Impact of malfunctioning stm dumps	3.7	1
055 Condenser Air Removal									03			Monitor automatic diversion of CARS 2 exhaust		1
062 AC Electrical Distribution			01									Major system loads	3.9	1
064 Emergency Diesel Generator											2.2.18	K/O process for managing maint act during shutdown ops	3.6	1
073 Process Radiation Monitoring														
075 Circulating Water												·		
079 Station Air				01								Cross connect with IAS	3.2	1
086 Fire Protection								02				Impact of low FPS header pressure	3.3	1
103 Containment							01					Predict/monitor changes in containment press, temp, and humididty	4.1	1
K/A Category Point Totals:	2	0	2	1	0	2	3	3	1	1	2	Group Point Total:		17

ES-401 PWR SRO Examination Outline Form ES- Plant Systems - Tier 2/Group 3										n ES-401-3				
System # / Name	К1	К2	КЗ	K4	K5	K6	A1	A2	A3	A4	G	K/A Topic(s)	Imp.	Points
005 Residual Heat Removal														
007 Pressurizer Relief/Quench Tank														
008 Component Cooling Water														
041 Steam Dump/Turbine Bypass Control														
045 Main Turbine Generator														
076 Service Water														
078 Instrument Air														
K/A Category Point Totals:												Group Point Total:		4
						Plant	-Specif	ic Prio	rities					
System / Topic						Recommended Replacement for						Reason		Points
				·										
· · · · · · · · · · · · · · · · · · ·														
										<u> </u>				
Plant-Specific Priority Total: (limit 10)														

ES-401

## Generic Knowledge and Abilities Outline (Tier 3)

Form ES-401-5

Facility: CPSE	<u> </u>	Date of Exam: 05/2001 Exam	Level:	RO					
Category	K/A #	Торіс	Imp.	Points					
	2.1.2	K/O Operator responsibilities during all modes of plant operation	1						
Conduct of Operations	2.1.9	A/T Direct personnel activities inside control room	1						
	2.1.23	A/T Perform specific system integrated plant procedures during all modes	3.9	1					
	2.1.								
	Total								
	2.2.12	K/O Surveillance procedures	3.0	1					
	2.2.13	K/O Clearance and tagging procedures	3.6	1					
Equipment Control	2.2.30	K/O RO duties in control room during fuel handling	3.5	1					
	2.2.								
	2.2.								
	Total								
Radiation Control	2.3.2	K/O Facility ALARA Program	2.5	1					
	2.3.4	K/O radiation exposure limits and contamination control	2.5	1					
	2.3.9	K/O process for performing cont. purge	2.5	1					
	2.3.10	A/T Perform procedures to reduce excessive levels of radiation	2.9	1					
l	Total								
Emergency Procedures/ Plan	2.4.3	Ability to identify post-accident instrumentation	3.5	1					
	2.4.26	K/O Facility protection requirements including fire brigade and portable fire fighting equip use	2.9	1					
	2.4.43	K/O Emergency communications systems and techniques	2.8	1					
	2.4.								
	Total			3					
Tier 3 Point T	otal (RO)			13					

ES-401

Form ES-401-5

Facility: CPSE	S	Date of Exam: 05/2001 Exam	Level:	SRO					
Category	K/A #	Торіс	imp.	Points					
	2.1.4	K/O shift staffing requirements	3.4	1					
Conduct of	2.1.9	A/T Direct personnel activities inside control room	4.0	1					
Operations	2.1.11	K/O Less than 1 hour TS action statements	3.8	1					
	2.1.12	A/T apply Technical Specifications for a system	4.0	1					
	2.1.32	A/T explain/apply system limits & precautions	3.8	1					
	Total			5					
	2.2.8	K/O determining if change involves a USQ	3.3	1					
1	2.2.12	K/O Surveillance procedures	3.4	1					
Equipment Control	2.2.13	K/O clearance and tagging procedures	3.8	1					
	2.2.24	A/T analyze affect of maint on LCO status	3.8	1					
	2.2.28	K/O new and spent fuel movement proc.	3.5	1					
	Total								
	2.3.2	K/O Facility ALARA Program	2.9	1					
	2.3.4	K/O radiation exposure limits and contamination control	3.1	1					
Radiation	2.3.11	A/T control radiation releases	3.2	1					
Control	Total		3						
	2.4.7	K/O event based EOP mitigation strategies	3.8	1					
Emergency Procedures/ Plan	2.4.26	K/O Facility protection requirements including fire brigade and portable fire fighting equip use	3.3	1					
	2.4.43	K/O Emergency communications systems and techniques	3.5	1					
	2.4.48	A/T understand impact of oper actions and affect on the plant	3.8	1					
	Total								
Tier 3 Point T	otal (SRO	)		17					

NUREG-1021, Revision 8