March 21, 1990

Docket No. 50-305

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Mr. Ken H. Evers
Manager - Nuclear Power
Wisconsin Public Service
Corporation
P.O. Box 19002
Green Bay, Wisconsin 54037-9002

Dear Mr. Evers:

SUBJECT: CORRECTION TO AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. DPR-43 (TAC NO. 75891)

On March 19, 1990, the Commission issued Amendment No. 85 to Facility Operating License No. DPR-43 for the Kewaunee Nuclear Power Plant. The amendment revised the Technical Specifications in response to your application dated January 29, 1990.

Page TS 3.8-3 contained a typographical error in the second line of the second paragraph under "Basis" in that the reference to TS 2.8.a.5 should have read "TS 3.8.a.5." A corrected page TS 3.8-3 is enclosed.

Please accept our apologies for any inconvenience this error may have caused you.

Sincerely,

/s/

Michael J. Davis, Project Manager
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
TS page 3.8-3

cc: See next page

DOCUMENT NAME: AMD 85 CORRECTION

Office:	LA/PDIII-3	PM/PDIII-3	PD/PDIII-3
Surname:	PKreutzer	MDavis/tg	JHannon
Date:	3/20/90	3/20/90	3/21/90

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Manager - Nuclear Power
Wisconsin Public Service
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Project Directorate III-3
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TS page 3.8-3

cc: See next page

Mr. Ken H. Evers
Wisconsin Public Service Corporation

Kewaunee Nuclear Power Plant

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U.S. Nuclear Regulatory Commission
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Glen Ellyn, Illinois 60137

Mr. Robert S. Cullen
Chief Engineer
Wisconsin Public Service Commission
P.O. Box 7854
Madison, Wisconsin 53707

12. A licensed senior reactor operator will be on site and designated in charge of the refueling operation.

b. If any of the specified limiting conditions for refueling are not met, refueling of the reactor shall cease. Work shall be initiated to correct the violated conditions so that the specified limits are met, and no operations which may increase the reactivity of the core shall be performed.

Basis

The equipment and general procedures to be utilized during refueling are discussed in the USAR. Detailed instructions, the above specified precautions, and the design of the fuel handling equipment incorporating built-in interlocks and safety features, provide assurance that no incident occurs during the refueling operations that would result in a hazard to public health and safety.(1) Whenever changes are not being made in core geometry, one flux monitor is sufficient. This permits maintenance of the instrumentation. Continuous monitoring of radiation levels (2 above) and neutron flux provides immediate indication of an unsafe condition. The residual heat removal pump is used to maintain a uniform boron concentration.

A minimum shutdown margin of greater than or equal to 5% $\Delta k/k$ must be maintained in the core. A boron concentration of 2100 ppm, as required by TS 3.8.a.5, is sufficient to maintain a typical core shutdown by approximately 10% $\Delta k/k$ and is specified to ensure an adequate margin of safety. The specification for refueling shutdown margin is based on a dilution during refueling accident.(4) With an initial shutdown margin of 5% $\Delta k/k$, under the postulated accident conditions, it will take approximately 61 minutes for the reactor to go critical. This is ample time for the operator to recognize the audible high count rate signal, and isolate the reactor makeup water system. Periodic checks of refueling water boron concentration ensure that proper shutdown margin is maintained. Specification 3.8.a.6 allows the control room operator to inform the manipulator operator of any impending unsafe condition detected from the main control board indicators during fuel movement.

Interlocks are utilized during refueling to ensure safe handling. Only one assembly at a time can be handled. The fuel handling hoist is dead weight tested prior to use to assure proper crane operation. It will not be possible to lift or carry heavy objects over the spent fuel pool when fuel is stored