

February 3, 1987

Docket No. 50-305

Mr. D. C. Hintz  
Manager, Nuclear Power  
Wisconsin Public Service Corporation  
Post Office Box 19002  
Green Bay, Wisconsin 54307-9002

Dear Mr. Hintz:

The Commission has issued the enclosed Amendment No. 72 to Facility Operating License No. DPR-43 for the Kewaunee Nuclear Power Plant. This amendment is in response to your application dated September 30, 1986.

The amendment removes requirements from the Technical Specifications (TS) that are duplicated in the Operational Quality Assurance Program. In addition, a recent change in a management title is made on three pages of the TS. This action completes our TAC No. 63067.

This amendment also deletes Section 6.5.2.4 of the TS. This section delineates the authority of the Corporate Nuclear Engineering Staff (CNES). The licensee has agreed to add the CNES authority to the Operational Quality Assurance Program Description within 60 days of the date of this license amendment.

A copy of our related Safety Evaluation is also enclosed. This action will appear in the Commission's next regular biweekly notice publication in the Federal Register.

Sincerely,

/s/

Morton B. Fairtile, Project Manager  
Project Directorate #1  
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 72 to License No. DPR-43
2. Safety Evaluation

cc w/enclosures:  
See next page

Office: LA/PAD#1:DPLA  
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Date: 01/13/87

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Mr. D. C. Hintz  
Wisconsin Public Service Corporation

Kewaunee Nuclear Power Plant

cc:  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

DOCKET NO. 50-305

KEWAUNEE NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72  
License No. DPR-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Public Service Corporation, Wisconsin Power and Light Company, and Madison Gas and Electric Company (the licensees) dated September 30, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-43 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 72, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Morton B. Fairtile*

Morton B. Fairtile, Project Manager  
Project Directorate #1  
Division of PWR Licensing-A

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 3, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 72

FACILITY OPERATING LICENSE NO. DPR-43

DOCKET NO. 50-305

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

iv  
viii  
6-1  
6-5  
6-7  
6-11a  
Figure TS 6.2-1  
Figure TS 6.2-2

INSERT

iv  
viii  
6-1  
6-5  
6-7  
6-11a  
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<u>Section</u>	<u>Title</u>	<u>Page TS</u>
5.0	Design Features	5.1-1
5.1	Site	5.1-1
5.2	Containment	5.2-1
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5.2.b	Reactor Containment Vessel	5.2-2
5.2.c	Shield Building	5.2-2
5.2.d	Shield Building Ventilation System	5.2-2
5.2.e	Auxiliary Building Special Ventilation Zone and Special Ventilation System	5.2-3
5.3	Reactor	5.3-1
5.3.a	Reactor Core	5.3-1
5.3.b	Reactor Coolant System	5.3-2
5.4	Fuel Storage	5.4-1
6.0	Administrative Controls	6-1
6.1	Responsibility	6-1
6.2	Organization	6-1
6.2.1	Off-Site	6-1
6.2.2	Facility Staff	6-1
6.2.3	Organizational Changes	6-2
6.3	Plant Staff Qualifications	6-2
6.4	Training	6-2a
6.5	Review and Audit	6-2a
6.5.1	Plant Operations Review Committee (PORC)	6-2a
6.5.1.1	Function	6-2a
6.5.1.2	Composition	6-3
6.5.1.3	Alternates	6-3
6.5.1.4	Meeting Frequency	6-3
6.5.1.5	Quorum	6-3
6.5.1.6	Responsibilities	6-4
6.5.1.7	Authority	6-5
6.5.1.8	Records	6-5
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6.5.2.1	Function	6-5
6.5.2.2	Organization	6-6
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6.5.2.4	Deleted	
6.5.3	Nuclear Safety Review and Audit Committee	6-7
6.5.3.1	Function	6-7
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6.5.3.3	Alternates	6-8
6.5.3.4	Consultants	6-8
6.5.3.5	Meeting Frequency	6-8
6.5.3.6	Quorum	6-9
6.5.3.7	Review	6-9
6.5.3.8	Audits	6-10
6.5.3.9	Authority	6-11
6.5.3.10	Records	6-11
6.6	Reportable Events	6-11a
6.6.1	Actions	6-11a

## LIST OF FIGURES

<u>Figure - TS</u>	<u>Titles</u>
2.1-1	Safety Limits Reactor Core, Thermal and Hydraulic
3.1-1	Reactor Coolant System Heat-up Limitations
3.1-2	Reactor Coolant System Cool-down Limitations
3.1-3	Deleted
3.1-4	Deleted
3.10-1	Required Shut-down Reactivity vs. Reactor Boron Concentration
3.10-2	Hot Channel Factor Normalized Operating Envelope
3.10-3	Control Rod Insertion Limits as a Function of Power
3.10-4	Permissible Operating Band on Indicated Flux Difference as a Function of Burn-up (Typical)
3.10-5	Target Band on Indicated Flux Difference as a Function of Operating Power Level (Typical)
3.10-6	V(Z) as a function of Core Height
3.10-7	Deleted
6.2-1 (TS 6-26)	Deleted
6.2-2 (TS 6-27)	Deleted

## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The Plant Manager has overall on-site responsibility for plant operation. In the absence of the Plant Manager, the succession to this responsibility shall be in the following order:

- a. Assistant Manager-Plant Operations
- b. Assistant Manager-Plant Maintenance
- c. Superintendent-Plant Operations
- d. Assistant Manager-Plant Technical and Services
- e. Shift Supervisor

### 6.2 ORGANIZATION

6.2.1 The offsite organization for plant management and technical support shall be as described in the Operational Quality Assurance Program Description.

#### FACILITY STAFF

6.2.2 The plant organization shall be as described in the Operational Quality Assurance Program Description.

- a. Each on-duty shift complement shall consist of at least:
  - (1) One Shift Supervisor (SRO)
  - (2) Two licensed Reactor Operators
  - (3) One Auxiliary Operator
  - (4) One Equipment Operator
  - (5) One Radiation Technologist
- b. While above cold shutdown, the on-duty shift complement shall consist of the personnel required by 6.2.2a. above and an additional SRO.
- c. In the event that one of the shift members becomes incapacitated due to illness or injury or the Radiation Technologist has to accompany an injured person to the hospital, reactor operations may continue with the reduced complement until a replacement arrives. In all but severe weather conditions, a replacement is required within two hours.



- i. Review of all Reportable Events
- j. Review of changes to the Process Control Program, the Offsite Dose Calculation Manual, and the Radiological Environmental Monitoring Manual.

#### AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend to the Plant Manager approval or disapproval of items considered under 6.5.1.6a through e above.
- b. Make determinations with regard to whether or not each item considered under 6.5.1.6 above constitutes an unreviewed safety question.
- c. Provide immediate notification in the form of draft meeting minutes to the Vice President - Nuclear Power and the Chairman-Nuclear Safety Review and Audit Committee of disagreement between the PORC and the Plant Manager. The Plant Manager shall have responsibility for resolution of such disagreements.

#### RECORDS

6.5.1.8 Minutes shall be kept of all meetings of the PORC and copies shall be sent to the Vice President - Nuclear Power and the Chairman - Nuclear Safety Review and Audit Committee.

### 6.5.2 CORPORATE NUCLEAR ENGINEERING STAFF (CNES)

#### FUNCTION

6.5.2.1 The CNES shall function to provide engineering,

6. Review and/or prepare safety evaluations of all plant design changes.
7. Audits as required by the Quality Assurance Program and as outlined in Section 6.5.3.8.

6.5.2.4 Deleted

### 6.5.3 NUCLEAR SAFETY REVIEW AND AUDIT COMMITTEE (NSRAC)

#### FUNCTION

6.5.3.1 The NSRAC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear Power Plant Operations
- b. Nuclear Engineering
- c. Chemistry and Radio-Chemistry
- d. Metallurgy
- e. Instrumentation
- f. Radiological Safety
- g. Mechanical and Electrical Engineering
- h. Quality Assurance Practices
- i. Other appropriate fields as determined by the Committee, to be associated with the unique characteristics of the nuclear power plant.

#### COMPOSITION

6.5.3.2 The NSRAC shall be composed of, but not necessarily limited to:

## 6.6 REPORTABLE EVENTS

### Actions

6.6.1 The following actions shall be taken for Reportable Events:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of 10CFR50.73, and
- b. Each Reportable Event shall be reviewed by PORC, and the results of this review shall be submitted to NSRAC and the Vice President - Nuclear Power.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO.72 TO FACILITY OPERATING LICENSE NO. DPR-43

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

KEWAUNEE NUCLEAR POWER PLANT

DOCKET NO. 50-305

1.0 INTRODUCTION

By submittal letter dated September 30, 1986, Wisconsin Public Service Corporation, et al, (WPSC or the licensees) requested certain changes be made to the Administrative Controls section of the Technical Specifications for the Kewaunee Nuclear Power Plant. The requested changes involved deletion of organization information and figures that are repeated in the WPSC Operational Quality Assurance Program Description (OQAPD) plus one position title change and index changes.

The staff's evaluation of the requested changes is given below.

2.0 EVALUATION

2.1 Offsite Organization

The licensees have proposed to delete or revise the following Kewaunee Technical Specifications administrative organization information:

- ° Revise Section 6.2.1 to refer to the OQAPD for the organization rather than referring to Figure TS 6.2-1.
- ° Delete Figure TS 6.2-1.

The current OQAPD contains an organization chart identical to the figure deleted above. This proposed change then eliminates the redundancy of offsite organization charts while continuing review by the Nuclear Regulatory Commission (NRC) is assured by the requirements of 10 CFR 50.54(a).

The staff concludes that this change: (1) does not reduce the technical support being provided for Kewaunee, (2) continues to provide adequate management span of control, and (3) continues to meet the acceptance criteria of NUREG-0800 "Standard Review Plant," Sections 17.1, Item 1A5 and 17.2 for an operating plant.

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## 2.2 Station Organization

The licensees have proposed to delete or revise the following Kewaunee Technical Specifications administrative plant organization information:

- ° Revise Section 6.2.2 to make reference to the OQAPD for the organization chart.
- ° Delete Figure TS 6.2.2.
- ° Delete Section 6.5.2.4, the Corporate Nuclear Engineering Staff (CNES) authority.

This proposed change eliminates the redundancy of station organization charts since the current OQAPD contains an organization chart equal to the deleted figure. The licensee shall add the CNES authority to the OQAPD within 60 days of the date of this license amendment. This includes the following:

- ° Nuclear Design Change, Nuclear Services, Nuclear Training, Kewaunee Plant, Nuclear Administration, and Nuclear Licensing and Systems report to the Vice President-Nuclear Power.
- ° Members of the Fuel Services, Environmental Services, and System Planning and Engineering Groups, although not directly responsible to the Vice President-Nuclear Power are available for special projects and support to the Kewaunee Plant.
- ° The Vice President-Nuclear Power, along with Power Plant Design and Construction, Quality Assurance, and Nuclear Safety and Audit Committee are all responsible to the Vice President-Power Production.

The staff concludes that the deletions and the proposed changes, when completed will not reduce technical support for Kewaunee and that continuing review by the NRC is assured by the 10 CFR 50.54(a) requirements.

## 2.3 Index and Title Changes

The following proposed title and index changes are acceptable.

- ° Index pages TS iv and TS viii reflect the deletion of Section 6.5.2.4 and of Figures 6.2-1 and 6.2-2.
- ° The title change from Manager-Nuclear Power to Vice President-Nuclear Power occurs in Sections 6.5.1.7.c, 6.5.1.8, and 6.6.1.6.

The title changes and index changes do not affect safety and are, therefore, acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to administrative controls. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(7). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

### 5.0 ACKNOWLEDGEMENT

Principal Contributor:  
T. E. Vandel

Dated: February 3, 1987

Feb 3, 1977

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