Docket File
50-305

JANUARY 19 1979

Docket No. 50-305

Mr. E. W. James
Senior Vice President
Wisconsin Public Service Corporation
Post Office Box 1200
Green Bay, Wisconsin 54305

Dear Mr. James:

The Commission has issued the enclosed Amendment No. 25 to Facility Operating License No. DPR-43 for the Kewaunee Nuclear Power Plant. This amendment consists of changes to the Technical Specifications in response to your request dated November 10, 1978.

The amendment corrects the definition of rated power, updates the listing of safety-related shock suppressors, corrects several errors in the listing of fire detectors, corrects the minimum auto start pressure for the fire pumps, and deletes a definition relevant to Cycle 1 operation.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original Signed By

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:
1. Amendment No. 25 to DPR-43
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 19, 1979

Docket No. 50-305

Mr. E. W. James
Senior Vice President
Wisconsin Public Service Corporation
Post Office Box 1200
Green Bay, Wisconsin 54305

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A. Schwencer, Chief
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Division of Operating Reactors

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cc: w/enclosures
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

DOCKET NO. 50-305

KEWAUNEE NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25
License No. DPR-43

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Public Service Corporation, Wisconsin Power and Light Company and Madison Gas and Electric Company (the licensee) dated March 20, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility License No. DPR-43 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 25 , are hereby incorporated in the license.. The licensee shall operate the facility in accordance with the Technical Specifications."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 19, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 25

FACILITY OPERATING LICENSE NO. DPR-43

DOCKET NO. 50-305

Remove the following pages and replace with identically numbered pages.

TS 1.1-5
Table TS 3.14-1
(pages 1-11)

Table TS 3.15-1
TS 4.14-2
TS 4.14-3a
TS 4.15-2

l. Refueling Operation

Refueling operation is any operation involving movement of Reactor Vessel internal components (those that could affect the reactivity of the core) within the containment when the vessel head is unbolted or removed.

m. Rated Power

Rated power is the steady-state reactor core output of 1650 MWt.

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 1 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
Steam Supply to Turbine Driven Aux. FW Pump (MS)	SS-H-67	6'-1 3/4" E of col. (E) 1'-10 3/4" S of col. (9) El. 593'-0"	A		
(MS)	SS-H-72	6'-1 3/4" E of col. (E) 0'-0" of col. (8) El. 594'-10 1/2"	A		
(MS)	SS-H-73	6'-1 3/4" E of col. (E) 0'-0" of col. (8) El. 594'-10 1/2"	A		
(MS)	SS-H-76	2'-6" E of col. (HE) 1'-6" S of col. (4) El. 603'-1"	A		
(MS)	SS-H-86	8'-7" E of col. (G) 0'-0" of col. (8) El. 588'-1"	A		
(MS)	SS-H-87	9'-10" E of col. (G) 0'-0" of col. (8) El. 588'-1"	A		
(MS)	SS-H-88	9'-10" E of col. (G) 0'-0" of col. (8) El. 588'-6"	A		
(MS)	SS-H-103	2'-4" E of col. (G) 3'-0" N of col. (5) El. 593'-3"	A		
(MS)	SS-H-129	2'-6" E of col. (GW) 0'-0" of col. (6) El. 638-2 3/4"	A	X	

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 2 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
(MS)	SS-H-146	10'-8" W of col. (K) 2'-5 1/2" S of col. (6) El. 634'- 9 1/2"	A	X	
(MS)	SS-H-150	3" N of col. (6s) 1'-11" W of col. (LW) El. 635'-0"	A		
Reactor Coolant (RC)	Anchor Holth 900K Custom Suppressors 1 thru 8	Lateral Restraint For Steam Generators (4 on Each)	I	X	X
(RC)	RC-H-29A	0'-3" E of col. (S) 16'-5" S of col. (E) El. 603'-0"	I		
(RC)	RC-H-37	Inside Containment SE Quadrant Top of Pressurizer El. 657'-7"	I		
(RC)	RC-H-38	Inside Containment SE Quadrant Bottom of Pressurizer El. 602'-0"	I		
(RC)	RC-H-39	Inside Containment SE Quadrant Bottom of Pressurizer El. 602'-0"	I		
(RC)	RC-H-40	Inside Containment SE Quadrant Top of Pressurizer El. 659'-7"	I		

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 3 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
(RC)	RTD-H-2	7'-10" E from center line of steam generator 1A 6'-0" S from center line of steam generator 1A El. 619'-0"	I		X
(RC)	RTD-H-6	15'-3 1/2" E from center line of steam generator 1A 11'-9" S from center line of steam generator 1A El. 615'-3 3/16"	I		X
(RC)	RTD-H-8	9'-10" N of center line of steam generator 1B 10'-9" W of center line of steam generator 1B El. 616'-0"	I		X
(RC)	RTD-H-11	6'-2" N from center line of steam generator 1B 6'-3" W from center line of steam generator 1B El. 616'-10 1/4"	I		X
Residual Heat Removal (RHR)	RHR-H-10H	20'-9" N of col. (6) 21'-0" E of col. (K) El. 601'-0"	I		X
(RHR)	RHR-H-12A	8'-9 1/2" N of col. (6) 25'-2" E of col. (K) El. 626'-6"	I		X

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 4 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
(RHR)	RHR-H-12B	9'-7 1/2" N of col. (6) 25'-2" E of col. (K) El. 626'-6"	I		X
(RHR)	R-RHR-H-14	36'-1" N of col. (W) 18'-6 5/8" E of col. (N) El. 607'-6", inside containment	I		
(RHR)	R-RHR-H-15	36'-0 3/8" N of col. (W) 18'-6 5/8" E of col. (N) El. 607'-6", inside containment	I		
(RHR)	RHR-H-16A	5'-2 1/2" N of col. (6) 12'-8 1/2" E of col. (K) El. 617'-9"	I	X	X
(RHR)	R-RHR-H-18	27'-2 7/16" N of col. (E) 23'-4 7/16" E of col. (N) El. 611'-0", inside containment	I		X
(RHR)	RHR-H-21A	14'-6" N of col. (6) 24'-7" E of col. (K) El. 598'-6"	I		
(RHR)	RHR-H-35A	14'-3 1/2" N of col. (6) 5'-4" E of col. (K) El. 575'-0"	A		X

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 5 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
Residual Heat Removal (RHR)	RHR-H-36A	5'-0" S of col. (6N) 15'-0" E of col. (K) El. 576'-6"	A		X
(RHR)	RHR-H-38A	14'-3 1/2" N of col. (6) 19'-3" E of col. (K) El. 576'-6"	A		X
(RHR)	RHR-H-41A	7'-5" N of col. (6) 17'-11 5/8" E of col. (K) El. 576'-6"	A		X
(RHR)	RHR-H-49	20'-9" N of col. (6) 22'-1" E of col. (K) El. 601'-6"	I		X
(RHR)	R-RHR-H-55	31'-10" N of col. (W) 12'-7" E of col. (N) El. 598'-1", inside containment	I		
(RHR)	R-RHR-H-57	29'-0" N of col. (W) 8'-11" E of col. (N) El. 592'-0", inside containment	I		
Aux. Feedwater Pump Turbine Driven Exhaust	FWP-H-14	11'-2" S of col. (9) 2'-10 1/2" W of col. (E) El. 600'-1"	A		

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 6 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
S.G. Blowdown (SGB)	SGB-H-137	Inside Containment NW Quadrant El. 592'-0"	I		
Safety Injection (SI)	SI-H-6D	3'-0" S of col. (6) 1'-6" W of col. (HW) El. 629'-11 3/4"	A		X
(SI)	SI-H-35	2'-0" N of col. (6) 15'-11 3/8" E of col. (K) El. 606'-9"	I		
(SI)	RSI-H-2	46'-11 1/2" N of col. (W) 16'-5 3/8" E of col. (N) El. 606'-6"	I		
(SI)	RSI-H-2A	46'-11 1/2" N of col. (W) 16'-5 3/8" E of col. (N) El. 607'-0"	I		
(SI)	RSI-H-15A	1'-6" N of col. (W) 22'-2" W of col, (N) El. 602'-2"	I		
(SI)	RSI-H-38	31'-2 3/4" N of col. (E) 3'-5 1/2" E of col. (N) El. 607'-5"	I		
(SI)	RSI-H-59	13'-4 5/8" N of col. (E) 34'-3 3/4" E of col. (N) El. 621'-8 1/4"	I	X	
(SI)	RSI-H-61	10'-6" N of col. (E) 36'-9 7/8" E of col. (N) El. 593'-6"	I		

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 7 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
(SI)	RSI-H-63	36'-9 7/8" E of col. (N) 2'-10 7/8" S of col. (E) El. 593'-6"	I		
(SI)	RSI-H-67	10'-8 7/8" S of col. (E) 23'-9 1/4" E of col. (N) El. 603'-7"	I		
(SI)	RSI-H-78	34'-7 1/4" N of col. (W) 15'-9 1/2" E of col. (N) El. 601'-3"	I		
(SI)	RSI-H-83	17'-5" N of col. (W) 0'-5 3/4" E of col. (N) El. 601'-0"	I		X
(SI)	RSI-H-94	Inside Containment NE Quadrant El. 620'-5"	I	X	
(SI)	RSI-H-95	Inside Containment NE Quadrant El. 614'-0"	I	X	
(SI)	RSI-H-96	Inside Containment NE Quadrant El. 614'-5"	I	X	
(SI)	RSI-H-97	Inside Containment NE Quadrant El. 630'-2"	I		
(SI)	RSI-H-98	Inside Containment NE Quadrant El. 607'-0"	I		

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 8 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
(SI)	RSI-H-99	Inside Containment NE Quadrant El. 599'-6"	I		
(SI)	RSI-H-100	Inside Containment NW Quadrant El. 599'-6"	I		
(SI)	RSI-H-101	Inside Containment NE Quadrant El. 599'-6"	I		
(SI)	RSI-H-102	Inside Containment NE Quadrant El. 607'-0"	I		
Aux. Coolant (AC)	RAC-H-21	39'-9 3/4" N of col. (E) 8'-8 1/16" E of col. (N) El. 600'-8 1/4"	I		
(AC)	RAC-H-37	9'-4 5/16" S of col. (W) 34'-1 9/16" W of col. (S) El. 599'-8 1/4"	I		
(AC)	RAC-H-38	9'4 5/16" S of col. (W) 34'-1 9/16" W of col. (S) El. 598'-10"	I		
(AC)	RAC-H-39	9'-4 5/16" S of col. (W) 34'-1 9/16" W of col. (S) El. 599'-8 1/4"	I		

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 9 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
(AC)	RAC-H-75	36'-0" N of col. (W) 1'-9 5/8" E of col. (N) El. 601'-8 1/4"	I		
(AC)	RAC-H-76 (NE) RAC-H-76 (SE)	6'-6" N of col. (W) 24'-5 5/8" W of col. (N) El. 601'-8 1/4"	I I		
(AC)	AC-H-68	15'-1" E of col. (K) 6'-6" S of col. (9) El. 615'-6 3/4"	A		
(AC)	AC-H-78	4'-0" N of col. (6) 15'-0" W of col. (J) El. 615'-1"	A		
Service Water (SW)	SW-H-401	13'-3" E of col. (K) 13'-0" S of col. (9) El. 601'-0"	A		
Containment Spray (ICS)	ICS-H-7	13'-8 5/16" E of col. (N) 47'-10" N of center line of containment vessel El. 626'-8"	I		
(ICS)	ICS-H-8	13'-8 5/16" E of col. (N) 47'-10" N of center line of containment vessel El. 627'-10"	I		

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 10 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
(ICS)	ICS-H-9	8'-7 1/8" E of col. (N) 52'-2" N of col. (E) El. 649'-6"	I		
(ICS)	ICS-H-10	49'-6" R from center line of containment vessel 8'-7 1/8" N of col. (E) El. 626'-8"	I		
(ICS)	ICS-H-11	49'-6" R from center line of containment vessel 8'-7 1/8" N of col. (E) El. 627'-0"	I		
(ICS)	ICS-H-12	52'-1 7/8" R from center line of containment vessel 9'-0 5/8" N from center line of containment vessel El. 649'-6"	I		
Chemical volume Control (CVC)	RCVC-H-31A RCVC-H-31B	Inside Containment NW Quadrant El. 600'-1"	I I		X X
(CVC)	RCVC-H-32	Inside Containment NE Quadrant El. 607'-4"	I		
(CVC)	RCVC-H-33A RCVC-H-33B	Inside containment NW Quadrant El. 600'-0"	I I		X X

Table TS 3.14-1
Safety Related Hydraulic Shock Suppressors
Page 11 of 11

System Name	Snubber I.D. Number	Approximate Location & Elevation	Accessible or Inaccessible (A or I)	Difficult to Remove (X)	High Radiation Area at Shutdown (X)
(CVC)	RCVC-H-34	Inside Containment NW Quadrant El. 600'-0"	I		X
(CVC)	RCVC-H-35	Inside Containment NW Quadrant El. 626'-0"	I		X
(CVC)	RCVC-H-36	Inside Containment Pen. 13 N El. 612'-0"	I		
(CVC)	CVC-H-84	4'-9" N of col. (6) 5'-0" E of col. (J) El. 606'-7 7/8"	A		
(CVC)	CVC-H-96	1'-10" N of col. (6) 7'-0" E of col. (HW) El. 597'-0"	A		
Containment Spray (ICS)	CS-H-33A	11'-0" N of col. (4) 7'-0" W of col. (H) El. 607'-0"	A		

TABLE TS 3.15-1

FIRE DETECTION INSTRUMENTATION

Fire Area		# of Detectors	Minimum # Required	Operators Actions
AX-21	4160 Switchgear Room	3	2	Establish an hourly fire watch inspection
AX-23	Special Vent Rooms	9	9	If special ventilation is operating with charcoal filters in service establish an hourly fire watch inspection. If not in service establish a 4-hour inspection frequency.
AX-24	Fuel Handling Area	3	3	Establish an hourly fire watch inspection
AX-30	Relay Room	19	6	Establish an hourly fire watch inspection
AX-32	Cable run area	11	8	Establish an hourly fire watch inspection
AX-35	Control Room	13	0	Control room is continuously manned
AX-37	CRD Room	7	4	Establish an hourly fire watch inspection
SB-65	Shield Building	6	2	Establish a four hour fire watch inspection
SC-70	Screen House	3	2	Establish an hourly fire watch inspection
TU-90/91	D/G 1A and day tank room	7	5	Establish an hourly fire watch inspection
TU-92/93	D/G 1B and day tank room	7	5	Establish an hourly fire watch inspection
TU 95	Air Compressor & Pump Room	4	4	Establish an hourly fire watch inspection
TU 97	Battery Room 1A	1	1	Establish an hourly fire watch inspection
TU 98	Battery Room 1B	1	1	Establish an hourly fire watch inspection

* No two adjacent detectors can be out of service

All hydraulic shock suppressors whose seal materials are other than ethylene propylene or other material that has been demonstrated to be compatible with the operating environment shall be visually inspected for operability every 31 days.

Shock suppressors are categorized as "accessible" or "inaccessible" as noted on Table TS 3.14-1. For the purpose of this inspection these two groups may be considered independently and scheduled accordingly.

b. A representative sample of nine shock suppressors or 10% of the total shall be functionally tested for operability including verification of proper piston movement, lockup, and bleed at each refueling. For each shock suppressor or subsequent shock suppressor found inoperable by this testing requirement, an additional 10% or nine hydraulic shock suppressors shall be tested until no more failures are found or all shock suppressors have been tested. Those shock suppressors designated to be difficult to remove or in a high radiation area during shutdown as noted on Table TS 3.14-1 need not be selected for functional testing. The Anchor Holth suppressors used on the steam generators are exempt from functional testing requirements.

c. The initial inspection shall be performed at the cycle 3 refueling. For the purpose of entering the schedule in Specification 4.14.a, it shall be assumed that the facility had been on a 12 month inspection interval.

Basis

All safety related hydraulic shock suppressors are visually inspected for overall integrity and operability. The inspection will include verification of proper orientation, adequate hydraulic fluid level and proper attachment of snubber to piping and structures.

To further increase the assurance of snubber reliability, functional tests should be performed once each refueling cycle. These tests will include stroking of the snubbers to verify proper piston movement and snubbing action. Ten percent or nine snubbers, represents an adequate smaple for such tests. Observed failures on these samples should require testing of additional units. The Anchor Holth suppressors used on the steam generators are exempt from the functional test requirement due to the impracticability of functionally testing 900 Kip suppressors.

c) Verifying that each high pressure pump auto-start setpoint is ≥100 psig.

5. At least once per 3 years by performing a flow test of the system in accordance with the section titled "Flow Test of Public Main at Plant Site" in Chapter 5, Section 11 of the Fire Protection Handbook, 14th Edition, published by the National Fire Protection Association.

c. Spray/Sprinkler Systems

Each of the spray and/or sprinkler systems in Specification 3.15.c shall be demonstrated OPERABLE:

1. At least once per 12 months by cycling each testable valve in the flow path through at least one complete cycle of full travel.

2. At least once per 18 months:

a) By performing a system functional test which includes simulated automatic actuation of the system, and:

1. Verifying that the automatic valves in the flow path actuate to their correct positions, and

2. Cycling each valve in the flow path that is not testable during plant operation through at least one complete cycle of full travel.

b) By visual inspection of the spray headers to verify their integrity, and

c) By visual inspection of each nozzle to verify no blockage.

3. At least once per three years by performing an air flow test through each open head spray/sprinkler header and verifying each open head spray/sprinkler nozzle is unobstructed.

d. Low Pressure CO_2 Systems

Each of the low pressure CO_2 systems in Specification 3.15.d shall be demonstrated OPERABLE:

1. At least once per 7 days by verifying CO_2 storage tank level and pressure, and

2. At least once per 18 months by verifying:

a) The system valves and associated ventilation dampers actuate manually and automatically, upon receipt of a simulated actuation signal, and

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 25 TO FACILITY LICESENE NO. DPR-43

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

KEWAUNEE NUCLEAR POWER PLANT

DOCKET NO. 50-305

Introduction

By letter dated November 10, 1978, Wisconsin Public Service Corporation (the licensee) requested amendment of the Technical Specifications appended to Facility Operating License No. DPR-43 for the Kewaunee Nuclear Power Plant (Kewaunee). The amendment would correct the definition of rated power, update the listing of safety-related shock suppressors, correct several errors in the listing of fire detectors, correct the minimum auto start pressure for fire pumps and delete a definition that was applicable to Cycle 1 only.

Evaluation

Several changes to the Technical Specifications were requested by the licensee in its letter of November 10, 1978. The change in definition of rated power from "rated system output of 1650 MWt" to "reactor core output of 1650 MWt" resolves an inconsistency between the Appendix A Technical Specifications and the definition of rated power as stated in the main body of Facility Operating License No. DPR-43. The inconsistency resulted from an error in definition made in the original Appendix A. The licensee has verified that safety analyses performed in support of the original facility licensing effort are consistent "in method and power level assumptions employed on other facilities which have reactor core output as the limiting parameter". The staff finds this change in definition of rated power acceptable.

Deletion of definition n., Interim Fuel Limits, is acceptable to the staff as the definition applied to Cycle 1 operation only and the facility is now in Cycle 4.

The licensee has requested that we approve an update in the listing of safety-related shock suppressors (snubbers) and an increase in the number of snubbers on which a functional test is to be performed during each operating cycle. The Technical Specifications currently require a functional test of ten percent of safety-related snubbers once each cycle. The licensee has determined that the number of safety-related snubbers is greater than the number currently listed in TS Table 3.14-1. The licensee has revised TS Table 3.14-1 to list all safety-related snubbers; in addition, the licensee has increased the number of snubbers to be tested each cycle to observe the ten percent sampling requirement currently stated in the Technical Specifications. The staff finds the licensee's changes acceptable.

The licensee has also requested a change to the listing of fire detectors in the Technical Specifications. The licensee's revision is to correct errors in the number of fire detectors located in various fire areas of the facility. The staff has reviewed the proposed corrections and finds them acceptable.

Finally, the licensee has requested that we correct a typographical error in TS 4.15.b.4.c, which incorrectly lists the design minimum auto start pressure for the fire water pumps as $\geq$ 120 psig. The correct value is $\geq$ 100 psig. This change is acceptable to the staff.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and(3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: January 19, 1979

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-305

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 25 to Facility Operating License No. DPR-43 issued to Wisconsin Public Service Corporation, Wisconsin Power and Light Company, and Madison Gas and Electric Company (the licensee) which revised Technical Specifications for operation of the Kewaunee Nuclear Power Plant located in Kewaunee, Wisconsin. The amendment is effective as of the date of issuance.

The amendment corrects the definition of rated power, updates the listing of safety-related shock suppressors, corrects several errors in the listing of fire detectors, corrects the minimum auto start pressure for fire pumps, and deletes a definition relevant to Cycle 1 only.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated November 10, 1978, (2) Amendment No. 25 to Facility Operating License No. DPR-43, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C. 20555, and at the Kewaunee Public Library, 314 Milwaukee Street, Kewaunee, Wisconsin 54216. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 19th day of January, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors