

February 7, 1992

Docket No. 50-305

Mr. C. A. Schrock
Manager - Nuclear Engineering
Wisconsin Public Service
Corporation
P. O. Box 19002
Green Bay, Wisconsin 54037-9002

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Dear Mr. Schrock:

SUBJECT: KEWAUNEE NUCLEAR POWER PLANT (TAC NO. M82685)

Enclosed is a "Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for Hearing" concerning your application for amendment dated January 27, 1992, to allow use of Combustion Engineering Nuclear Services sleeves and plugs for tube repair in the Kewaunee Nuclear Power Plant steam generator.

The notice has been forward to the Office of the Federal Register for publication.

Sincerely,

Original signed by Robert B. Samworth for:

Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc: See next page

PDIII-3:LA
PKreutzer
2/17/92

for
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AHansen
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IBH for
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JHannon
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DPD

Mr. C. A. Schrock
Wisconsin Public Service Corporation

Kewaunee Nuclear Power Plant

cc:

David Baker, Esquire
Foley and Lardner
P. O. Box 2193
Orlando, Florida 32082

Glen Kunesh, Chairman
Town of Carlton
Route 1
Kewaunee, Wisconsin 54216

Mr. Harold Reckelberg, Chairman
Kewaunee County Board
Kewaunee County Courthouse
Kewaunee, Wisconsin 54216

Chairman
Public Service Commission of Wisconsin
Hill Farms State Office Building
Madison, Wisconsin 53702

Attorney General
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Madison, Wisconsin 53702

U.S. Nuclear Regulatory Commission
Resident Inspectors Office
Route #1, Box 999
Kewaunee, Wisconsin 54216

Regional Administrator - Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Mr. Robert S. Cullen
Chief Engineer
Wisconsin Public Service Commission
P.O. Box 7854
Madison, Wisconsin 53707

UNITED STATES NUCLEAR REGULATORY COMMISSIONWISCONSIN PUBLIC SERVICE CORPORATIONDOCKET NO. 50-305NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-43 issued to Wisconsin Public Service Corporation (the licensee) for operation of the Kewaunee Nuclear Power Plant located in Kewaunee County, Wisconsin.

The proposed amendment would revise the Technical Specifications (TS) in Section 4.2, "ASME Code Class Inservice Inspection and Testing," and in Figure TS 4.2-1, "Application of Plugging Limits," and would revise Section 2.C(5), "Steam Generator Tube Repairs," of the Operating License. The proposed amendment would allow use of Combustion Engineering Nuclear Services sleeves and plugs for tube repair in the Kewaunee Nuclear Power Plant steam generators. Administrative changes are also proposed dealing with format and typographical inconsistencies.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in

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accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

- (1) The proposed change will not involve a significant increase in the probability or consequences of an accident previously evaluated. The intent of this proposed amendment is to allow WPSC to use CENS welded sleeves to repair the KNPP SG tubes showing degradation in sleeveable regions of the tube sheet and the tube support plate crevice areas. To support this, changes are being proposed to TS 4.2.b.4.a to reference both WCAP 11643 and CEN-413-P as acceptable methods to repair the tubes, revise TS 4.2.b.4.b to state it is only applicable to Westinghouse mechanical sleeves, and to add a plugging limit for the CENS welded leak tight sleeves. In addition, the existing license condition 2.C(5) is being deleted.

Deleting license condition 2.C.(5) is an administrative change. The information is being relocated to TS 4.2.b.4.a. This will consolidate the requirements for SG tube repair into one TS while still maintaining the original intent of the license condition; that is, repair methods shall be submitted to the NRC for prior review and approval. Therefore, this proposed change will not increase the probability or consequences of an accident previously evaluated.

Report CEN-413-P, "Kewaunee Steam Generator Tube Repair Using Leak Tight Sleeves", demonstrates that repair of degraded tubes using the CENS sleeves will result in tube bundle integrity consistent with the original design basis.

The sleeve design, materials, and joints were designed to the applicable ASME Boiler and Pressure Vessel Codes. An extensive analysis and test program was undertaken to prove the adequacy of the CENS welded sleeve. This program determined the effect of normal

operating postulated accident conditions on the sleeve-tube assembly, as well as the adequacy of the assembly to perform its intended function.

The postulated LOCA and non-LOCA transients were assessed to determine the impact of repair by sleeving. The installation of sleeves into a SG results in additional flow restriction within the primary system and associated increase in the pressure drop across the SG. The effects of this flow restriction for installing tubesheet and/or tube support plate sleeves were evaluated. The current safety analysis supports plant operation with up to 13.94% of the SG tubes plugged. Any combination of sleeving and plugging up to this limit are bounded by the existing analysis. WPSC maintains an administratively controlled program to assure that this allowable SG tube plugging level is not exceeded.

Furthermore, in accordance with Regulatory Guide 1.83 recommendations, the sleeves can be regularly monitored by ET inspection techniques. A sleeve plugging limit is being proposed based on a Regulatory Guide 1.121 analysis.

Therefore, based on extensive analysis and test programs performed, the provisions not to exceed the current analyzed tube plugging limit, and the ability to monitor and remove degraded sleeves from service, it can be concluded that this proposed change will not significantly increase the probability or consequences of an accident previously evaluated.

- (2) The proposed change will not create the possibility of a new or different kind of accident from any accident previously evaluated. Using CENS welded sleeves for tube repair in the KNPP SGs does not alter the design basis of the plant. The structural adequacy of the repair sleeve-tube assembly has been demonstrated to meet regulatory requirements, therefore, the overall functional capability of the SG tube bundle remains unchanged. Use of CENS sleeves will not create the possibility of a new or different kind of accident from any accident previously evaluated.
- (3) The proposed change will not involve a significant reduction in the margin of safety. Sleeve repair of degraded SG tubes has been demonstrated to maintain the structural ability of the SG tubing during normal and postulated accident conditions, and to prevent

The effect of sleeving on the transients and accidents evaluated in the KNPP USAR has been reviewed. The existing safety analysis supports plant operation with up to 13.94% of the SG tubes plugged (USAR section 14.3.10). An administratively controlled program exists to assure that the allowable SG tube plugging level is not exceeded. In addition, the sleeve assemblies can be monitored through periodic inspection and a sleeve plugging limit has been established based on RG 1.121 criteria.

Use of CENS leak tight sleeves for the upcoming refueling outage provides several advantages. The current sleeving method permitted by the KNPP TSs only allows access to 84% of the SG tube bundle. The curved peripheral sleeves will extend the tubesheet sleeving boundary to all but the outer most tubes. Approval to use the tube support plate sleeves will allow the option to repair pluggable indications up to the sixth support plate in the cold leg and the unsleeved hot leg tubes prior to the approval of the alternate plugging criteria for the tube support plates.

Repair of SG tubes by sleeving will decrease the number of tubes that would be taken out of service by plugging. Installation of tube plugs reduces the coolant flow rate available for core cooling, thus sleeving will maintain the margin of flow that would otherwise be reduced by plugging. Based on the above, it is concluded that the proposed change does not result in a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within thirty (30) days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Written comments may be submitted by mail to the Regulatory Publications Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room P-223, Phillips Building, 7920 Norfolk Avenue, Bethesda, Maryland, from 7:30 a.m. to 4:15 p.m. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555. The filing of requests for hearing and petitions for leave to intervene is discussed below.

By March 16, 1992, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the local public document room located at the Government Documents Section, Library Learning Center, University of Wisconsin, 2420 Nicolet Drive, Green Bay, Wisconsin. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board Panel, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the

Secretary or the designated Atomic Safety and Licening Board Panel will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation

of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555, by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 325-6000 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to John N. Hannon:

petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to David Baker,

Esq., Foley and Lardner, P. O. Box 2103, Orlando, Florida 31082, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board Panel that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated January 27, 1992, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the local public document room located at the Government Documents Section, Library Learning Center, University of Wisconsin, 2420 Nicolet Drive, Green Bay, Wisconsin.

Dated at Rockville, Maryland, this 7th day of February 1992.

FOR THE NUCLEAR REGULATORY COMMISSION



Jon B. Hopkins, Acting Director
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation