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J. M. Taylor

L. J. Harmon

T. Barnhart (4)

L. Schneider

D. Brinkman

ACRS (10)

OPA (Clare Miles)

R. Diggs

NSIC

ASLAB

January 3, 1983

Docket No. 50-305

Mr. C. W. Giesler, Vice President  
Nuclear Power  
Wisconsin Public Service Corporation  
Post Office Box 1200  
Green Bay, Wisconsin 54305

Dear Mr. Giesler:

The Commission has issued the enclosed Amendment No. 48 to Facility Operating License No. DPR-43 for Kewaunee Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated September 27, 1982.

The amendment revises the Technical Specifications to reflect recent reorganization of the Nuclear Department.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Marshall Grotenhuis, Project Manager  
Operating Reactors Branch No. 1  
Division of Licensing

## Enclosures:

1. Amendment No. 48 to DPR-43
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:

See next page

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OFFICE	ORB 1	ORB 1	ORB 1	AB:OR	OELD		
SURNAME	CParrish	MGrotenhuis	rsbVarga	GLainas	B. B. Borden		
DATE	12/11/82	12/11/82	12/11/82	12/11/82	12/12/82		

Mr. C. W. Giesler  
Wisconsin Public Service Corporation

cc: Steven E. Keane, Esquire  
Foley and Lardner  
777 East Wisconsin Avenue  
Milwaukee, Wisconsin 53202

Stanley LaCrosse, Chairman  
Town of Carlton  
Route 1  
Kewaunee, Wisconsin 54216

Mr. Donald L. Quistroff, Chairman  
Kewaunee County Board  
Kewaunee County Courthouse  
Kewaunee, Wisconsin 54216

Chairman  
Public Service Commission of Wisconsin  
Hill Farms State Office Building  
Madison, Wisconsin 53702

Mr. Patrick Walsh  
Assistant Attorney General  
114 East, State Capitol  
Madison, Wisconsin 53702

U. S. Nuclear Regulatory Commission  
Resident Inspectors Office  
Route #1, Box 999  
Kewaunee, Wisconsin 54216

Regional Radiation Representative  
EPA Region V  
230 South Dearborn Street  
Chicago, Illinois 60604

James G. Keppler  
Regional Administrator - Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN PUBLIC SERVICE CORPORATION  
WISCONSIN POWER AND LIGHT COMPANY  
MADISON GAS AND ELECTRIC COMPANY

DOCKET NO. 50-305

KEWAUNEE NUCLEAR PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 48  
License No. DPR-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Public Service Corporation Wisconsin Power and Light Company and Madison Gas and Electric Company (the licensees) dated September 27, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

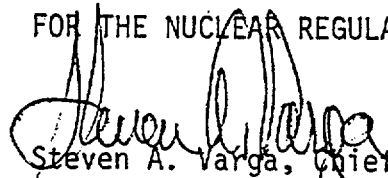
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-43 is hereby amended as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 48, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: January 3, 1983

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 48 TO FACILITY OPERATING LICENSE NO. DPR-43

DOCKET NO. 50-305

Revise Appendix A as follows:

Remove Pages

TS 6-2a  
TS 6-5  
TS 6-6

TS 6-7  
TS 6-8  
TS 6-11

TS 6-12  
TS 6-17

TS 6-26 (Fig. TS 6.2-1)  
TS 6-27 (Fig. TS 6.2-2)  
TS 6-28

-  
-

Insert Pages

TS 6-2a  
TS 6-5  
TS 6-6

TS 6-7  
TS 6-8  
TS 6-11

TS 6-12  
TS 6-17

TS 6-26  
TS 6-27  
TS 6-28  
Figure TS 6.2-1  
Figure TS 6.2-2

#### 6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the Plant Staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI-N18.1-1971 and Appendix A of 10 CFR Part 55.
- 6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Fire Marshall and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except that training sessions shall be held quarterly.

#### 6.5 REVIEW AND AUDIT

##### 6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

##### FUNCTION

- 6.5.1.1 The PORC shall function to advise the Plant.

changes to the Vice-President - Nuclear Power.

#### AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend to the Plant Manager approval or disapproval of items considered under 6.5.1.6a through d above.
- b. Make determinations with regard to whether or not each item considered under 6.5.1.6a through e above constitutes an unreviewed safety question.
- c. Provide immediate notification in the form of draft meeting minutes to the Manager-Nuclear Power and the Chairman-Nuclear Safety Review and Audit Committee of disagreement between the PORC and the Plant Manager. The Plant Manager shall have responsibility for resolution of such disagreements.

#### RECORDS

6.5.1.8 Minutes shall be kept of all meetings of the PORC and copies shall be sent to the Manager -Nuclear Power and the Chairman-Nuclear Safety Review and Audit Committee.

#### 6.5.2 CORPORATE NUCLEAR ENGINEERING STAFF (CNES)

##### FUNCTION

6.5.2.1 The CNES shall function to provide engineering,

technical and quality assurance activities in support of the Kewaunee Plant Staff.

#### ORGANIZATION

6.5.2.2 The CNES consists of the following groups:

- a. Nuclear Licensing and Systems
- b. Nuclear Services
- c. Nuclear Training
- d. Nuclear Design Change
- e. System Planning and Engineering
- f. Power Plant Design and Construction
- g. Fuel and Fossil Operations
- h. Administrative Staff

#### ACTIVITIES

- 6.5.2.3
1. Review and report all violations of the Technical Specifications, codes, regulations, and statutes.
  2. Review all activities associated with nuclear safety for technical adequacy and compliance with internal procedures or instructions.
  3. Review and report significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
  4. Review and report all events which are required by regulations or Technical Specifications to be reported to the NRC (Plant personnel will provide the initial reporting to the NRC of those events requiring 24 hour notification.)
  5. Investigate any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems or components.



6. Review and/or prepare safety evaluations of all plant design changes.
7. Audits as required by the Quality Assurance Program and as outlined in Section 6.5.3.8.

#### AUTHORITY

- 6.5.2.4 Members of the Fuel and Fossil Operations, Power Plant Design and Construction, and System Planning and Engineering groups, although not directly responsible to the Vice President - Nuclear Power, are available for special projects and support of the Kewaunee Plant.

The Nuclear Design Change, Nuclear Licensing and Systems, Nuclear Services, and Nuclear Training groups are responsible to the Manager Nuclear Power.

### 6.5.3 NUCLEAR SAFETY REVIEW AND AUDIT COMMITTEE (NSRAC)

#### FUNCTION

- 6.5.3.1 The NSRAC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear Power Plant Operations
- b. Nuclear Engineering
- c. Chemistry and Radio-Chemistry
- d. Metallurgy
- e. Instrumentation
- f. Radiological Safety
- g. Mechanical and Electrical Engineering
- h. Quality Assurance Practices
- i. Other appropriate fields as determined by the Committee, to be associated with the unique characteristics of the nuclear power plant.

#### COMPOSITION

- 6.5.3.2 The NSRAC shall be composed of, but not necessarily limited to:

- a. At least three technically qualified persons who are not members of the plant staff.
- b. One member from the supervisory staff of the plant.
- c. At least two qualified non-company affiliated technical consultants.
- d. Plus in-house staff management advisors as required.

The Committee membership and its Chairman and Vice Chairman shall be appointed by the Vice-President - Nuclear Power or such person as he shall designate. Each member of the NSRAC shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of five years technical experience, of which a minimum shall be in one or more areas given in 6.5.3.1.

#### ALTERNATES

- 6.5.3.3 Alternate members shall be appointed by the NSRAC Chairman, upon approval by the Vice-President - Nuclear Power, to serve on a temporary basis; however, no more than two alternates shall participate in NSRAC activities at any one time.

#### CONSULTANTS

- 6.5.3.4 Consultants may be utilized as determined by the Chairman - NSRAC to provide expert advice to the NSRAC.

#### MEETING FREQUENCY

- 6.5.3.5 The NSRAC shall meet at least once every six months.

- g. Any other area of plant operation considered appropriate by the NSRAC or the Vice-President - Nuclear Power.

#### AUTHORITY

- 6.5.3.9 The NSRAC shall report to and advise the Vice-President - Nuclear Power on those areas of responsibility specified in Section 6.5.3.7 and 6.5.3.8.

#### RECORDS

- 6.5.3.10 Records of NSRAC activities shall be prepared, approved and distributed as follows:
  - a. Minutes of each NSRAC meeting forwarded to the Vice-President - Nuclear Power within 14 days following each meeting.
  - b. Reports of reviews required by Section 6.5.3.7e, f, g and h above, forwarded to the Vice-President - Nuclear Power within 14 days following completion of the review.
  - c. Reports of audits performed by NSRAC shall be forwarded to the Vice-President - Nuclear Power and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 DELETED

## 6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a safety limit is violated:
- a. The reactor shall be shutdown and operation shall not be resumed until authorized by the Commission.
  - b. The Safety Limit violation shall be reported to the Commission, the Manager-Nuclear Power, and to the NSRAC-Chairman within 14 days of the violation.
  - c. The Report shall be prepared in accordance with Section 6.9 of the Technical Specifications.

## 6.8 PROCEDURES

- 6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet the requirements and recommendations of Section 5.1 and 5.3 of ANSI N18.7-1972.
- 6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the Plant Manager prior to implementation and periodically as determined by the Plant Manager.
- 6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:
- a. The intent of the original procedure is not altered.
  - b. The change is approved by two members of the Plant Management Staff, at least one of which holds a Senior Reactor Operator's License, if the procedure affects nuclear safety.
  - c. The change is documented, reviewed by the PORC, and approved by the Plant Manager.

steady state conditions greater than or equal to one percent; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if subcritical, an unplanned reactivity insertion of more than 50¢; or occurrence of any unplanned criticality.

- (5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.
- (6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For items 6.9.2.a(5) and 6.9.2.a(6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 6.9.2.b(2) and 6.9.2.b(3) below.

- (7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
- (8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications

6.14 ENVIRONMENTAL QUALIFICATION

6.14.1 By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification Class IE Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," December 1979. Copies of these documents are attached to Order for Modification of License No. DPR-43 dated October 24, 1980.

6.14.2 By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

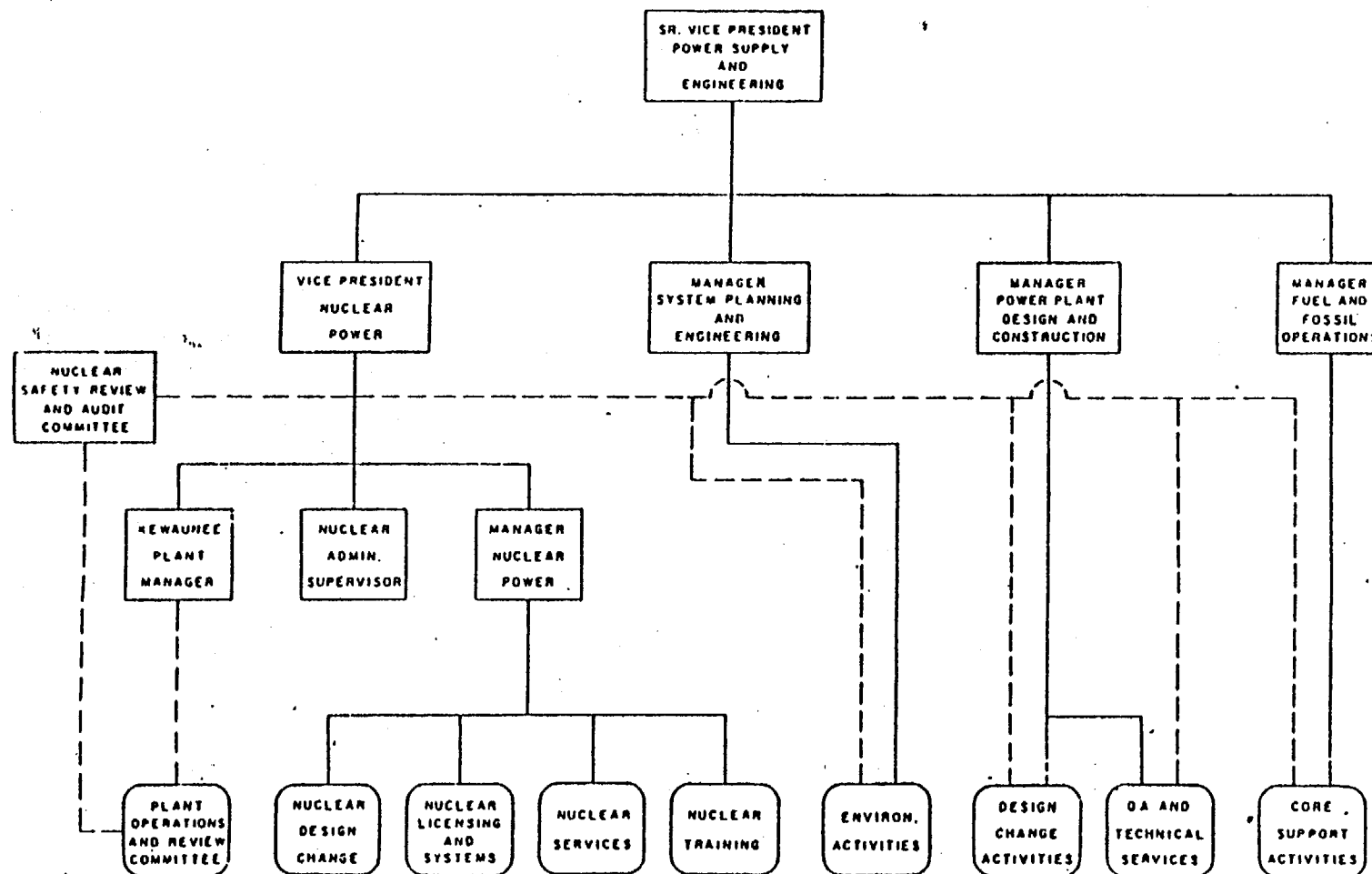
NOTE: 10CFR50.49 suspends the June 30, 1982, requirement.

PAGE TS 6-27 DELETED

Page TS 6-26 replaces page TS 6-28

Page TS 6-28 deleted.

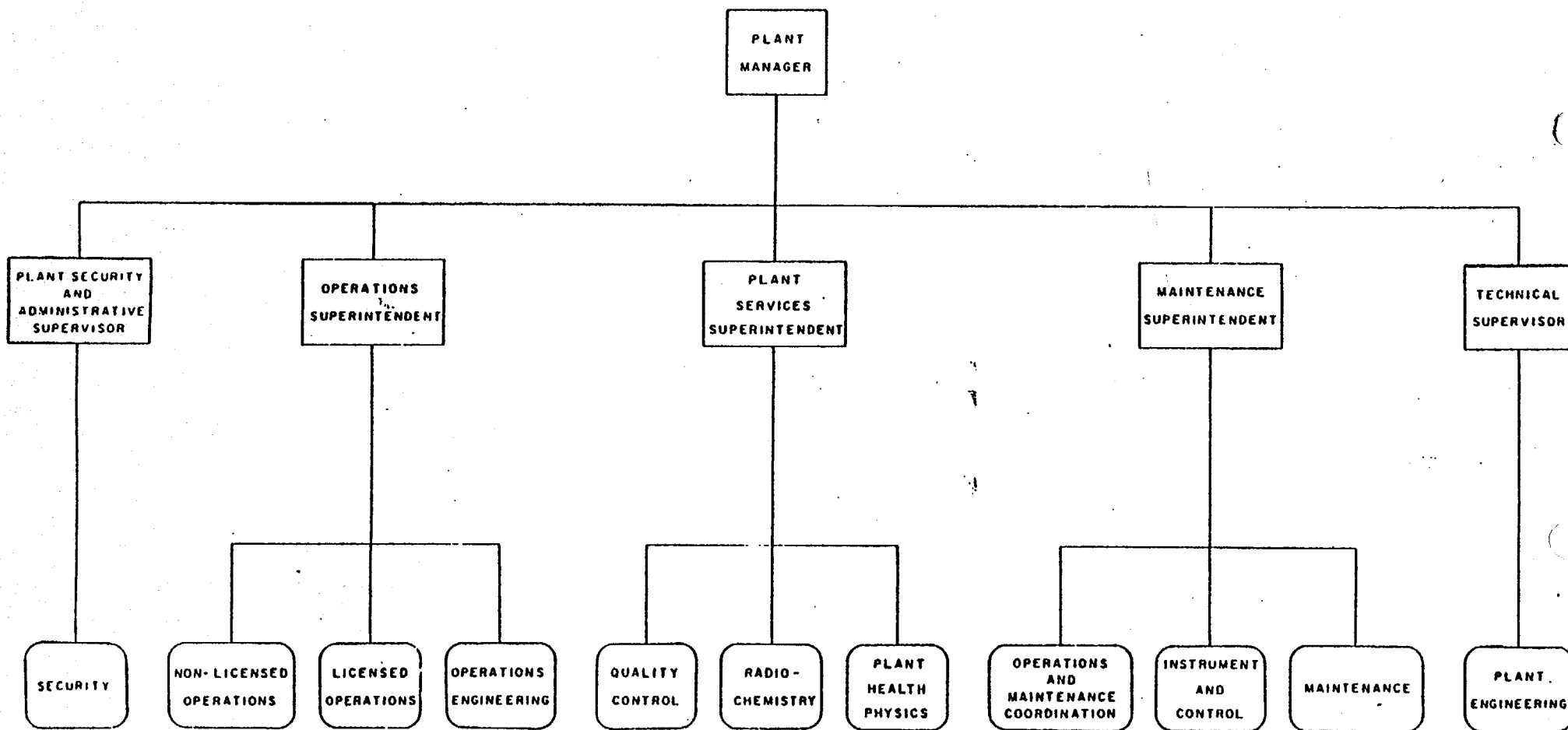




# LEGEND

- Direct Responsibility
- - - - Lines of Communication

Figure TS 6.2-1  
Effective Date 06/01/82  
Amendment No. 48



Amendment No. 48

Figure TS 6.2-2  
Effective Date 06/01/82



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 48 TO FACILITY OPERATING LICENSE NO. DPR-43

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

KEWAUNEE NUCLEAR POWER PLANT

DOCKET NO. 50-305

INTRODUCTION

On September 27, 1982, the Wisconsin Public Service Corporation (the licensee) submitted proposed Amendment No. 50 to facility Operating License No. DPR-43 for the Kewaunee Nuclear Power Plant (the facility). The proposed amendments would change the Technical Specifications for the facility to reflect recent reorganization of the Nuclear Department.

In addition to title changes, the Technical Specifications are revised to (1) reflect the organization and authority of the Corporate Nuclear Engineering Staff (CNES), (2) clarify the meeting frequency and line of authority for the Nuclear Safety Review and Audit Committee (NSRAC), (3) show functional responsibilities instead of titles in the Power Supply and Engineering Department and at the Kewaunee Nuclear Power Plant, and (4) correct two minor typographical errors.

EVALUATION

A. Change of Titles (Pages TS 6-5, 6-7, 6-8, 6-11 and 6-12)

The changes involve title changes (Assistant Manager - Nuclear Power to Manager - Nuclear Power, Plant Superintendent to Plant Manager, and Manager - Nuclear Power to Vice President - Nuclear Power). The duties and qualifications of the positions remain unchanged. The changes are therefore acceptable.

B. Organization and Authority of the Corporate Nuclear Engineering Staff (CNES) (Pages TS 6-6 and 6-7)

Section 6.5.2.2 - The CNES now consists of the following groups:

- a. Nuclear Licensing and Systems
- b. Nuclear Services

- c. Nuclear Training
- d. Nuclear Design Change
- e. System Planning and Engineering
- f. Power Plant Design and Construction
- g. Fuel and Fossil Operations

Section 6.5.2.4 - The Nuclear Design Change, Nuclear Licensing and Systems, Nuclear Services, and Nuclear Training groups are responsible to the Manager - Nuclear Power.

The above changes reflect the present functional groups in the Power Supply and Engineering Department and the current organization of the Nuclear Department. We believe that the changes should improve the organization and authority of the corporate support staff. The changes are therefore acceptable.

C. Meeting Frequency and Authority for the Nuclear Safety Review and Audit Committee (NSRAC) (Pages TS 6-8 and 6-11)

Section 6.5.3.5 - The present TS states: "The NSRAC shall meet at least once per calendar quarter during the initial year of plant operation following fuel loading and at least once per six months thereafter." The proposed TS states: "The NSRAC shall meet at least once every six months."

Section 6.5.3.9 - The present TS states: "The NSRAC shall report to and advise the Vice President - Power Supply and Engineering on those areas of responsibility specified in Section 6.5.3.7 and 6.5.3.8." The proposed TS change the reporting from the Vice President - Power Supply and Engineering to the Vice President - Nuclear Power.

The proposed rewording of the NSRAC meeting frequency reflect the current operation of the plant and the change in authority reflect the line of authority from the Vice President - Nuclear Power to the NSRAC. We find the changes acceptable.

D. Figures TS 6.2-1 and TS 6.2-2

The figures were changed to show functional responsibilities instead of job titles.

Figure TS 6.2-1 shows that the Vice President - Nuclear Power reports to the Senior Vice President - Power Supply and Engineering. Reporting to the President - Nuclear Power are the Kewaunee Plant Manager, the Nuclear Admin. Supervisor and the Manager - Nuclear Power.

Figure TS 6.2-2 shows that the Plant Manager has the Technical Supervisor, Maintenance Superintendent, Plant Services Superintendent, Operations Superintendent, and the Plant Security and Administrative Supervisor reporting to him. The health physics organization remains under the Plant Services Superintendent who reports directly to the Plant Manager. Thus, the Health Physics Supervisor is independent from plant operations. This is in accordance with the recommendations of Regulatory Guide 8.8, "Information Relevant to Ensuring That Occupational Exposure At Nuclear Power Stations Will Be As Low As Is Reasonably Achievable" and is acceptable. The changes to Figure 6.2-2 do not affect the specifications for staff qualifications. As per Section 6.3.1 of the Technical Specifications, each member of the plant staff shall meet or exceed the minimum acceptable levels of ANSI-N18.1 - 1971. The qualifications for the Health Physics Supervisor will be in accordance with the recommendations of Regulatory Guide 1.8, Rev. 1-R, Sept. 1975. Proposed Amendment No. 50 to the Kewaunee Technical Specifications is, therefore, acceptable to the staff.

We have reviewed the above organizational changes and found that they should strengthen the management structure and are therefore acceptable.

#### ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: January 3, 1983

#### Principal Contributors

Erik Pederson  
Roger L. Pederson

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-305WISCONSIN PUBLIC SERVICE CORPORATIONWISCONSIN POWER AND LIGHT COMPANYMADISON GAS AND ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 48 to Facility Operating License No. DPR-43, issued to Wisconsin Public Service Corporation, Wisconsin Power and Light Company, and Madison Gas and Electric Company (the licensees), which revised Technical Specifications for operation of the Kewaunee Nuclear Plant (the facility) located in Kewaunee, Wisconsin. The amendment is effective as of the date of issuance.

The amendment revises the Technical Specifications to reflect recent reorganization of the Nuclear Department.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

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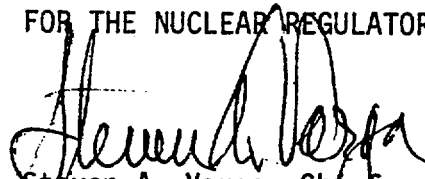
- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.55(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 27, 1982, (2) Amendment No. 48 to License No. DPR-43 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Kewaunee Public Library, 822 Juneau Street, Kewaunee, Wisconsin 54216. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 3rd day of January 1983.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing