

March 1, 1977

Docket No. 50-305

Wisconsin Public Service Corporation
ATTN: Mr. E. W. James
Senior Vice President
Post Office Box 1200
Green Bay, Wisconsin 54305

Gentlemen:

The Commission has issued Amendment No. 14 to Facility License No. DPR-43 for the Kewaunee Nuclear Power Plant. The amendment consists of changes to the Technical Specifications and is in response to your request dated October 19, 1976.

The amendment revises the Technical Specifications to incorporate Limiting Conditions for Operations and Surveillance Requirements for shock suppressors (snubbers).

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

/s/

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

1. Amendment No. 14 to DPR-43
2. Safety Evaluation
3. Federal Register Notice

cc w/encl:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

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A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

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cc w/encl:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

DOCKET NO. 50-305

KEWAUNEE NUCLEAR POWER PLANT


AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 14
License No. DPR-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Public Service Corporation, Wisconsin Power and Light Company and Madison Gas and Electric Company (the licensees) dated October 19, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read 'A. Schwencer', is written over the typed name.

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 1, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 14
FACILITY OPERATING LICENSE NO. DPR-43
DOCKET NO. 50-305

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Insert Pages

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TS iv

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3.14 SHOCK SUPPRESSORS (SNUBBERS)

Applicability

Applies to the operability of shock suppressors which are related to plant safety.

Objective

To ensure that shock suppressors, which are used to restrain safety related piping under dynamic load conditions, are functional during reactor operation.

Specification

- a. The reactor shall not be made critical unless all safety related shock suppressors listed on Table TS 3.14-1 are operable except as noted in 3.14.b.
- b. During power operation or recovery from inadvertent trip, if any shock suppressor listed on Table TS 3.14-1 is found inoperable one of the following actions shall be taken within 72 hours.
 1. The inoperable shock suppressor shall be restored to an operable condition or replaced with a spare shock suppressor of similar specifications; or
 2. The fluid line restrained by the inoperable shock suppressor shall, if feasible, be isolated from other safety related systems if otherwise permitted by the Technical Specifications and thereafter operation may continue subject to any limitations by the Technical Specifications for that fluid line; or
 3. Actions shall be initiated to shut down the reactor and the reactor shall be in a hot shutdown condition within 36 hours.

Basis

Shock suppressors (snubbers) are designed to prevent unrestrained pipe motion under dynamic loads, as might occur during seismic activity or severe plant transients, while allowing normal thermal motion during startup or shutdown. The consequence of an inoperable snubber is an increase in the probability of structural damage to piping as a result of a seismic event or other events initiating dynamic loads. It is therefore required that all snubbers designed to protect the reactor coolant and other safety related systems or components be operable during reactor operation. The intent of this specification is to prohibit startup or continued operation with defective safety related shock suppressors.

Because the protection afforded by snubbers is required only during low probability events, Specification 3.14.b allows a period of 72 hours for repairs or feasible alternative action before reactor shutdown is required.

Table TS 3.14-1
Safety Related Shock Suppressors (Snubbers)
Page 1 of 6

<u>System Name</u>	<u>Snubber I.D. Number</u>	<u>Approximate Location & Elevation</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>Difficult to Remove (X)</u>	<u>* High Radiation Area at Shutdown (X)</u>
Reactor Coolant	Anchor Holth 900K custom suppressors 1 through 8	Lateral restraint for steam generators (4 on each)	I	X	X
Steam Supply to Aux. FW Pump 1C (MS)	SS-H67	1'-10 3/4" S of col. 9 6'-1 3/4" E of col. E EL. 593'-0"	A		
(MS)	SS-H76	1'-6" S of col. 4 2'-6" E of col. HE EL. 603'-1"	A		
(MS)	SS-H-86	0'-0" N of col. 8 8'-7" E of col. G EL. 588'-1"	A		
(MS)	SS-H87	0'-0" N of col. 8 9'-10" E of col. G EL. 588'-1"	A		
(MS)	SS-H88	0'-0" N of col. 8 9'-10" E of col. G EL. 588'-6"	A		
(MS)	SS-H129	0'-0" N of col. 6 2'-6" E of col. GW EL. 638'-2 3/4"	A	X	
(MS)	SS-H150	3" N of col. 65 1'-11" W of col. LW EL. 635'-0"	A		
(MS)	SS-H103	3'-0" N of col. 5 2'-4" E of col. G EL 593'-3"	A		

Table TS 3.14-1
Safety Related Shock Suppressors (Snubbers)
Page 2 of 6

<u>System Name</u>	<u>Snubber I.D. Number</u>	<u>Approximate Location & Elevation</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>Difficult to Remove (X)</u>	<u>*High Radiation Area at Shutdown (X)</u>
(MS)	SS-H146	10'-8" W of col. K 2'-5 1/2" S of col. 6 EL 634'-9 1/2"	A	X	
Residual Heat Removal (RHR)	RHR-H10H	20'-9" N of col. 6 21'-0" E of col. K EL. 601'-0"	I		X
(RHR)	RHR-H12B	9'-7 1/2" N of col. 6 22'-2" E of col. 95 EL. 598'-6"	I		X
(RHR)	RHR-H14	36'-1" N of col. W 18'-6 5/8" E of col. N EL. 607'-6"	I		
(RHR)	RHR-H15	36'-0 3/8" N of col. W 18'-6 5/8" E of col. N EL. 607'-6"	I		
(RHR)	RHR-H16A	5'-2 1/2" N of col. 6 12'-8 1/2" E of col. K EL. 617'-9"	I	X	X
(RHR)	RHR-H18	27'-2 7/16" N of col. E 23'-4 7/16" E of col. N EL. 611'-0"	I		X
(RHR)	RHR-H49	20'-0" N of col. 6 22'-1" E of col. 95 EL. 601'-6"	I		X
Safety Injection (SI)	SI-H35	2'-0" N of col. 6 15'-11 3/8" E of col. K EL. 606'-9"	I	X	X
(SI)	SI-H60	3'-0" S of col. 6 1'-6" W of col. HW EL 629'-11 3/4"	I		X

Table TS 3.14-1
Safety Related Shock Suppressors (Snubbers)
Page 3 of 6

<u>System Name</u>	<u>Snubber I.D. Number</u>	<u>Approximate Location & Elevation</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>Difficult to Remove (X)</u>	<u>* High Radiation Area at Shutdown (X)</u>
(SI)	2180	Inside Containment EL. 620'-5" NE Quadrant	I	X	
(SI)	2243	Inside Containment EL. 614'-0" NE Quadrant	I	X	
(SI)	2295	Inside Containment NE Quadrant EL. 620'-5"	I	X	
(SI)	2513	Inside Containment NE Quadrant EL. 630'-2"	I		
(SI)	RSI-H2A	46'-11 1/2" N of col. W. 16'-15 3/8" E of col. N EL. 607'-0"	I		
(SI)	RSI-15A	1'-6" N of col. W 22'-2" W of col. N EL. 602'-2"	I		
(SI)	RSI-H20B	25'-7 1/2" S of col. W 35'-3" W of col. N EL. 600'	I		
(SI)	RSI-H38	31'-2 3/4" N of col. E 3'-5 1/2" E of col. N EL. 607'-5"	I		
(SI)	RSI-H78	34'-7 1/4" N of col. W 15'-9 1/2" E of col. N EL. 601'	I		
(SI)	RSI-H83	17'-5" N of col. W 0'-5 3/4" E of col. N EL. 601'-0"	I		

Amendment No. 1A

Table TS 3.14-1
Safety Related Shock Suppressors (Snubbers)
Page 4 of 6

<u>System Name</u>	<u>Snubber I.D. Number</u>	<u>Approximate Location & Elevation</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>Difficult to Remove (X)</u>	<u>* High Radiation Area at Shutdown (X)</u>
Reactor Coolant RTD Line (RC)	RTD-H2	7'-10" E from C of stm. gen. 1A 6'-10" S from C of stm. gen. 1A	I		X
(RC)	RTD-H6	15'-3 1/2" E from C of stm. gen. 1A 11'-9" S from C of stm. gen. 1A EL 615'-3 3/16"	I		X
(RC)	RTD-H11	6'-2" N from C of stm. gen. 1B 6'-3" W from C of stm. gen. 1B EL. 616'-10 1/4"	I		X
Internal Contain- ment Spray (ICS)	ICS-H7	13'-8 5/16" E of col. N 47'-10" N of C of cont. vessel EL. 626'-8"	I		
(ICS)	ICS-H8	13'-8 5/16" E of col. N 97'-10" N of C of cont. vessel EL. 627'-0"	I		
(ICS)	ICS-H9	8'-7 1/8" E of col. N 52'-2" N of col. E. EL. 644'-6"	I	X	
(ICS)	ICS-H10	49'-6" R from C of cont. vessel 8'-7 1/8" N of col. E EL. 626'-8"	I		
(ICS)	ICS-H11	49'-6" R from C of cont. vessel 8'-7 1/8" N of col. 5 EL. 627'-0"	I		
(ICS)	ICS-H12	52'-1 7/8" from C of cont. vessel 9'-0 5/8" N from C of cont. vessel EL. 649'-6"	I		
Main Steam (MS)	MS-H15A	4'-8" N of col. 6 1'-0 5/16" W of col. J EL. 664'-6"	A		

Amendment No. 14

Table TS 3.14-1
Safety Related Shock Suppressors (Snubbers)
Page 5 of 6

<u>System Name</u>	<u>Snubber I.D. Number</u>	<u>Approximate Location & Elevation</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>Difficult to Remove (X)</u>	<u>* High Radiation Area at Shutdown (X)</u>
(MS)	MS-H80	0'-9" S of col. 6 6'-8 3/8" W of col. F EL. 619'-2"	A		
(MS)	MS-H85	4'-11 1/2" N of col. 5 4'-0 1/4" E of col. G EL. 619'-6"	A		
(MS)	MS-H86	11'-0" N of col. 7 8'-9 7/8" E of col. G EL. 619'-5 15/16"	A		
(MS)	MS-H87	12'-6" N of col. 7 9'-6" E of col. G EL. 619'-5 7/8"	A		
(MS)	MS-H95	5'-0" N of col. 6 1'-10" E of col. L EL. 664'-10 1/2"	A	X	
(MS)	MS-H96	3'-6" S of col. 6N 6'-5 1/16" W of col. H EL. 664'-4 1/8"	A		
(MS)	MS-H97	6'-6" N of col. 7N 5'-11 15/16" W of col. H EL. 664'-2 1/4"	A		
(MS)	MS-H98	8'-0" N of col. 7 1'-4" W of col. G EL. 652'-6"	A		
(MS)	MS-H99	23'-6 13/16" N of col. 7 2'-11" W of col. E EL. 619'-1 13/16"	A		

Table TS 3.14-1
Safety Related Shock Suppressors (Snubbers)
Page 6 of 6

<u>System Name</u>	<u>Snubber I.D. Number</u>	<u>Approximate Location & Elevation</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>Difficult to Remove (X)</u>	<u>* High Radiation Area at Shutdown (X)</u>
(MS)	MS-H94	13'N of col. 6 6' W of col. LW EL. 664'-11"	A	X	
Aux. Coolant (AC)	RAC-H-75	36'-0" N of col. W 1'-9 5/8" E of col. N EL. 601'-3 1/4"	I		
(AC)	RAC-H-76	6'-6" N of col. W 24'-5 21/32" W of col. N EL. 601'-1 1/4"	I		
(AC)	RAC-H-39	9'-9 5/16" S of col. W 34'-1 1/16" W of col. EL. 559'-8 1/4"	I		
(AC)	RAC-H-38	9'-4 5/16" S of col. W. 34'-1 8/16" W of col. EL. 559'-1 1/8"	I		
(AC)	RAC-H-21	39'-9 3/4" N of col. E 9'-8 1/16" E of col. N EL 600'-8 1/4"	I		

*Modifications to this Table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

4.14 TESTING AND SURVEILLANCE OF SHOCK SUPPRESSORS (SNUBBERS)

Applicability

Applies to periodic testing and surveillance requirements of safety related shock suppressors.

Objective

To verify operability of shock suppressors.

Specification

The following surveillance and testing is required for hydraulic shock suppressors required to be operable by Specification 3.14:

- a. All hydraulic shock suppressors whose seal material has been demonstrated by operating experience, lab testing or analysis to be compatible with the operating environment shall be visually inspected to verify integrity of hydraulic fittings, reservoirs and cylinders and mechanical integrity of linkage connections to piping and anchors. These inspections shall be in accordance with the following schedule:

<u>Number of hydraulic shock suppressors found inoperative during inspection or during inspection interval</u>	<u>Next required inspection interval</u>
0	18 months \pm 25%
1	12 months \pm 25%
2	6 months \pm 25%
3 - 4	124 days \pm 25%
5 - 7	62 days \pm 25%
\geq 8	31 days \pm 25%

The required inspection interval shall not be lengthened more than one step per inspection interval.

All hydraulic shock suppressors whose seal materials are other than ethylene propylene or other material that has been demonstrated to be compatible with the operating environment shall be visually inspected for operability every 31 days.

Shock suppressors are categorized as "accessible" or "inaccessible" as noted on Table TS 3.14-1. For the purpose of this inspection these two groups may be considered independently and scheduled accordingly.

- b. A representative sample of six shock suppressors or 10%, whichever is less, shall be functionally tested for operability including verification of proper piston movement, lockup, and bleed at each refueling. For each shock suppressor or subsequent shock suppressor found inoperable by this testing requirement, an additional 10% or six hydraulic shock suppressors shall be tested until no more failures are found or all shock suppressors have been tested. Those shock suppressors designated to be difficult to remove or in a high radiation area during shutdown as noted on Table TS 3.14-1 need not be selected for functional testing. The Anchor Holth suppressors used on the steam generators are exempt from functional testing requirements.
- c. The initial inspection shall be performed at the cycle 3 refueling. For the purpose of entering the schedule in Specification 4.14.a, it shall be assumed that the facility had been on a 12 month inspection interval.

Basis

All safety related hydraulic shock suppressors are visually inspected for overall integrity and operability. The inspection will include verification of proper orientation, adequate hydraulic fluid level and proper attachment of snubber to piping and structures.

The inspection frequency is based upon maintaining a constant level of snubber protection. Thus the required inspection interval varies inversely with the observed snubber failures. The number of inoperable snubbers found during a required inspection determines the time interval for the next required inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

Experience at operating facilities has shown that the surveillance program should assure an acceptable level of snubber performance provided that the seal materials are compatible with the operating environment.

Snubbers containing seal material which has not been demonstrated by operating experience, lab tests, or analysis to be compatible with the operating environment should be inspected more frequently (every month) until material compatibility is confirmed or an appropriate changeout is completed.

Examination of defective snubbers at reactor facilities and material tests performed at several laboratories have shown that millable gum polyurethane deteriorates rapidly under the temperature and moisture conditions present in many snubber locations. Although molded polyurethane exhibits greater resistance to these conditions, it also may be unsuitable for application in the higher temperature environments. Data are not currently available to precisely define an upper temperature limit for the molded polyurethane. Lab tests and in-plant experience indicate that seal materials are available, primarily ethylene propylene compounds, which should give satisfactory performance under the most severe conditions expected in reactor installations.

To further increase the assurance of snubber reliability, functional tests should be performed once each refueling cycle. These tests will include stroking of the snubbers to verify proper piston movement and snubbing action. Ten percent or six snubbers, represents an adequate sample for such tests. Observed failures on these samples should require testing of additional units. The Anchor Holth suppressors used on the steam generators are exempt from the functional test requirement due to the impracticability of functionally testing 900 Kip suppressors.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 14 TO LICENSE NO. DPR-43

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

KEWAUNEE NUCLEAR POWER PLANT

DOCKET NO. 50-305

INTRODUCTION

During the summer of 1973, inspections at two reactor facilities revealed a high incidence of inoperable hydraulic shock suppressors (snubbers) manufactured by Bergen Paterson Pipesupport Corporation. As a result of those findings, the Office of Inspection and Enforcement required each operating reactor licensee to immediately inspect all Bergen Paterson snubbers utilized on safety systems and to reinspect them 45 to 90 days after the initial inspection. Snubbers supplied by other manufacturers were to be inspected on a lower priority basis.

Since a long term solution to eliminate recurring failures was not immediately available, the Division of Reactor Licensing sent a letter dated October 2, 1973, to operating facilities utilizing Bergen Paterson snubbers specifying continuing surveillance requirements and requesting a submittal of proposed Technical Specifications for a snubber surveillance program. On October 19, 1976, Wisconsin Public Service Corporation (licensee) proposed Technical Specifications for snubbers at Kewaunee. The Kewaunee snubbers are not Bergen Paterson snubbers. During our review of the proposed change, we found that certain modifications were necessary. These modifications were discussed with the licensee and have been incorporated into the proposed Technical Specifications.

EVALUATION

Snubbers are designed to prevent unrestrained pipe motion under dynamic loads as might occur during an earthquake or severe transient while allowing normal thermal movement during startup and shutdown.

Examination of defective snubbers at reactor facilities has shown that the high incidence of failures observed in the summer of 1973 was caused by severe degradation of seal materials and subsequent leakage of the hydraulic fluid. The basic seal materials used in Bergen Paterson snubbers were two types of polyurethane; a millable gum polyester type containing plasticizers and an unadulterated molded type. Material tests performed at several laboratories (Reference 1) established that the millable gum polyurethane deteriorated rapidly under the temperature and moisture conditions present in many snubber locations. Although the molded polyurethane exhibited greater resistance to these conditions, it also may be unsuitable for application in the higher temperature environments. Data are not currently available to precisely define an upper temperature limit for the molded polyurethane. The investigation indicated that seal materials are available, primarily ethylene propylene compounds, which should give satisfactory performance under the most severe conditions expected in reactor installations.

An extensive seal replacement program has been carried out at many reactor facilities. Experience with ethylene propylene seals has been very good with no serious degradation reported thus far. Although the seal replacement program has significantly reduced the incidence of snubber failures, some failures continue to occur. These failures have generally been attributed to faulty snubber assembly and installation, loose fittings and connections and excessive pipe vibrations. The failures have been observed in both PWRs and BWRs and have not been limited to units manufactured by Bergen Paterson. Because of the continued incidence of snubber failures, we have concluded that snubber operability and surveillance requirements should be incorporated into the Technical Specifications. We have further concluded that these requirements should be applied to all safety related snubbers, regardless of manufacturer, in all light water cooled reactor facilities.

We have developed the attached Technical Specifications and Bases to provide additional assurance of satisfactory snubbers performance and reliability. The specifications require that snubbers be operable

- (1) Report H. R. Erickson, Bergen Paterson to K. R. Goller, NRC, October 7, 1974, Subject: Hydraulic Shock Sway Arrestors

during reactor operation. Because snubber protection is required only during low probability events, a period of 72 hours is allowed for repair or replacement of defective units before the reactor must be shut down. The licensee will be expected to commence repair or replacement of a failed snubber expeditiously. However, the allowance of 72 hours is consistent with that provided for other safety-related equipment and provides for remedial action to be taken in accordance with 10 CFR 50.36(c)(2). Failure of a pipe, piping system, or major component would not necessarily result from the failure of a single snubber to operate as designed, and even a snubber devoid of hydraulic fluid would provide support for the pipe or component and reduce pipe motion. The likelihood of a seismic event or other initiating event occurring during the time allowed for repair or replacement is very small. Considering the large size and difficult access of some snubber units, repair or replacement in a shorter time period is not practical. Therefore, the 72 hour period provides a reasonable and realistic period for remedial action to be taken.

An inspection program is specified to provide additional assurance that the snubbers remain operable. The inspection frequency is based upon maintaining a constant level of snubber protection. Thus the required inspection interval varies inversely with the observed snubber failures. The longest inspection interval allowed in the Technical Specifications after a record of no snubber failures has been established is nominally 18 months. Experience at operating facilities has shown that the required surveillance program should provide an acceptable level of snubber performance provided that the seal materials are compatible with the operating environment. Snubbers containing seal material which has not been demonstrated to be compatible with the operating environment are required to be inspected every 31 days until the compatibility is established or an appropriate seal change is completed.

To further increase the level of snubber reliability, the Technical Specifications require functional tests. The tests will verify proper piston movement, lock up and bleed.

We have concluded that the proposed additions to the Technical Specifications, as modified, increase the probability of successful snubber performance, increase reactor safety and we therefore find them acceptable.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this

determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 1, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-305

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 14 to Facility Operating License No. DPR-43, issued to Wisconsin Public Service Corporation, Wisconsin Power & Light Company, and Madison Gas & Electric Company (the licensees), which revised Technical Specifications for operation of the Kewaunee Nuclear Power Plant, located in Kewaunee, Wisconsin. The amendment is effective as of its date of issuance.

The operation of shock suppressors is required to protect the reactor coolant system and all other safety related systems and components and was assumed in the Staff Safety Evaluation Report. Operating history of other plants have indicated that shock suppressors were not always operable. Accordingly, this amendment requires the operability and surveillance of safety related shock suppressors.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 19, 1976, (2) Amendment No. 14 to License No. DPR-43, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C. and at the Kewaunee Public Library, 314 Milwaukee Street, Kewaunee, Wisconsin 54216. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 1st day of March 1977.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

PRELIMINARY DETERMINATION

NOTICING OF PROPOSED LICENSING AMENDMENT

PD-67

Licensee: Kewaunee

Request for: License amendment for Tech Spec Additions concerning hydraulic
snubber requirements .

Request Date: August 15, 1975

Proposed Action: () Pre-notice Recommended
(X) Post-notice Recommended
() Determination delayed pending
completion of Safety Evaluation

Basis for Decision: This amendment is at the request of NRC and increases the
probability of successful snubber performance thereby
increasing reactor safety. This change proposed by the
license constitutes an additional LCO for operability of
snubbers not presently included in the Technical Specifications
and does not result from an unreviewed safety question.
(ref. example 4, App 3b, RLOP 601) The change does not
involve an increase in the probability or consequences of
accidents previously considered, and does not involve a
decrease in safety margin. The proposed change is inadequate

OELD believes no pre-notice
is necessary because there is
no significant hazards consideration

(52.105(a)(3))

- Penny Seffner 9/24/75

CONCURRENCES:

DATE: (over)

1. J. D. Neighbors 9/5
2. R. A. Purple 9/5
3. K. R. Collier 9/5
4. Office of Executive Legal Director

with respect to our position on surveillance requirements
for snubbers; an order pursuant to §2.204 may be necessary.

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completion of Safety Evaluation

Basis for Decision: This amendment is at the request of NRC and increases
the probability of successful snubber performance.
thereby increasing reactor safety. The change does not
involve a significant hazards consideration.

This Tech Spec change is identical to one issued for
Pilgrim which was not pre-noticed.

Probably
NO Good

This Applicant refuses to give you the
surveillance program.
So Pilgrim precedent
doesn't help.

CONCURRENCES:

DATE:

1. J. D. Neighbors 8/25/75
2. for R. A. Purple 8/25/75
3. K. R. Collier KRG, 8/25
4. Office of Executive Legal Director

J. [Signature]
9/3/75