JUL 03 1984

Docket No. 50-305

Mr. C. W. Giesler, Vice President
Nuclear Power
Wisconsin Public Service Corporation
Post Office Box 1200
Green Bay, Wisconsin 54305

Dear Mr. Giesler:

The Commission has issued the enclosed Amendment No. 55 to Facility Operating License No. DPR-43 for the Kewaunee Nuclear Power Plant. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated January 13, 1984, as supplemented March 13, 1984.

The amendment revises the Technical Specifications as required by 10 CFR 50.55a(g)4(ii) to comply with an updated ISI program. This amendment revises items in the areas of Limiting Conditions for Operation, Surveillance Requirements, and Administrative Controls.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

Original signed by:
Joseph D. Neighbors, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. to DPR-43
2. Safety Evaluation

cc: w/enclosures
See next page

DISTRIBUTION:

Docket File	NRC PDR
L PDR	ORB#1 reading
DEisenhut	CParrish
DNeighbors	OELD
LHarmon	EJordan
JNGrace	TBarnhart, 4
WJones	DBrinkman
ACRS, 10	OPA, CMiles
RDiggs	

ORB#1:DL	ORB#1:DL	ORB#1:DL	OELD	AD/OR:DL
CParrish/dn	DNeighbors	SVarga		GLainas
6/20/84	6/20/84	6/21/84	6/29/84	6/3/84

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN PUBLIC SERVICE CORPORATION
WISCONSIN POWER AND LIGHT COMPANY
MADISON GAS AND ELECTRIC COMPANY

DOCKET NO. 50-305

KEWAUNEE NUCLEAR PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 55
License No. DPR-43

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Public Service Corporation, Wisconsin Power and Light Company, and Madison Gas and Electric Company (the licensee) dated January 13, 1984, as supplemented March 13, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-43 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical Specifications

Date of Issuance: July 3, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE NO. DPR-43

DOCKET NO. 50-305

Revise Appendix A as follows:

Remove Pages	Insert Pages
ii	ii
3.3-2	3.3-2
4.2-1	4.2-1
4.2-2	4.2-2
4.2-2.A	4.2-2.A
4.2-7	4.2-7
4.2-8	4.2-8
4.2-8A	4.2-8A
4.5-2	4.5-2
4.5-3	4.5-3
4.5-4	4.5-4
4.8-1	4.8-1
Table 4.1-3 (page 1 of 2)	Table 4.1-3 (page 1 of 2)
Table 4.2-1 (pages 1 thru 14)	

F.1. Isolation valves 8806A (S19A), 8801A (S111A) and 8801B (S111B) in the discharge of the high head SIS and block valve 8809C (S13) are in the open position with their power breaker locked out

2. Accumulator isolation valves 8800A (SI20A) and 8800B (SI20B) shall be opened with their power breaker locked out before reactor coolant system pressure exceeds 1000 psig.

C. Automatic valves, instrumentation, piping, and interlocks associated with the above components and required to function during accident and/or post-accident conditions are operable.

H. During the Quarterly Valve Operation Surveillance Testing of the Safety Injection System it is permissible to close the hand operated valve isolating the Concentrated Boric Acid Tanks from the Safety Injection Pump Suction. During this short test period an operator shall stand by the valve to open it if Safety Injection is required. He will have headset communication with the Control Room. At completion of the test he will verify the valve is returned to open, and this will be checked by at least one additional person.

2. During power operation or recovery from inadvertent trip, any one of the following conditions of inoperability may exist during the time intervals specified. The reactor shall be placed in the hot shutdown condition if operability is not restored within the time specified, and it shall be placed in the cold shutdown condition if operability is not restored within an additional 48 hours.

A. ONE safety injection pump may be out of service, provided the pump is restored to operable status within 24 hours. The other safety injection pump shall be tested to demonstrate operability prior to initiating repair of the inoperable pump.

B. ONE residual heat removal pump may be out of service, provided the pump is restored to operable status within 24 hours. The other residual heat removal pump shall be tested to demonstrate operability prior to initiating repair of the inoperable pump.

C. ONE residual heat exchanger may be out of service for a period of no more than 48 hours.

4.2 ASME CODE CLASS IN-SERVICE INSPECTION AND TESTING

Applicability

Applies to in-service structural surveillance of the ASME Code Class components and supports and functional testing of pumps and valves.

Objective

To assure the continued integrity and operational readiness of ASME Code Class 1, 2 and 3 components.

Specification

a. ASME Code Class 1, 2 and 3 Components and Supports

1. Inservice inspection of ASME Code Class 1, Class 2 and Class 3 components and supports shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50 Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55a(g)(6)(i). The testing and surveillance of shock suppressors (snubbers) is detailed in Technical Specification Sections 3.14 and 4.14.

2. Inservice testing of ASME Code Class 1, Class 2 and Class 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).

3. Surveillance testing of pressure isolation valves:

   a. Periodic leakage testing (1) on each valve listed in Table TS 3.1-2 shall be accomplished prior to entering the operating mode after every time the plant is placed in the cold shutdown condition for refueling, after each time the plant is placed in a

cold shutdown condition for 72 hours if testing has not been accomplished in the preceding 9 months, and prior to returning the valve to service after maintenance, repair, or replacement work is performed.

b. Whenever integrity of a pressure isolation valve listed in Table TS 3.1-2 cannot be demonstrated, the integrity of the remaining pressure isolation valve in each high pressure line having a leakage valve shall be determined and recorded daily. In addition, the position of the other closed valve located in the high pressure piping shall be recorded daily.

(1) To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria.

Deleted

5. Reports

a. Following each inservice inspection of steam generator tubes, if there are any tubes requiring plugging, the number of tubes plugged shall be reported to the Commission within 15 days.

b. The results of the steam generator tube inservice inspection shall be included in the Annual Operating Report for the period in which this inspection was completed. This report shall include:

1. Number and extent of tubes inspected.

2. Location and percent of wall-thickness penetration for each indication of a degradation.

3. Identification of tubes plugged.

c. Results of steam generator tube inspection which fall into Category C-3 and require prompt notification of the Commission shall be reported pursuant to Specification 6.9.2.a. prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

Basis

The plant was not specifically designed to meet the requirements of Section XI of the ASME Code; therefore, 100 percent compliance may not be feasible or practical. However, access for inservice inspection was considered during the design and modifications have been made where practical to make provisions for maximum access within the limits of the current plant design. Where practical, the inspection of ASME Code Class 1, Class 2 and Class 3 components is performed in accordance with Section XI of the ASME Code. If a code required inspection is impractical, a request for a deviation from the requirement is submitted to the Commission for approval.

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on the general guidance of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

DELETED

2. Containment Vessel Internal Spray System

A. System tests shall be performed once every operating cycle or once every 18 months, whichever occurs first. The test shall be performed with the isolation valves in the supply lines at the containment blocked closed. Operation of the system is initiated by tripping the normal actuation instrumentation.

B. The spray nozzles shall be checked for proper functioning at least every five years using either air with telltales or smoke tests to determine that all nozzles are clear.

C. The test will be considered satisfactory if control board indications or visual observations indicate all components have operated satisfactorily.

3. Containment Fan-Coil Units

Each fan-coil unit shall be tested once every operating cycle or once every 18 months, whichever occurs first, to verify proper operation of the motor-operated service water outlet valves.

b. Component Tests

1. Pumps

A. The safety injection pumps, residual heat removal pumps, and containment spray pumps shall be started and operated on recirculation flow quarterly during power operation and within one week after the plant is returned to power operation, if the test was not performed during plant shutdown. |

B. Acceptable levels of performance shall be that the pumps start, reach their required developed heat at miniflow, and operate for at least fifteen minutes on the miniflow line.

2. Valves

A. The Refueling Water Storage Tank and containment sump outlet valves shall be tested in performing the pump tests.

B. The accumulator check valves shall be checked for operability during each major refueling outage. The accumulator block valves shall be checked to assure "valve open" requirements during each major refueling outage.

C. The boric acid tank isolation valves to the safety injection pumps shall be tested at intervals not to exceed quarterly during power operation.

D. Spray additive tank valves shall be tested during each major refueling outage.

E. Closing of the boric acid tank isolation valves and concurrent opening of refueling water storage tank valves upon receipt of simulated Lo Lo boric acid tank level signal shall be tested at intervals not to exceed quarterly during power operation.

F. Residual Heat Removal System valve interlocks shall be tested once per operating cycle (not to exceed 18 months).

Basis

The Safety Injection System and the Containment Vessel Internal Spray System are principal plant safety systems that are normally inoperative during reactor operation. Complete systems tests cannot be performed when the reactor is operating because a safety injection signal causes containment isolation and a Containment Vessel Internal Spray System test requires the system to be temporarily disabled. The method of assuring operability of these systems is therefore to combine system tests to be performed during periodic shutdowns with more frequent component tests, which can be performed during reactor operation.

The systems tests demonstrate proper automatic operation of the Safety Injection and Containment Vessel Internal Spray Systems. With the pumps blocked from starting, a test signal is applied to initiate automatic action and verification is made that the components receive the safety injection signal in the proper sequence. The test demonstrates the operation of the valves, pump circuit breakers, and automatic circuitry.(1)

During reactor operation, the instrumentation which is depended upon to initiate safety injection and containment spray is checked daily and the initiating and logic circuits are tested monthly (in accordance with Specification 4.1). In addition, the active components (pumps and valves) are to be tested quarterly to check the operation of the starting circuits and to verify that the pumps are in satisfactory running order. The quarterly test interval is based on the judgment that more frequent testing would not significantly increase the reliability (i.e., the probability that the component would operate when required), yet more frequent testing would result in increased wear over a long period of time.

Testing of the closure of the boric acid tank isolation valves with concurrent opening of the refueling water storage tank valves upon receipt of simulated lo-lo boric acid tank level signal is performed to verify proper operation to prevent inadvertent spillage of refueling water storage tank water through the boric acid tank should the isolation valves fail to close.

Other systems that are also important to the emergency cooling function are the accumulators, the Component Cooling System, the Service Water System, and the containment fan-coil units. The accumulators are a passive safety feature. In accordance with Specification 4.1, the water volume and pressure in the accumulators are checked each shift. The other systems mentioned operate when the reactor is in operation and by these means are continuously monitored for satisfactory performance.

Reference: (1) FSAR Section 6.2

4.8 AUXILIARY FEEDWATER SYSTEM

Applicability

Applies to periodic testing requirements of the turbine-driven and motor-driven auxiliary feedwater pumps.

Objective

To verify the operability of the auxiliary feedwater equipment and its ability to respond properly when required.

Specification

a. The operability of individual auxiliary feedwater pumps as required by Specification 3.4.a.2 shall be demonstrated quarterly during power operation and within one week after the pumps are required to be operable by the Technical Specifications, if the test was not performed during plant shutdown.

b. The valves on the discharge side of the turbine-driven pump that direct flow to either steam generator shall be tested by operator action whenever the turbine-driven pump is tested.

c. The service water supply valves to the auxiliary feedwater pump suctions shall be tested by operator action following the auxiliary feedwater pump tests.

d. These tests shall be considered satisfactory if control board indication or visual observation of the equipment demonstrate that all components have operated properly.

Basis

Quarterly testing of the auxiliary feedwater pumps will verify their operability. The discharge valves of the two motor-operated pumps are normally open, as are the suction valves from the condensate storage tanks and the two valves on a cross tie line that directs the turbine driven pump discharge to either or both steam generators. The only valve required to function upon initiation of

TABLE 4.1-3

MINIMUM FREQUENCIES FOR EQUIPMENT TESTS
(Page 1 of 2)

	Equipment Tests***	Test	Frequency	Maximum Time Between Test (Days)
1.	Control Rods	Rod drop times of all full length rods	Each refueling outage	N.A.
		Partial movement of all rods	Every 2 weeks	17
1a.	Reactor Trip Breakers	Open trip	Monthly	37
1b.	Reactor Coolant Pump Breakers-Open-Reactor Trip	Operability	Each refueling outage	N.A.
2.	Deleted			
3.	Deleted			
4.	Containment Isolation Trip	Operability	Each refueling outage	N.A.
5.	Refueling System Interlocks	Operability	Prior to each refueling outage	N.A.
6.	Ventilation System a. Shield Building b. Auxiliary Building SV Zone c. Spent Fuel Pool	Halide, DOP and Methyl Iodide Pressure Drop Test Visual Inspection	During each refueling outage except as specified in Note**	N.A.
7.	Fire Protection Pump and Power Supply	*Operability	Monthly	37
8.	RCS Leak Detection	Operability	Weekly	8
9.	Diesel Fuel Supply	*Fuel inventory	Weekly	8
10.	Turbine Stop and Governor Valves	Operability	Monthly(1)(2)	37(1)(2)
11.	Fuel Assemblies	Visual Inspection	Each refueling outage	N.A.
12.	Guard Pipes	Visual Inspection	Each refueling outage	N.A.

Table TS 4.2-1 has been deleted.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE NO. DPR-43

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

KEWAUNEE NUCLEAR POWER PLANT

DOCKET NO. 50-305

By letter dated January 13, 1984, as supplemented March 13, 1984, Wisconsin Public Service Corporation submitted a proposed amendment to the KNPP Technical Specifications. The proposed amendment revises the Technical Specifications to reflect the inservice inspection requirements and methods of the 1980 edition of Section XI of the ASME Code up to and including the Winter 1981 Addenda, per 10 CFR 50.55a(b)2 including pumps and valves. The updated inservice inspection program became effective on June 16, 1984 and will be in effect for a 120 month inspection interval. However, this amendment does not approve the updated program except as stated herein. The updated program is still under review.

An earlier proposed amendment submittal, dated August 24, 1983 and updated April 25, 1984 also modifies some of the pages affected by this proposed amendment. Since the earlier submittal has not yet been approved, the proposed Technical Specification pages affected by both the earlier submittal and this submittal have been modified to delete the earlier changes. This affects pages TS ii and changes affected page TS 3.3-3 in the submittal to page TS 3.3-2.

This safety evaluation is a review of the requested changes and their impact on the inservice inspection program as well as their conformance with the 1980 Code requirements.

SUMMARY OF EVALUATION

The changes proposed by the licensee, discussed in detail below, are primarily of an administrative nature and do not adversely affect the safety of the plant nor the general public. The changes consist of changing the testing frequency requirements to meet the requirements of the 1980 edition of the ASME Code and applicable addenda. The present inservice inspection requirements in the Technical Specifications have been deleted. These inspection requirements are now addressed in the licensee's inservice program based upon Section XI of the ASME Code submitted in accordance with the requirements of 10 CFR 50.55a.

The staff agrees with the changes as described in the proposed amendment.

EVALUATION

In accordance with 10 CFR 50.55a(g)4(i) and 10 CFR 50.55a(g)5(i), Wisconsin Public Service Corporation (WPSC), has prepared an updated inservice inspection program to conform to the inspection requirements of the 1980 edition of Section XI of the ASME Boiler and Pressure Vessel Code. Regulation 10 CFR 50.55a(g)5(ii), requires that an amendment to the facility Technical Specifications be submitted to the Commission for review and approval if a revised or updated inservice inspection program conflicts with the Technical Specifications. The following changes proposed by the licensee remove the inservice inspection table and change test frequencies to agree with Section XI, since these conditions are committed to in the inservice inspection plan submitted on March 30, 1984.

Proposed Changes to TS ii

DESCRIPTION OF CHANGES

Specification 4.2 and 4.2.a have been retitled to indicate that inservice inspection and testing will be conducted per Section XI of the ASME Code and applicable addenda requirements on ASME Code Class 1, 2 and 3 components and supports.

EVALUATION

This change is purely administrative in nature and has no effect on safety. The new titles more accurately describe the bases and methods to be used for surveillances. This change is acceptable.

Proposed Changes to Pages TS 3.3-2, TS 4.5-2, TS 4.5-3, TS 4.5-4, TS 4.8-1

DESCRIPTION OF CHANGES

Test frequency has been decreased from monthly to quarterly for operational surveillance testing of pumps and valves in the safety injection systems, containment spray systems, boric acid tank isolation valves, refueling water storage tank valves, and auxiliary feedwater pumps.

EVALUATION

Test frequency has been changed in order to comply with the 1980 edition of Section XI of the ASME Code and applicable addenda requirements as defined in the KNPP Inservice Inspection and Testing Program, dated March 30, 1984. These changes are acceptable.

Proposed Changes on Pages TS 4.2-1, TS 4.2-2, TS 4.2-2A

DESCRIPTION OF CHANGES

Specification 4.2.a has been retitled "ASME Code Class 1, 2 and 3 Components and Supports." The section was reorganized so that the specification now agrees with the wording of Standard Technical Specifications, requiring that an inspection and testing program be developed and implemented in accordance with the applicable ASME Code and Addenda as required by the Code of Federal Regulations. In addition, a reference to the testing and surveillance of shock suppressors, in accordance with Technical Specifications 3.14 and 4.14, has been added to this section for completeness.

EVALUATION

The basis for surveillances has been modified to reflect the current 10 CFR 50 requirements in accordance with paragraphs 50.55a(g)5(ii) and 50.55a(g)4(ii). Revised Section 4.2 and the submitted Inservice Testing Plan, dated March 30, 1984, commits the licensee to perform inservice inspections and testing of Class I, II and III components in conformance with Section XI of the ASME Boiler and Pressure Code and applicable addenda through the Winter 1981 Addenda.

The revised page TS 4.2-1 inaccurately called out 10 CFR 50, Section 50.55a (g)(6)(i) as Section 50.55(g)(6)(i). Also Section 50.55a (g) was mislabelled as Section 50.55(g). These corrections have been made to the attached Technical Specification pages.

This change is acceptable.

Proposed Changes for Pages TS 4.2-7, TS 4.2-8, TS 4.2-8A

DESCRIPTION OF CHANGE

The Basis for Section 4.2 has been modified to reflect the changes discussed in the previous item.

EVALUATION

The discussion of the inspection program and examination requirements has been deleted as these items are addressed in detail in the KNPP Inservice Inspection and Testing Program and Section XI of the ASME Code. Inclusion of this portion of the Technical Specifications would be redundant as the inspection guidelines are addressed in the change discussed immediately preceding. This change is acceptable.

Proposed Changes to Table 4.1.3 (Page 1 of 2)

DESCRIPTION OF CHANGE

Test frequencies for the Pressurizer Safety Valves and the Main Steam Safety Valves have been deleted.

EVALUATION

Testing methods, requirements and frequencies of tests for these valves are discussed in detail in Section XI of the ASME Code, and need not be addressed further in the Technical Specification. This change is acceptable.

Proposed Changes to Table 4.2-1 (Pages 1-14)

DESCRIPTION OF CHANGE

Table 4.2-1 is deleted in its entirety.

EVALUATION

Table 4.2-1, "In-Service Inspection" describes the inservice inspection requirements for various components in the plant. These requirements are discussed in detail in the Submitted KNPP Inservice Inspection and Testing Program and Section XI of the ASME Code and need not be further addressed in the Technical Specifications. In addition, Table 4.2-1 also included testing which is no longer required by the Code. This change is acceptable.

The licensee's letter dated March 13, 1984, proposed a change in the wording of the objective of Technical Specification 4.2. This change would have changed the terms "important to safety" to "safety related". Discussion with the licensee resulted in changing the wording to reflect the integrity and readiness of ASME Code Class 1, 2 and 3 components. These words are taken from Technical Specification 4.2.a and make the Technical Specification consistent with its objective. This is an insignificant change and is for clarification and does not substantially affect the intent of the January 13, 1984 application.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area. The staff has determined that the amendment involves no significant increase in the amounts of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupation radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated:

Principal Contributors:
P. L. Eng
R. Nelson