

Docket No. 50-305

DEC 18 1975

Wisconsin Public Service Corporation
ATTN: Mr. E. W. James
Senior Vice President
Post Office Box 1200
Green Bay, Wisconsin 01581

Gentlemen:

The Commission has issued the enclosed Amendment No. 5 to Facility Operating License No. DPR-43 for the Kewaunee Nuclear Power Plant. This amendment includes Change No. 7 to the Technical Specifications and is in response to your request dated January 15, 1975.

The amendment incorporates into the Kewaunee Nuclear Power Plant Technical Specifications changes to the reporting requirements. Changes to your proposal were necessary to meet our requirements. These have been discussed with your staff. The technical specifications are based on Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4.

We request that you use the formats presented in the Appendices to Regulatory Guide 1.16, Revision 4, for reporting operating information and that you report events of the type described under the section "Events of Potential Public Interest". Instructions for using these reporting formats are contained in Regulatory Guide 1.16 (a copy is enclosed for your use), and AEC report OGE-SS-001 titled "Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File" (a copy of which was provided you previously). This report is modified by updated instructions dated December 8, 1975, which are enclosed. Copy requirements are summarized in Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulations", a copy of which is also enclosed. This Guide will assist you in identifying reports that are required by the Commission's regulations set forth in Title 10 Code of Federal Regulations but are not contained in your technical specifications. Reports that are required by the regulations have not been repeated in your technical specifications.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Original signed by
R. A. Purple
Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing

| | | | | | | | |
|-----------|---------------|--|--|--|--|--|--|
| OFFICE ▶ | Enclosures: | | | | | | |
| SURNAME ▶ | See next page | | | | | | |
| DATE ▶ | | | | | | | |

Wisconsin Public Service
Corporation

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DEC 18 1975

Enclosures:

1. Amendment No. 5
2. Regulatory Guide 1.16
3. Updated Instructions
4. Regulatory Guide 10.1
5. Safety Evaluation
6. Federal Register Notice

cc w/enclosures:

See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

DOCKET NO. 50-305

KEWAUNEE NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 5
License No. DPR-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Public Service Corporation (the licensee) dated January 15, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility License No. DPR-43 is hereby amended to read as follows:



"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change¹No. 1 0."

3. This license amendment is effective 30 days after the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing

Attachment:
Change No. 7 to the Technical
Specifications

Date of Issuance: DEC 18 1975

ATTACHMENT TO LICENSE AMENDMENT NO. 5
CHANGE NO. 7 TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-43
DOCKET NO. 50-305

Revise Appendix A as follows:

Remove TS pages

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3.6-1
4.9-1
6.1-1 thru 6.6-10

Tables TS 6.4-1 and Notes
Figures TS 6.1-1 and 6.1-2

Insert New TS pages

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iii
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1.0 DEFINITIONS

The following terms are defined for uniform interpretation of the specifications.

a. Quadrant-to-Average Power Tilt Ratio | 7

The quadrant-to-average power tilt ratio is defined as the ratio of maximum-to-average of the upper excore detector currents or that of the lower excore detector currents whichever is greater. If one excore detector is out of service, the three in-service units are used in computing the average.

b. Safety Limits | 7

Safety limits are the necessary quantitative restrictions placed upon those process variables that must be controlled in order to reasonably protect the integrity of certain of the physical barriers which guard against the uncontrolled release of radioactivity.

c. Limiting Safety System Settings | 7

Limiting safety system settings are set points for automatic protective devices responsive to the variables on which safety limits have been placed. These set points are so chosen that automatic protective actions will correct the most severe, anticipated abnormal situation so that a safety limit is not exceeded.

d. Limiting Conditions for Operation | 7

Limiting conditions for operation are those restrictions on reactor operation, resulting from equipment performance capability, that must be enforced to ensure safe operation of the facility.

e.

Operable

A system or component is operable when it is capable of performing its intended function within the required range. The system or component shall be considered to have this capability when: (1) it satisfies the Limiting Conditions for Operation defined in Specification 3, and (2) it has been tested periodically in accordance with Specification 4 and has met its performance requirements.

f.

Operating

A system or component is considered to be operating when it is performing the intended function in the intended manner.

g.

Containment System Integrity

Containment System integrity is defined to exist when:

1. The non-automatic Containment System isolation valves and blind flanges are closed as required.
2. The Reactor Containment Vessel and Shield Building equipment hatches are properly closed.
3. At least ONE door in both the personnel and the emergency airlocks is properly closed.
4. The required automatic Containment System isolation valves are operable or are deactivated in the closed position or at least one valve in each line having an inoperable valve is closed.
5. All requirements of Specification 4.4 with regard to Containment System leakage and test frequency are satisfied.
6. The Shield Building Ventilation System and the Auxiliary Building Special Ventilation System satisfy the requirements of Specification 3.6.b.

h.

Protective Instrumentation Logic

1. Protection System Channel

A protection system channel is an arrangement of components and modules as required to generate a single protective action signal when required by a plant condition. The channel loses its identity where single action signals are combined.

2. Logic Channel

A logic channel is a matrix of relay contacts which operate in response to protective system channel signals to generate a protective action signal.

3. Degree of Redundancy

Degree of redundancy is defined as the difference between the number of operating channels and the minimum number of channels which, when tripped, will cause an automatic shutdown.

4. Protection System

The protection system consists of both the Reactor Protection System and the Engineered Safety Features System. The protection system encompasses all electric and mechanical devices and circuitry (from sensors through actuated device) which are required to operate in order to produce the required protective function. Tests of protection system will be considered acceptable when tests are run in part and it can be shown that all parts satisfy the requirements of the system.

i. Instrumentation Surveillance

1. Channel Check

Channel check is a qualitative determination of acceptable operability by observation of channel behavior during operation. This determination shall include, where possible, comparison of the channel indication with other indications derived from independent channels measuring the same variable.

2. Channel Functional Test

A channel functional test consists of injecting a simulated signal into the channel as close to the primary sensor as practicable to verify that it is operable, including alarm and/or trip initiating action.

3. Channel Calibration

Channel calibration consists of the adjustment of channel output such that it responds, with acceptable range and accuracy, to known values of the parameter which the channel monitors. Calibration shall encompass the entire channel, including alarm and/or trip, and shall be deemed to include the channel functional test.

j.

Operating Modes

| <u>Mode</u> | <u>Reactivity $\Delta k/k$</u> | <u>Coolant Temp T_{avg} °F</u> | <u>Fission Power %</u> |
|------------------------------|---|---|----------------------------|
| Refueling | $\leq -10\%$ | ≤ 140 | ~ 0 |
| Cold Shutdown | $\leq -1\%$ | ≤ 200 | ~ 0 |
| Intermediate Shutdown | (1) | > 200 < 540 | ~ 0 |
| Hot Shutdown | (1) | ≥ 540 | ~ 0 |
| Hot Standby | $< 0.25\%$ | $\sim T_{oper}$ | < 2 |
| Operating | $< 0.25\%$ | $\sim T_{oper}$ | ≥ 2 |
| Low Power Physics Testing | (To be specified by specific tests) | | |

(1) Refer to Figure TS 3.10-1

k.

Reactor Critical

The reactor is said to be critical when the neutron chain reaction is self-sustaining.

1. Refueling Operation

Refueling operation is any operation involving movement of Reactor Vessel internal components (those that could affect the reactivity of the core) within the containment when the vessel head is unbolted or removed.

m. Rated Power

Rated power is the steady-state reactor system output of 1650 MWt.

n. Interim Fuel Limits

Interim Limits on core power distributions are those values used in the loss-of-coolant accident analysis to demonstrate compliance with (a) the AEC Interim Policy Statement published June 29, 1971, in the Federal Register, and (b) the Regulatory Staff's "Technical Report on Densification of Light Water Reactor Fuels", published November 14, 1972. The fuel residence time for cycle 1 shall be limited to 13,000 effective full-power hours under design operating conditions.

3.6 CONTAINMENT SYSTEM

Applicability

Applies to the integrity of the Containment System.

Objective

To define the operating status of the Containment System.

Specification

- a. Containment System integrity shall not be violated if there is fuel in the reactor which has been used for power operation, except whenever either of the following conditions remains satisfied:
 1. The reactor is in the cold shutdown condition with the reactor vessel head installed, or
 2. The reactor is in the refueling shutdown condition.

- b. All of the following conditions shall be satisfied whenever Containment System integrity as defined by Specification 1.0.g is required:
 1. BOTH trains of the Shield Building Ventilation System including filters and heaters are operable, or one train is operable as confirmed by demonstration and the other has been inoperable for no more than seven days under conditions for which containment integrity is required by Specification 3.6.a.
 2. BOTH trains of the Auxiliary Building Special Ventilation System including filters and heaters are operable, or one train is operable as confirmed by demonstration and the other has been inoperable for no more than seven days under conditions for which containment integrity is required by Specification 3.6.a.

Applicability

Applies to potential reactivity anomalies.

Objective

To require evaluation of reactivity anomalies within the reactor.

Specification

Following a normalization of the computed boron concentration as a function of burnup, the actual boron concentration of the coolant shall be periodically compared with the predicted value. If the difference between the observed and predicted steady-state concentrations reaches the equivalent of one percent in reactivity, an evaluation as to the cause of the discrepancy shall be made and reported pursuant to Specification 6.9.2.

Basis

To eliminate possible errors in the calculations of the initial reactivity of the core and the reactivity depletion rate, the predicted relation between fuel burn-up and the boron concentration, necessary to maintain adequate control characteristics, must be adjusted (normalized) to accurately reflect actual core conditions. When full power is reached initially, and with the control rod groups in the desired positions, the boron concentration is measured and the predicted curve is adjusted to this point. As power operation proceeds, the measured boron concentration is compared with the predicted concentration and the slope of the curve relating burn-up and reactivity is compared with that predicted. This process of normalization should be completed after about 10% of the total core burn-up. Thereafter, actual boron concentration can be compared with prediction, and the reactivity

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Superintendent has overall on-site responsibility for plant operation. In the absence of the Plant Superintendent, the succession to this responsibility shall be in the following order:

- a. Assistant Plant Superintendent - Maintenance
- b. Assistant Plant Superintendent - Operations
- c. Operations Supervisor
- d. Technical Supervisor

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for plant management and technical support shall be as shown on Figure TS 6.2-1.

FACILITY STAFF

6.2.2 The plant organization shall be as shown on Figure TS 6.2-2 and:

- a. Each on-duty shift complement shall consist of at least:
 - (1) One Shift Supervisor (SRO)
 - (2) Two licensed Reactor Operators
 - (3) One Auxiliary Operator
 - (4) One Equipment Operator
- b. In the event that one of the shift members becomes incapacitated due to illness or injury, reactor operations may continue with the reduced complement until his replacement arrives. In all but severe weather conditions, a replacement is required within two hours.

- c. At least one licensed operator shall be in the control room when fuel is in the reactor.
- d. At least two licensed operators shall be present in the control room during reactor startup, turbine generator synchronization to the grid, and during recovery from reactor trips.
- e. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor. This individual may be one of the shift operators.
- f. Refueling operations shall be directed by a licensed Senior Reactor Operator assigned to the refueling operation who has no other concurrent responsibilities during the refueling operation.

6.3 PLANT STAFF QUALIFICATIONS

6.3.1 Qualifications of each member of the Plant Staff shall meet or exceed the minimum acceptable levels of ANSI-N18.1-1971 for comparable positions.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the Plant Staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI-N18.1-1971 and Appendix A of 10 CFR Part 55.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The PORC shall function to advise the Plant Superintendent on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PORC shall be composed of, but not necessarily limited to:

Chairman: Plant Superintendent
Required Members: Assistant Plant Superintendent - Maintenance
Assistant Plant Superintendent - Operations
Operations Supervisor
Reactor Supervisor
Technical Supervisor
Plant Performance Engineer

ALTERNATES

6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PORC meetings at any one time.

MEETING FREQUENCY

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the Chairman.

QUORUM

6.5.1.5 A quorum of the PORC shall consist of the Chairman or his designated successor plus three members of which not more than two shall be alternates.

RESPONSIBILITIES

6.5.1.6 The PORC shall be responsible for:

- a. Review proposed normal, abnormal and emergency operating procedures. Review of all proposed maintenance procedures and proposed changes to those procedures; and any other proposed procedures or changes thereto as determined by the Plant Superintendent to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.

- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Review all reports covering the investigation of all violations of the Technical Specifications and the recommendations to prevent recurrence.
- f. Review plant operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and prepare reports thereon as requested by the Chairman of the Nuclear Safety Review and Audit Committee.
- h. Review of the Security Plan and Emergency Plan and their respective implementing procedures and shall submit recommended changes to the Superintendent - Nuclear Power.

AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend to the Plant Superintendent approval or disapproval of items considered under 6.5.1.6a through d above.
- b. Make determinations with regard to whether or not each item considered under 6.5.1.6a through e above constitutes an unreviewed safety question.

- c. Provide immediate notification in the form of draft meeting minutes to the Superintendent - Nuclear Power and the Chairman - Nuclear Safety Review and Audit Committee of disagreement between the PORC and the Plant Superintendent. The Plant Superintendent shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

- 6.5.1.8 Minutes shall be kept of all meetings of the PORC and copies shall be sent to the Superintendent - Nuclear Power and the Chairman - Nuclear Safety Review and Audit Committee.

6.5.2 CORPORATE NUCLEAR ENGINEERING STAFF (CNES)

FUNCTION

- 6.5.2.1 The CNES shall function to provide engineering, technical and quality assurance activities in support of the Kewaunee Plant Staff.

ORGANIZATION

- 6.5.2.2 The CNES consists of the following groups:

- a. Licensing and Nuclear Systems
- b. Nuclear Services
- c. Steam Plant Engineering
- d. Quality Assurance and Control

AUTHORITY

6.5.2.3 Activities

1. Investigate and report all violations of the Technical Specifications, codes, regulations, statutes.

2. Review all activities associated with nuclear safety for technical adequacy and compliance with internal procedures or instructions.
3. Investigate and report significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
4. Investigate and report all events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours.
5. Investigate any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems or components.
6. Review and or prepare safety evaluations of all plant design changes.
7. Audits as required by the Quality Assurance Program and as outlined in Section 6.5.3.8.

AUTHORITY

6.5.2.4 The CNES is responsible to the Superintendent - Nuclear Power. Steam Plant Engineering, although directly responsible to the Superintendent - Steam Plants, is available to the Superintendent - Nuclear Power for special projects and support as it applies to the Kewaunee Plant.

6.5.3 NUCLEAR SAFETY REVIEW AND AUDIT COMMITTEE (NSRAC)

- 6.5.3.1 The NSRAC shall function to provide independent review and audit of designated activities in the areas of:
- a. Nuclear Power Plant Operations
 - b. Nuclear Engineering

- c. Chemistry and Radio-Chemistry
- d. Metallurgy
- e. Instrumentation and Control
- f. Radiological Safety
- g. Mechanical and Electrical Engineering
- h. Quality Assurance Practices
- i. Other appropriate fields as determined by the Committee, to be associated with the unique characteristics of the nuclear power plant.

COMPOSITION

6.5.3.2 The NSRAC shall be composed of, but not necessarily limited to:

Members: Superintendent - Nuclear Power
Superintendent - Kewaunee Plant*
Assistant Plant Superintendent - Operations
Mechanical Engineer
Quality Assurance Supervisor
Nuclear Fuels Supervisor
Consultants **

* Non-Voting Member

** Non-Company Affiliated

The Committee membership and its Chairman and Vice Chairman shall be appointed by the Senior Vice-President - Power Generation & Engineering or such person as he shall designate.

ALTERNATES

6.5.3.3 Alternate members shall be appointed by the NSRAC Chairman, upon approval by the Senior Vice President - Power Generation and Engineering, to serve on a temporary basis; however, no more than two alternates shall participate in NSRAC activities at any one time.

CONSULTANTS

6.5.3.4 Consultants may be utilized as determined by the Chairman - NSRAC to provide expert advice to the NSRAC.

MEETING FREQUENCY

6.5.3.5 The NSRAC shall meet at least once per calendar quarter during the initial year of plant operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.3.6 A quorum of the NSRAC shall consist of the Chairman or Vice Chairman and four members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the plant.

REVIEW

6.5.3.7 The NSRAC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or licenses.
- e. Reports covering violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.

- f. Reports covering significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Reports covering all events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours.
- h. Reports covering any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meeting minutes of the PORC.

AUDITS

- 6.5.3.8 Audits of plant activities shall be performed under the cognizance of the NSRAC; these audits shall include:
- a. Conformance of plant operation to all provisions contained within the Technical Specifications and applicable license conditions at least annually.
 - b. Performance, training and qualifications of the entire plant staff at least annually.
 - c. Results of all actions taken to correct deficiencies occurring in plant equipment, structures, systems or method of operation that affect nuclear safety at least semi-annually.
 - d. Performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once every two years.
 - e. Plant Emergency Plan and Security Plan and their implementing procedures at least once every two years.

- f. Any other area of plant operation considered appropriate by the NSRAC or the Senior Vice President - Power Generation & Engineering.

AUTHORITY

6.5.3.9 The NSRAC shall report to and advise the Senior Vice President - Power Generation & Engineering on those areas of responsibility specified in Section 6.5.3.7 and 6.5.3.8.

RECORDS

6.5.3.10 Records of NSRAC activities shall be prepared, approved and distributed as follows:

- a. Minutes of each NSRAC meeting forwarded to the Senior Vice President - Power Generation & Engineering within 14 days following each meeting.
- b. Reports of reviews required by Section 6.5.3.7 e, f, g and h above, forwarded to the Senior Vice President - Power Generation & Engineering within 14 days following completion of the review.
- c. Reports of audits performed by NSRAC shall be forwarded to the Senior Vice President - Power Generation & Engineering and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 DELETED

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a safety limit is violated:

- a. The reactor shall be shutdown and operation shall not be resumed until authorized by the Commission.
- b. The Safety Limit violation shall be reported to the Commission, the Superintendent - Nuclear Power and to the NSRAC - Chairman.
- c. The report shall be prepared in accordance with Section 6.9 of the Technical Specifications.

6.8 PROCEDURES

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet the requirements and recommendations of Section 5.1 and 5.3 of ANSI N18.7-1972.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the Plant Superintendent prior to implementation and periodically as determined by the Plant Superintendent.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the Plant Management Staff, at least one of which holds a Senior Reactor Operator's License, if the procedure affects nuclear safety.
- c. The change is documented, reviewed by the PORC and approved by the Plant Superintendent.

6.9 Reporting Requirements

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

6.9.1. Routine Reports

- a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

- b. Annual Operating Report.^{1/} Routine operating reports covering the operation of the unit during the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

The annual operating reports made by licensees shall provide a comprehensive summary of the operating experience gained during the year, even though some repetition of

previously reported information may be involved. References in the annual operating report to previously submitted reports shall be clear.

Each annual operating report shall include:

- (1) A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance not covered in item 1.b.(2)(e) below.
- (2) For each outage or forced reduction in power^{2/} of over twenty percent of design power level where the reduction extends for greater than four hours:
 - (a) the proximate cause and the system and major component involved (if the outage or forced reduction in power involved equipment malfunction);
 - (b) a brief discussion of (or reference to reports of) any abnormal occurrences pertaining to the outage or power reduction;
 - (c) corrective action taken to reduce the probability of recurrence, if appropriate;
 - (d) operating time lost as a result of the outage or power^{3/} reduction (for scheduled or forced outages, use the generator off-line hours; for forced reductions in power, use the approximate duration of operation at reduced power);
 - (e) a description of major safety-related corrective maintenance performed during the outage or power reduction, including the system and component involved and identification of the critical path activity dictating the length of the outage or power reduction; and
 - (f) a report of any single release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10% of the allowable annual values.

7

- (3) A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions, ⁴⁷ e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- (4) Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

- c. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the appropriate Regional Office, to arrive no later than the tenth of each month following the calendar month covered by the report.

6.9.2. Reportable Occurrences

Reportable occurrences, including corrective actions and measures to prevent reoccurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

a. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 2.a(5), 2.a(6), or 2.b(1) below.

(2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 2.b(2) below.

(3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

Note: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

- (4) Reactivity anomalies involving disagreement with the predicted value of reactivity balance under steady state conditions greater than or equal to \$1.00; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if subcritical, an unplanned reactivity insertion of more than 50¢; or occurrence of any unplanned criticality.
- (5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.
- (6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For items 2.a(5) and 2.a(6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 2.b(2) and 2.b(3) below.

- (7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
- (8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.
- (9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

Note: This item is intended to provide for reporting of potentially generic problems.

b.

Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- (1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- (2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 2.b(1) and 2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

- (3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
- (4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.9.3 Unique Reporting Requirements

a. Annual Environmental Operating Reports

- (1) For each medium sampled during the reporting period, e.g., air, lake bottom, surface water, soil, fish, include:
 - (a) Number of sampling locations,
 - (b) Total number of samples,
 - (c) Number of locations at which levels are found to be significantly above local backgrounds,
 - (d) Highest, lowest, and the average concentrations or levels of radiation for the sampling point with the highest average and description of the location of that point with respect to the site.
- (2) If levels of radioactive materials in environmental media as determined by an environmental monitoring program indicate the likelihood of public intakes in excess of 1% of those that could result from continuous exposure to the concentration values listed in Appendix B, Table II, Part 20, estimates of the likely resultant exposure to individuals and to population groups, and assumptions upon which estimates are based shall be provided.
- (3) If statistically significant variations of offsite environmental concentrations with time are observed, correlation of these results with effluent release shall be provided.

b. Radioactive Effluent Releases

A statement of the quantities of radioactive effluents released from the plant, with data summarized on a monthly basis following the format of USNRC Regulatory Guide 1.21 and submitted on a semiannual basis.

(1) Gaseous Effluents

(a) Gross Radioactivity Releases

- (i) Total gross radioactivity (in curies), primarily noble and activation gases.
- (ii) Maximum gross radioactivity release rate during any one hour period.
- (iii) Total gross radioactivity (in curies) by nuclide released, based on representative isotopic analyses performed.
- (iv) Percent of technical specification limit.

(b) Iodine Releases

- (i) Total iodine radioactivity (in curies) by nuclide released, based on representative isotopic analyses performed.
- (ii) Percent of technical specification limit for I-131 released.

(c) Particulate Releases

- (i) Total gross radioactivity (β, γ) released (in curies) excluding background radioactivity.

- (ii) Gross alpha radioactivity released (in curies) excluding background radioactivity.
- (iii) Total gross radioactivity (in curies) of nuclides with half-lives greater than eight days.
- (iv) Percent of technical specification limit for particulate radioactivity with half-lives greater than eight days.

(2) Liquid Effluents

- (a) Total gross radioactivity (β, γ) released (in curies) excluding tritium and average concentration released to the unrestricted area.
- (b) The maximum concentration of gross radioactivity (β, γ) released to the unrestricted area (averaged over the period of release).
- (c) Total tritium and total alpha radioactivity (in curies) released and average concentration released to the unrestricted area.
- (d) Total dissolved gas radioactivity (in curies) and average concentration released to the unrestricted area.
- (e) Total volume (in liters) of liquid waste released.
- (f) Total volume (in liters) of dilution water used prior to release from the restricted area.
- (g) Total gross radioactivity (in curies) by nuclide released, based on representative isotopic analyses performed.
- (h) Percent of technical specification limit for total radioactivity.

(3) Solid Waste Shipped

- (a) The total amount of solid waste packaged (in cubic feet).
- (b) The total estimated radioactivity (in curies) involved.
- (c) Disposition including date and destination if shipped offsite.

c. Safety Class I Inservice Inspection

Sixty days after the completion of the first refueling outage.

FOOTNOTES

1. A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.
2. The term "forced reduction in power" is normally defined in the electrical power industry as the occurrence of a component failure or other condition which requires that the load on the unit be reduced for corrective action immediately or up to and including the very next weekend. Note that routine preventive maintenance, surveillance and calibration activities requiring power reductions are not covered by this section.
3. The term "forced outage" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the unit be removed from service for corrective action immediately or up to and including the very next weekend.
4. This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of plant operation, including power levels and periods of operation at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment pertaining to nuclear safety.
- c. Reports of all events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours.
- d. Records of periodic checks, inspections and calibrations required by these Technical Specifications.
- e. Records of nuclear safety related tests or experiments.
- f. Records of radioactive shipments.
- g. Records of changes to operating procedures.
- h. Records of sealed source leak tests and results.
- i. Records of annual physical inventory of all source material of record.
- j. Records of Quality Assurance activities required by the QA Manual except where it is determined that the records should be maintained for a longer period of time.

6.10.2 The following records shall be retained for the duration of the Plant Operating License.

- a. Records of a complete set of as-built drawings for the plant as originally licensed and all print changes showing modifications made to the plant.
- b. Records of new and spent fuel inventory, fuel transfers, and assembly burnup histories.
- c. Records of plant radiation and contamination surveys.
- d. Records of radiation exposure of all plant personnel, and others who enter radiation control areas.
- e. Records of radioactivity in liquid and gaseous wastes released to the environment.
- f. Records of transient or operational cycles for these facility components.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of meetings of the NSRAC and PORC.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 RESPIRATORY PROTECTION PROGRAM

ALLOWANCE

6.12.1 Pursuant to 10 CFR 20.103 (c) (1) and (3), allowance can be made for the use of respiratory protective equipment in conjunction with activities authorized by the operating license for

this plant in determining whether individuals in restricted areas are exposed to concentrations in excess of the limits specified in Appendix B, Table I, Column 1, of 10 CFR 20, subject to the following conditions and limitations:

- a. The limits provided in Section 20.103 (a) and (b) are not exceeded.
- b. If the radioactive material is of such form that intake through the skin or other additional route is likely, individual exposures to radioactive material shall be controlled so that the radioactive content of any critical organ from all routes of intake averaged over 7 consecutive days does not exceed that which would result from inhaling such radioactive material for 40 hours at the pertinent concentration values provided in Appendix B, Table I, Column 1 of 10 CFR 20.
- c. For radioactive materials designated "sub" in the "Isotope" column of Appendix B, Table I, Column 1 of 10 CFR 20, the concentration value specified is based upon exposure to the material as an external radiation source. Individual exposures to these materials shall be accounted for as part of the limitation on individual dose in Section 20.101. These materials shall be subject to applicable process and other engineering controls.

PROTECTION PROGRAM

6.12.2 In all operations in which adequate limitation of the inhalation of radioactive material by the use of process or other engineering

controls is impracticable, the licensee may permit an individual in a restricted area to use respiratory protective equipment to limit the inhalation of airborne radioactive material, provided:

- a. The limits specified in 6.12.1 above are not exceeded.
- b. Respiratory protective equipment is selected and used so that the peak concentrations of airborne radioactive material inhaled by an individual wearing the equipment does not exceed the pertinent concentration values specified in Appendix B, Table I, Column 1, of 10 CFR 20. For the purposes of this subparagraph, the concentration of radioactive material that is inhaled when respirators are worn may be determined by dividing the ambient airborne concentration by the protection factor specified in Table TS 6.12-1 for the respirator protective equipment worn. If the intake of radioactivity is later determined by other measurements to have been different than that initially estimated, the later quantity shall be used in evaluating the exposures.
- c. The licensee advises each respirator user that he may leave the area at any time for relief from respirator use in case of equipment malfunction, physical or psychological discomfort, or any other condition that might cause reduction in the protection afforded the wearer.
- d. The licensee maintains a respiratory protective program adequate to assure that the requirements above are met and incorporates practices for respiratory protection consistent with those recommended by the American National Standards

Institute (NASI-Z88.2-1969). Such a program shall include:

1. Air sampling and other surveys sufficient to identify the hazard, to evaluate individual exposures, and to permit proper selection of respiratory protective equipment.
 2. Written procedures to assure proper selection, supervision, and training of personnel using such protective equipment.
 3. Written procedures to assure the adequate fitting of respirators; and the testing of respiratory protective equipment for operability immediately prior to use.
 4. Written procedures for maintenance to assure full effectiveness of respiratory protective equipment, including issuance, cleaning and decontamination, inspection, repair, and storage.
 5. Written operational and administrative procedures for proper use of respiratory protective equipment including provisions for planned limitations on working times as necessitated by operational conditions.
 6. Bioassays and/or whole body counts of individuals (and other surveys, as appropriate) to evaluate individual exposures and to assess protection actually provided.
- e. The licensee shall use equipment approved by the U.S. Bureau of Mines under its appropriate approval schedule as set forth in Table TS 6.12-1. Equipment not approved under

U.S. Bureau of Mines Approval Schedules may be used only if the licensee has evaluated the equipment and can demonstrate by testing, or on the basis of reliable test information, that the material and performance characteristics of the equipment are at least equal to those afforded by U.S. Bureau of Mines approved equipment of the same type, as specified in Table TS 6.12-1.

- f. Unless otherwise authorized by the Commission, the licensee shall not assign protection factors in excess of those specified in Table TS 6.12-1 in selecting and using respiratory protective equipment.

REVOCATION

6.12.3 The specifications of Section 6.12 shall be revoked upon adoption of proposed changes to 10 CFR 20, Section 20.103, which would make this specification unnecessary.

6.13 HIGH RADIATION AREA

6.13.1 In lieu of the "control device" or "alarm signal" required by 10 CFR Part 20, Section 20.203(c)(2):

- a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr, but less than 1000 mrem/hr, shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.13.1.a above, and in addition locked doors shall be provided to prevent unauthorized entry into these areas. The areas shall be maintained under the administrative control of the Shift Supervisor on duty.

TABLE TS 6.12-1

PROTECTION FACTORS FOR RESPIRATORS

| DESCRIPTION | MODES ^{1/} | PROTECTION FACTORS 2/ | GUIDES TO SELECTION OF EQUIPMENT |
|---|---------------------------------|--|---|
| | | PARTICULATES AND VAPORS AND GASES EXCEPT TRITIUM OXIDE ^{3/} | BUREAU OF MINES/NATIONAL INSTITUTE FOR OCCUPATIONAL SAFETY AND HEALTH APPROVALS |
| I. <u>AIR-PURIFYING RESPIRATORS</u> Facepiece, half-mask <u>4/</u> <u>7/</u> Facepiece, full <u>7/</u> | NP NP | 5 100 | 30 CFR Part 11 Subpart K 30 CFR Part 11 Subpart K |
| II. <u>ATMOSPHERE-SUPPLYING RESPIRATOR</u> 1. <u>Airline Respirator</u> Facepiece, half-mask Facepiece, full Facepiece, full <u>7/</u> Facepiece, full Hood Suit | CF CF D PD CF CF | . 100 1,000 100 1,000 5/ 5/ | 30 CFR Part 11 Subpart J 30 CFR Part 11 Subpart J (6) |
| 2. <u>Self-contained breathing apparatus (SCBA)</u> Facepiece, full <u>7/</u> Facepiece, full Facepiece, full | D PD R | 100 1,000 100 | 30 CFR Part 11 Subpart H 30 CFR Part 11 Subpart H 30 CFR Part 11 Subpart H |
| III. <u>COMBINATION RESPIRATOR</u> Any combination of air-purifying and atmosphere-supplying respirator | | Protection factor for type and mode of operation as listed above | 30 CFR Part 11 §11.63(b) |

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7

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1/, 2/, 3/, 4/, 5/, 6/, 7/, (These notes are on the following pages)

1/ See the following symbols:

CF: continuous flow
 D : demand
 NP: negative pressure (i.e., negative phase during inhalation)
 PD: pressure demand (i.e., always positive pressure)
 R : recirculating (closed-circuit)

2/ (a) For purposes of this specification the protection factor is a measure of the degree of protection afforded by a respirator, defined as the ratio of the concentration of airborne radioactive material outside the respiratory protective equipment to that inside the equipment (usually inside the face-piece) under conditions of use. It is applied to the ambient airborne concentration to estimate the concentration inhaled by the wearer according to the following formula:

$$\text{Concentration Inhaled} = \frac{\text{Ambient Airborne Concentration}}{\text{Protection Factor}}$$

(b) The protection factors apply:

- (i) only for trained individuals wearing properly fitted respirators used and maintained under supervision in a well-planned respiratory protective program.
- (ii) for air-purifying respirators only when high efficiency (above 99.9% removal efficiency by U. S. Bureau of Mines type dioctyl phthalate (DOP) test) particulate filters and/or sorbents appropriate to the hazard are used in atmospheres not deficient in oxygen.
- (iii) for atmosphere-supplying respirators only when supplied with adequate respirable air.

3/ Excluding radioactive contaminants that present an absorption or submersion hazard. For tritium oxide approximately half of the intake occurs by absorption through the skin so that an overall protection factor of not more than approximately 2 is appropriate when atmosphere-supplying respirators are used to protect against tritium oxide. Air-purifying respirators are not recommended for use against tritium oxide. See also footnote 5/, below, concerning supplied-air suits and hoods.

4/ Under chin type only. Not recommended for use where it might be possible for the ambient airborne concentration to reach instantaneous values greater than 50 times the pertinent values in Appendix B, Table I, Column 1 of 10 CFR, Part 20.

5/ Appropriate protection factors must be determined taking account of the design of the suit or hood and its permeability to the containment under conditions of use. No protection factor greater than 1,000 shall be used except as authorized by the Commission.

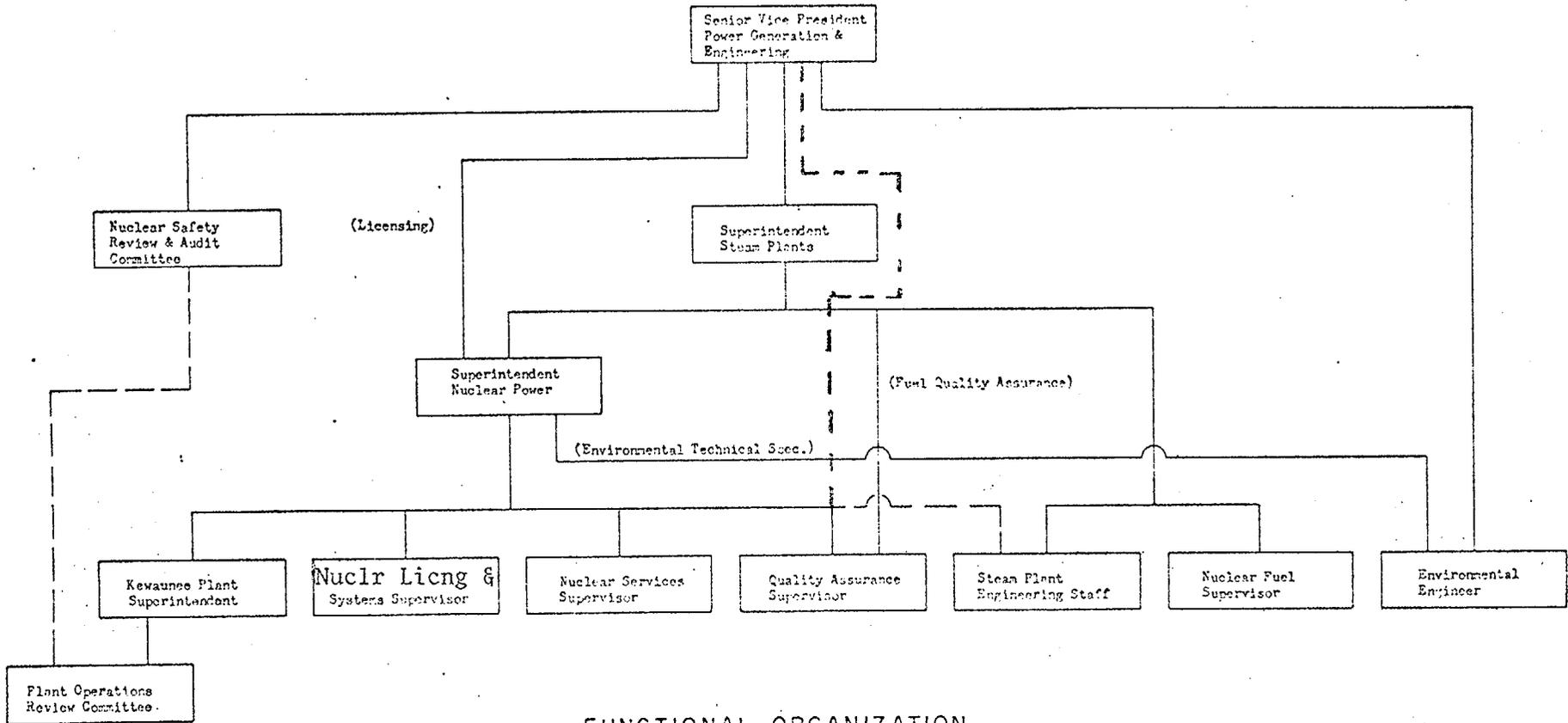
6/ No approval schedules currently available for this equipment. Equipment must be evaluated by testing or on basis of available test information.

7/ Only for shaven faces.

NOTE 1: Protection factors for respirators, as may be approved by the U. S. Bureau of Mines and/or NIOSH according to approval schedules for respirators to protect against airborne radionuclides, may be used to the extent that they do not exceed the protection factors listed in this Table. The protection factors in this Table may not be appropriate to circumstances where chemical or other respiratory hazards exist in addition to radioactive hazards. The selection and use of respirators for such circumstances should take into account approvals of the U. S. Bureau of Mines and/or NIOSH in accordance with its applicable schedules.

NOTE 2: Radioactive contaminants for which the concentration values in Appendix B, Table I of this part are based on internal dose due to inhalation may, in addition, present external exposure hazards at higher concentrations. Under such circumstances, limitations on occupancy may have to be governed by external dose limits.

TS 6-33



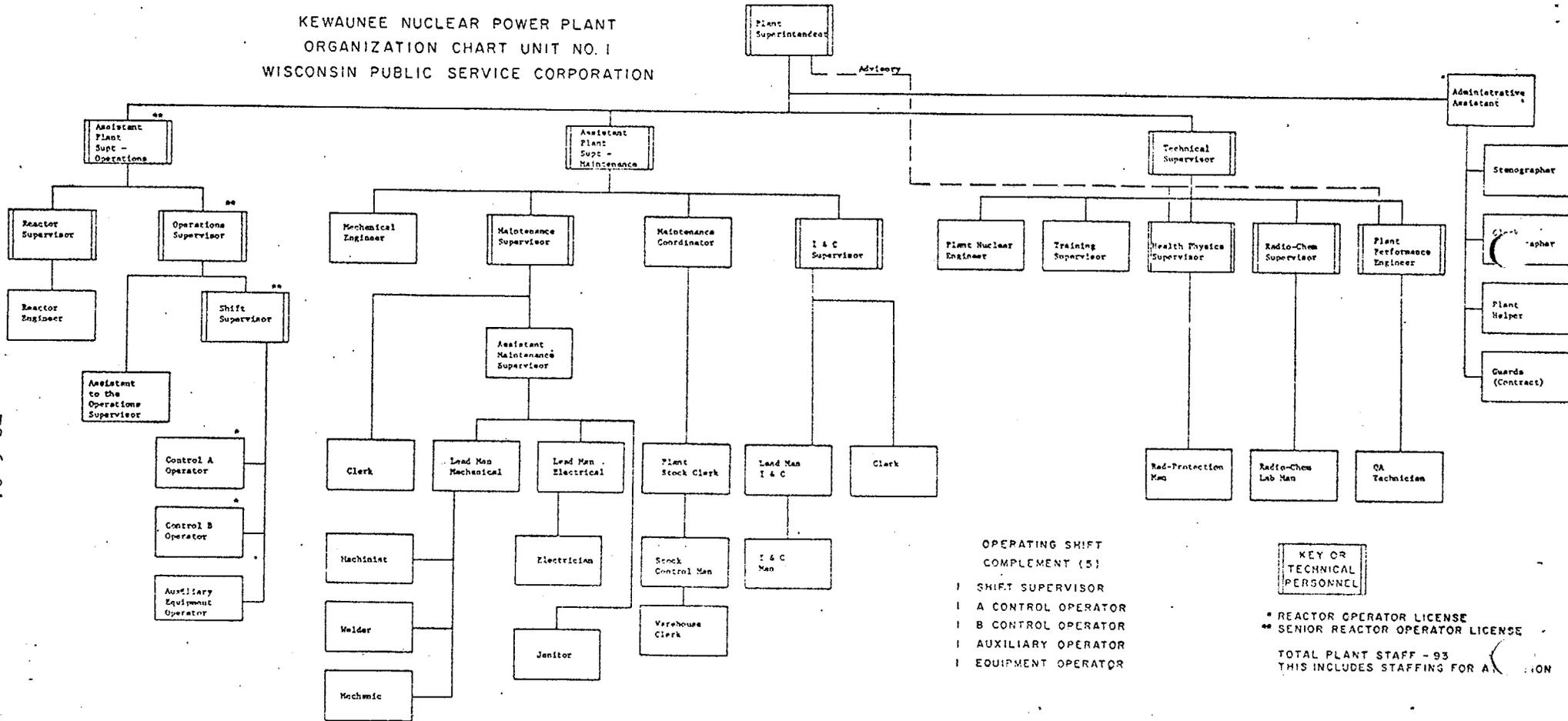
FUNCTIONAL ORGANIZATION
 POWER GENERATION & ENGINEERING DEPARTMENT
 WISCONSIN PUBLIC SERVICE CORPORATION

— Direct Line Responsibility
 - - - Communication/Coordination

FIGURE TS 6.2-1

DEC 18 1975

KEWAUNEE NUCLEAR POWER PLANT
 ORGANIZATION CHART UNIT NO. 1
 WISCONSIN PUBLIC SERVICE CORPORATION



TS 6-34

OPERATING SHIFT
 COMPLEMENT (5)
 I SHIFT SUPERVISOR
 I A CONTROL OPERATOR
 I B CONTROL OPERATOR
 I AUXILIARY OPERATOR
 I EQUIPMENT OPERATOR

KEY OR
 TECHNICAL
 PERSONNEL

* REACTOR OPERATOR LICENSE
 ** SENIOR REACTOR OPERATOR LICENSE
 TOTAL PLANT STAFF - 93
 THIS INCLUDES STAFFING FOR A SECTION

FIGURE TS 6.2-2

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 5 TO FACILITY LICENSE NO. DPR-43

CHANGE NO. 7 TO TECHNICAL SPECIFICATIONS

WISCONSIN PUBLIC SERVICE CORPORATION

WISCONSIN POWER AND LIGHT COMPANY

MADISON GAS AND ELECTRIC COMPANY

KEWAUNEE NUCLEAR POWER PLANT

DOCKET NO. 50-305

Introduction

By letter dated January 15, 1975, Wisconsin Public Service Corporation proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-43, for the Kewaunee Nuclear Power Plant. The proposed changes involve changes to the administrative controls including changes to the reporting requirements.

Discussion

The proposed changes would be administrative in nature and would affect the conduct of operation. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include licensee staffing qualifications and management procedures involved with operating the reactor, reporting requirements, abnormal occurrence definition change, and a respiratory protection program.

Members of the facility staff should meet the requirements set forth in Guide 1.8, "Personnel Selection and Training" which endorses proposed ANSI N18.1, which was subsequently issued as ANSI N18.1-1971. Provisions for independent review of facility operations should be in accord with Guide 1.33, "Quality Assurance Program Requirements" which endorses proposed standard ANS 3.2, which was subsequently issued as ANSI 18.7-1972.

In Section 208 of the Energy Reorganization Act of 1974 "abnormal occurrences" is defined as an unscheduled incident or event which the Commission determines is significant from the standpoint of public health or safety. The term "abnormal occurrence" is reserved for usage by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, enumerates required

| | | | | | | |
|-----------|--|--|--|--|--|--|
| OFFICE ➤ | | | | | | |
| SURNAME ➤ | | | | | | |
| DATE ➤ | | | | | | |

reports consistent with Section 208. The proposed change to required reports identifies the reports required of all licensees not already identified by the regulations and those unique to this facility. The proposal would formalize present reporting and would delete any reports no longer needed for assessment of safety related activities. In addition, a radiation protection program delineates use of respiratory equipment in the event personnel are to be exposed to concentrations in excess of Part 20 concentrations.

Evaluation

The new guidance for reporting operating information does not identify any event as an "abnormal occurrence". The proposed reporting requirements also delete reporting of information no longer required and duplication of reported information. The standardization of required reports and desired format for the information will permit more rapid recognition of potential problems.

Identifying minimum acceptable qualifications for facility personnel should assure capable performance from the facility staff. Other administrative requirements also restated by the specifications assure uniformity and conformance to the desired features in the review, staffing, and procedures. Incorporating the currently accepted respiratory protection program at this time assures that a consistent method of using respiratory equipment is immediately available whenever needed. Similar changes are being approved for all power reactor licensees, so all licensees will have the same requirements presented in a uniform manner.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired regulatory position. These changes were discussed with your staff and have been incorporated into the proposal.

We have concluded that the proposal as modified improves the licensee's program for evaluating plant performance and the reporting of the operating information needed by the Commission to assess safety related activities and is acceptable. The facility staff qualifications and training program conform to Guide 1.8 and therefore are acceptable. The administrative procedures and facility review and audit are consistent with Guide 1.33 and are acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4. The administrative controls are consistent with requirements being incorporated in Technical Specifications for new licensed facilities.

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Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: DEC 18 1975

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-305

WISCONSIN PUBLIC SERVICE CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 5 to Facility Operating License No. DPR-43 issued to Wisconsin Public Service Corporation which revised Technical Specifications for operation of the Kewaunee Nuclear Power Plant located in Kewaunee, Wisconsin. The amendment becomes effective 30 days after the date of issuance.

This amendment revises the administrative controls and reporting requirements of the Technical Specifications for the Kewaunee Nuclear Power Plant.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated January 15, 1975, (2) Amendment No. 5 to License No. DPR-43, with Change No. 7, and (3) the Commission's related Safety Evaluation. All of these items are available for public

inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Kewaunee Public Library, 314 Milwaukee Street, Kewaunee, Wisconsin 54216.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this **DEC 18 1975**

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing

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| OFFICE > | RL:ORB#1 | OELD | RL:ORB#1 | | |
| SURNAME > | JDNeighbors:dc <i>John</i> | See memo dtd 11/3 | RAPurple | | |
| DATE > | 12/18/75 | 12/ /75 | 12/18/75 | | |

attach a copy of this to the release of all our packages on this subject.

| ROUTING AND TRANSMITTAL SLIP | | ACTION | |
|--|----------|---------------------|--|
| 1 TO (Name, office symbol or location) OELD - f/concurrences | INITIALS | CIRCULATE | |
| | DATE | COORDINATION | |
| 2 DLZiemann - f/signatures | INITIALS | FILE | |
| | DATE | INFORMATION | |
| 3 Reba - for final checks | INITIALS | NOTE AND RETURN | |
| | DATE | PER CON - VERSATION | |
| 4 | INITIALS | SEE ME | |
| | DATE | SIGNATURE | |
| REMARKS | | | |
| <p>Attached for your concurrence are five packages (Dresden Station, Quad Cities Station, Cooper, Pilgrim and Calvert Cliffs) of nine from ORB 2 which incorporate <u>standard reporting requirement sections into the Appendix A Technical Specifications</u>. One package, Pilgrim also revises the entire administrative controls section.</p> <p>It is requested that, in the interest of review consistency, these packages (and the 4 future reporting requirements packages) be assigned to one OELD reviewer.</p> <p>Questions may be directed to the PM for the particular case or to Mike Fletcher, coordinator for reporting (Exts. 7403, 7450)</p> | | | |
| <p>Do NOT use this form as a RECORD of approvals, concurrences, disapprovals, clearances, and similar actions</p> | | | |
| FROM (Name, office symbol or location) DLZiemann <i>DZ</i> | | DATE 11-3-75 | |
| | | PHONE 7380 | |

11/3/75 No need for OELD concurrence this done on this subject

RD 11/3