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A&RS (16)

Docket No. 50-305

JAN 1 8 1976

Wisconsin Public Service Corporation
ATTN: Mr. E. W. James
Senior Vice President
Post Office Box 1200
Green Bay, Wisconsin 54305

Gentlemen:

Attached are pages which contained errors when issued with Amendment No. 5 (Change No. 7) to Facility Operating License No. DPR-43 dated December 18, 1975. Please remove pages i, iii, 6-1, 6-5, 6-7, 6-10, 6-14, 6-16, 6-17, 6-18, 6-33, and 6-34 and replace with the attached corrected pages.

Sincerely,

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing

Enclosures:
Corrected Pages

cc w/enclosures:
See next page

OFFICE ➤	RL:ORB#1	RL:ORB#1	RL:ORB#1			
SURNAME ➤	SMSheppard:dg	JNDeighbors	RAPurple			
DATE ➤	1/12/76	1/13/76	1/13/76			

Form AEC-318 (Rev. 9-53) AECM 0240 ☆ U. S. GOVERNMENT PRINTING OFFICE: 1974-526-166

cc: Steven E. Keane, Esquire
Foley, Sammond & Lardner
735 North Water Street
Milwaukee, Wisconsin 53202

Bruce W. Churchill, Esquire
Shaw, Pittman, Potts & Trowbridge
910 - 17th Street, NW
Washington, D. C. 20006

Kewaunee Public Library
314 Milwaukee Street
Kewaunee, Wisconsin 54216

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Title

TECHNICAL SPECIFICATIONS AND BASES

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Superintendent has overall on-site responsibility for plant operation. In the absence of the Plant Superintendent, the succession to this responsibility shall be in the following order:

a. Assistant Superintendent - Maintenance

b. Assistant Superintendent - Operations

c. Operations Supervisor

d. Technical Supervisor

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for plant management and technical support shall be as shown on Figure TS 6.2-1.

FACILITY STAFF

6.2.2 The plant organization shall be as shown on Figure TS 6.2-2 and:

a. Each on-duty shift complement shall consist of at least:

(1) One Shift Supervisor (SRO)
(2) Two licensed Reactor Operators
(3) One Auxiliary Operator
(4) One Equipment Operator

b. In the event that one of the shift members becomes incapacitated due to illness or injury, reactor operations may continue with the reduced complement until his replacement arrives. In all but severe weather conditions, a replacement is required within two hours.

c. Provide immediate notification in the form of draft meeting minutes to the Superintendent - Nuclear Power and the Chairman - Nuclear Safety Review and Audit Committee of disagreement between the PORC and the Plant Superintendent. The Plant Superintendent shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 Minutes shall be kept of all meetings of the PORC and copies shall be sent to the Superintendent - Nuclear Power and the Chairman - Nuclear Safety Review and Audit Committee.

6.5.2 CORPORATE NUCLEAR ENGINEERING STAFF (CNES)

FUNCTION

6.5.2.1 The CNES shall function to provide engineering, technical and quality assurance activities in support of the Kewaunee Plant Staff.

ORGANIZATION

6.5.2.2 The CNES consists of the following groups:

a. Licensing and Nuclear Systems
b. Nuclear Services
c. Steam Plant Engineering
d. Quality Assurance and Control

ACTIVITIES

6.5.2.3

1. Investigate and report all violations of the Technical Specifications, codes, regulations, statutes.

c. Chemistry and Radio-Chemistry

d. Metallurgy

e. Instrumentation and Control

f. Radiological Safety

g. Mechanical and Electrical Engineering

h. Quality Assurance Practices

i. Other appropriate fields as determined by the Committee, to be associated with the unique characteristics of the nuclear power plant.

COMPOSITION

6.5.3.2 The NSRAC shall be composed of, but not necessarily limited to

Members: Superintendent - Nuclear Power
Superintendent - Kewaunee Plant*
Assistant Plant Superintendent - Operations
Mechanical Engineer
Quality Assurance Supervisor
Nuclear Fuels Supervisor
Consultants **

* Non-Voting Member
** Non-Company Affiliated

The Committee membership and its Chairman and Vice Chairman shall be appointed by the Senior Vice-President - Power Supply & Engineering or such person as he shall designate.

ALTERNATES

6.5.3.3 Alternate members shall be appointed by the NSRAC Chairman, upon approval by the Senior Vice President - Power Supply and Engineering, to serve on a temporary basis; however, no more than two alternates shall participate in NSRAC activities at any one time.

f. Any other area of plant operation considered appropriate by the NSRAC or the Senior Vice President - Power Supply & Engineering.

AUTHORITY

6.5.3.9 The NSRAC shall report to and advise the Senior Vice President - Power Supply & Engineering on those areas of responsibility specified in Section 6.5.3.7 and 6.5.3.8.

RECORDS

6.5.3.10 Records of NSRAC activities shall be prepared, approved and distributed as follows:

a. Minutes of each NSRAC meeting forwarded to the Senior Vice President - Power Supply & Engineering within 14 days following each meeting.

b. Reports of reviews required by Section 6.5.3.7 e, f, g and h above, forwarded to the Senior Vice President - Power Supply & Engineering within 14 days following completion of the review.

c. Reports of audits performed by NSRAC shall be forwarded to the Senior Vice President - Power Supply & Engineering and to the management positions responsible for the areas audited within 30 days after completion of the audit.

previously reported information may be involved. References in the annual operating report to previously submitted reports shall be clear.

Each annual operating report shall include:

(1) A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance not covered in item 6.9.1.b.(2)(e) below.

(2) For each outage or forced reduction in power[2/] of over twenty percent of design power level where the reduction extends for greater than four hours:

(a) the proximate cause and the system and major component involved (if the outage or forced reduction in power involved equipment malfunction);

(b) a brief discussion of (or reference to reports of) any reportable occurrences pertaining to the outage or power reduction;

(c) corrective action taken to reduce the probability of recurrence, if appropriate;

(d) operating time lost as a result of the outage or power reduction (for scheduled or forced outages,[3/] use the generator off-line hours; for forced reductions in power, use the approximate duration of operation at reduced power);

(e) a description of major safety-related corrective maintenance performed during the outage or power reduction, including the system and component involved and identification of the critical path activity dictating the length of the outage or power reduction; and

(f) a report of any single release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10% of the allowable annual values.

7

a. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 6.9.2.a(5), 6.9.2.a(6), or 6.9.2.b(1) below.

(2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications. 7

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 6.9.2.b(2) below.

(3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

Note: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

(4) Reactivity anomalies involving disagreement with the predicted value of reactivity balance under steady state conditions greater than or equal to $1.00; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if subcritical, an unplanned reactivity insertion of more than 50¢; or occurrence of any unplanned criticality.

(5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.

(6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For items 6.9.2.a(5) and 6.9.2.a(6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 6.9.2.b(2) and 6.9.2.b(3) below.

(7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.

(8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

(9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

Note: This item is intended to provide for reporting of potentially generic problems.

b.

Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

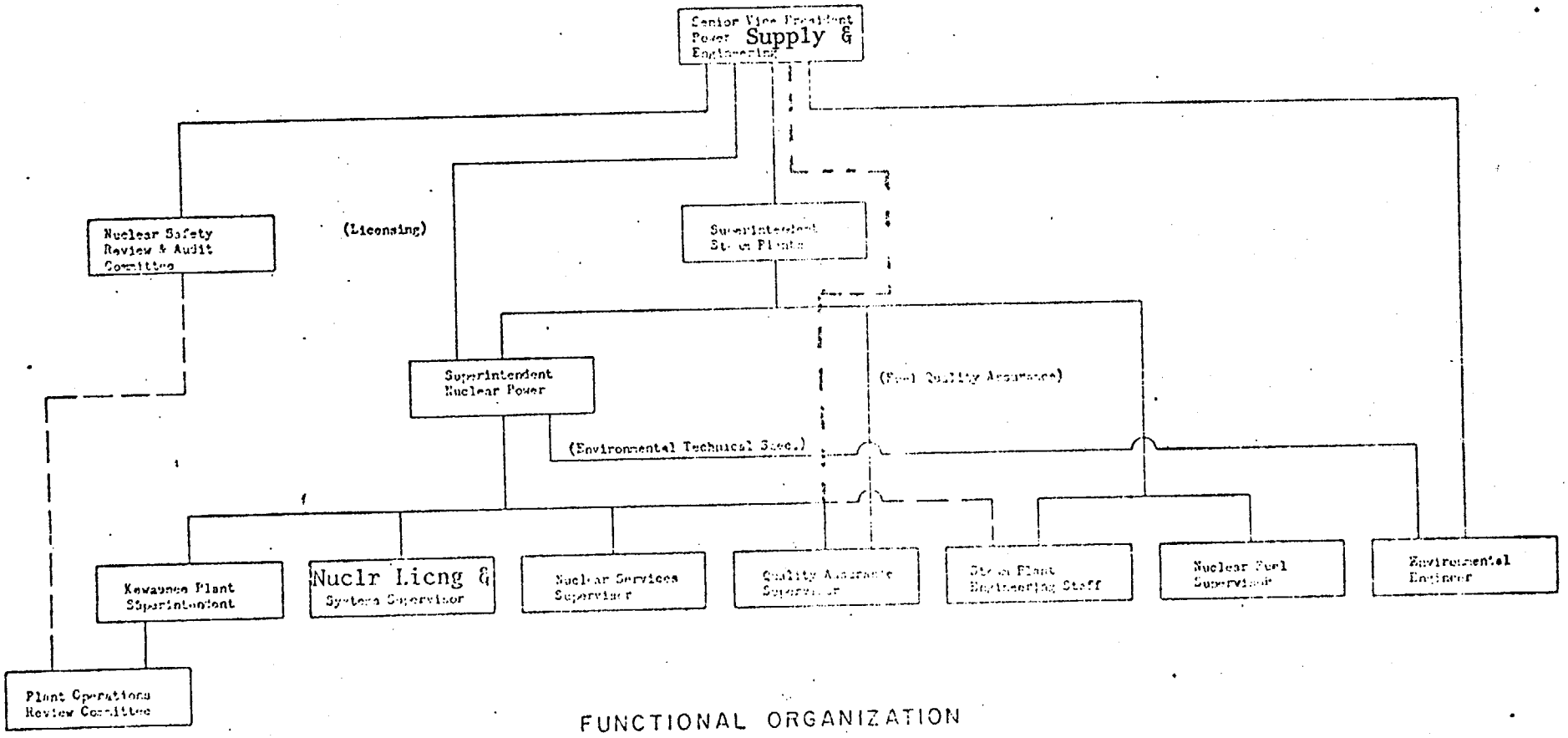
(2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 6.9.2.b(1) and 6.9.2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

(3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems. 7

(4) Abnormal degradation of systems other than those specified in item 6.9.2a(3) above designed to contain radioactive material resulting from the fission process.

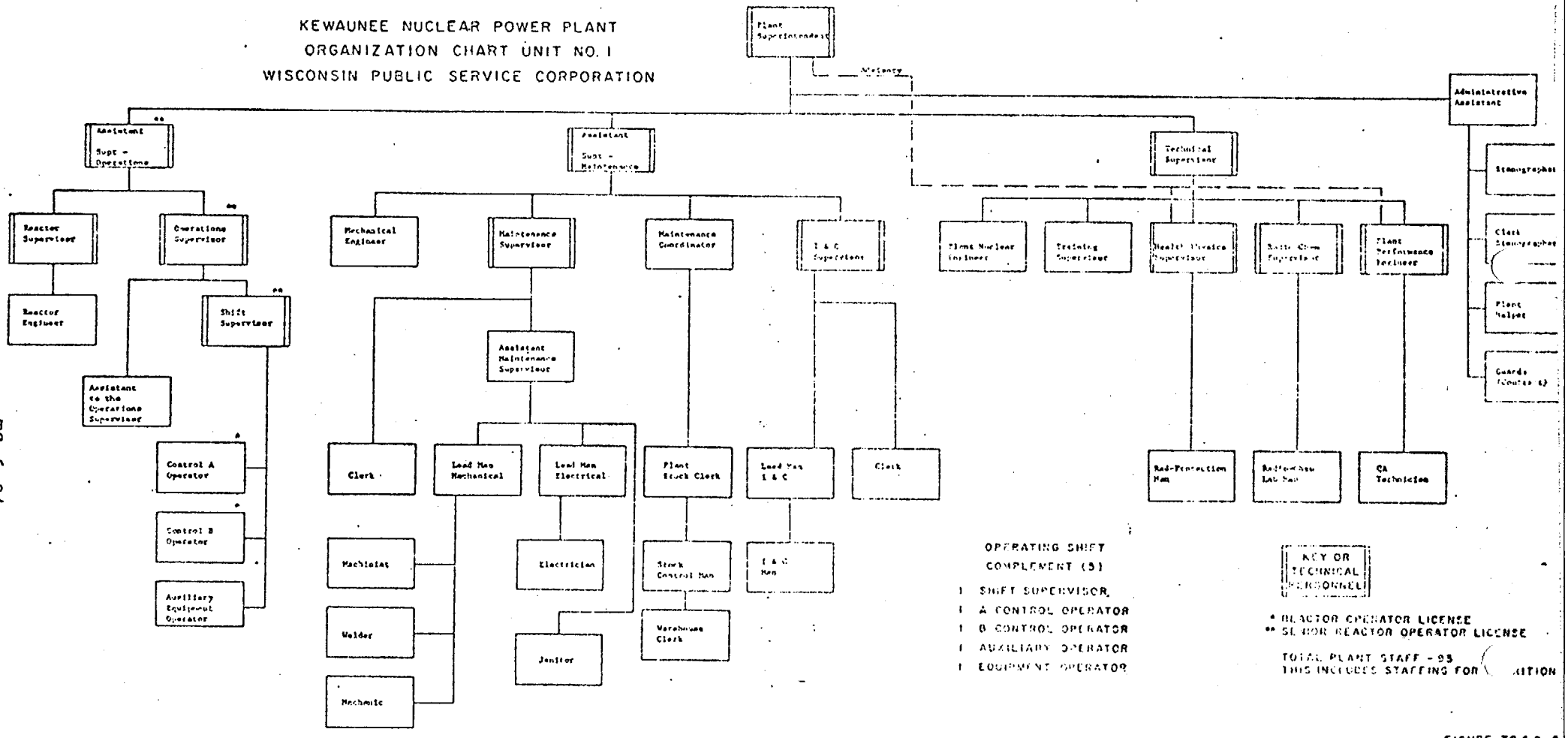
Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.



FUNCTIONAL ORGANIZATION
POWER GENERATION & ENGINEERING DEPARTMENT
WISCONSIN PUBLIC SERVICE CORPORATION

FIGURE TS 6.2-1

KEWAUNEE NUCLEAR POWER PLANT
ORGANIZATION CHART UNIT NO. 1
WISCONSIN PUBLIC SERVICE CORPORATION



OPERATING SHIFT COMPLEMENT (5)

1 SHIFT SUPERVISOR
1 A CONTROL OPERATOR
1 B CONTROL OPERATOR
1 AUXILIARY OPERATOR
1 EQUIPMENT OPERATOR

KEY OR TECHNICAL PERSONNEL

* REACTOR OPERATOR LICENSE
** SENIOR REACTOR OPERATOR LICENSE

TOTAL PLANT STAFF - 93
THIS INCLUDES STAFFING FOR [illegible]ITION

FIGURE TS 6.2-2