Mr. Douglas R. Gipson
Senior Vice President
Nuclear Generation
Detroit Edison Company
6400 North Dixie Highway
Newport, MI 48166

August 19, 1996

SUBJECT: FERMI-2 - CORRECTION TO AMENDMENT NO. 108 RE: IMPLEMENTATION OF 10 CFR PART 50 APPENDIX J OPTION B (TAC NO. M94366)

Dear Mr. Gipson:

On August 8, 1996, the Commission issued Amendment No. 108 to Facility Operating License No. NPF-43 for the Fermi-2 facility. The amendment consisted of changes to the Technical Specifications (TS) in response to your letter dated December 21, 1995 (NRC-95-0133).

Several of the TS overleaf pages were not issued with the amendment and several TS pages had been previously issued with Amendment No. 106 and should not have been issued with Amendment No. 108. A corrected TS change page listing the effective pages and a corrected set of TS pages is enclosed. Please replace the previously issued pages with the enclosed corrected TS pages.

We regret any inconvenience caused by these errors. If you have any questions regarding these changes, please give me a call on (301) 415-2828.

Sincerely,

Original signed by:

Andrew J. Kugler, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-341

Enclosure: Corrected TS pages

cc w/encl: See next page

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Ms. Lynne S. Goodman
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DATED: August 19, 1996

CORRECTION TO AMENDMENT NO. 108 TO FACILITY OPERATING LICENSE NO. NPF-43-FERMI-2

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CORRECTED TS PAGES FOR LICENSE AMENDMENT NO. 108

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following pages of the Appendix "A" Technical Specifications with the attached corrected TS pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

| REMOVE | INSERT |
|---|---|
| 3/4 0-3* | 3/4 0-3* |
| 3/4 0-4 | 3/4 0-4 |
| 3/4 0-5 | 3/4 0-5** |
| 3/4 0-6 | 3/4 0-6* |
| 3/4 6-7 | 3/4 6-7 |
| 3/4 6-8* | 3/4 6-8* |
| 3/4 6-9 | 3/4 6-9*** |
| 3/4 6-10* | 3/4 6-10* |
| 3/4 6-11 | 3/4 6-11*** |
| 3/4 6-12* | 3/4 6-12* |

*Overleaf page provided to maintain document completeness. No changes contained on these pages.

**TS pages previously issued with Amendment No. 106. Should not have been issued with Amendment No. 108.

***These pages are being re-issued as corrected because their overleaf pages were not issued with Amendment No. 108. Although marked "Corrected," there have been no futher changes to these pages.

SURVEILLANCE REQUIREMENTS (Continued)

| ASME Boiler and Pressure Vessel Code and applicable Addenda terminology for inservice inspection and testing activities | Required frequencies for performing inservice inspection and testing activities |
|---|---|
| Weekly | At least once per 7 days |
| Monthly | At least once per 31 days |
| Quarterly or every 3 months | At least once per 92 days |
| Semiannually or every 6 months | At least once per 184 days |
| Every 9 months | At least once per 276 days |
| Yearly or annually | At least once per 366 days |

c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities.

d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.

e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

f. The Inservice Inspection (NDE) Program for piping identified in NRC Generic Letter 88-01, dated January 25, 1988, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping", shall be performed in accordance with the staff positions on schedule, methods and personnel, and sample expansion included in this generic letter.

# TABLE 4.0.2-1

## SURVEILLANCE TEST INTERVALS EXTENDED TO OCTOBER 5, 1996

| SURVEILLANCE REQUIREMENT | DESCRIPTION |
|---|---|
| 4.1.3.1.4.a | Scram discharge vol. vent and drain valve operability |
| 4.3.1.1, Table 4.3.1.1-1, Item 3 | RPS Rx Steam Dome Press High cal. |
| 4.3.1.1, Table 4.3.1.1-1, Item 4 | RPS Rx Low Water Level - Level 3 cal |
| 4.3.1.1, Table 4.3.1.1-1, Item 5 | RPS MSIV Closure cal |
| 4.3.1.1, Table 4.3.1.1-1, Item 6 | RPS Main Steam Line Radiation High cal |
| 4.3.1.1, Table 4.3.1.1-1, Item 7 | RPS Drywell Pressure High cal |
| 4.3.1.3[(a)] | RPS Response Time Test |
| 4.3.2.1, Table 4.3.2.1-1, Item 1.a.1 | Pri Cont Isolation Actuation Rx Water Low Level - Level 3 cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 1.a.2 | Pri Cont Isolation Actuation Rx Water Low Level - Level 2 cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 1.a.3 | Pri Cont Isolation Actuation Rx Water Low Level - Level 1 cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 1.b | Pri Cont Isolation Actuation Drywell Press High cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 1.c.1 | Pri Cont Isolation Actuation Main Steam Line Radiation High cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 1.c.2 | Pri Cont Isolation Actuation Main Steam Line Press Low cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 1.d | Pri Cont Isolation Actuation Main Steam Line Tunnel Temp. High cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 1.e | Pri Cont Isolation Actuation Condenser Press High cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 1.f | Pri Cont Isolation Actuation Turbine Bldg. Area Temp. High cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 2.e | RWCU Isolation Rx Water Low Level - Level 2 channel cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 2.g | RWCU Manual Initiation channel functional test |
| 4.3.2.1, Table 4.3.2.1-1, Item 3.a.1 | RCIC Steam Line Flow High DP channel cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 3.a.2 | RCIC Steam Line Flow High Time Delay cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 4.a.1 | HPCI Steam Line Flow High DP cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 4.a.2 | HPCI Steam Line Flow High Time Delay cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 4.e | HPCI Manual Initiation functional test |
| 4.3.2.1, Table 4.3.2.1-1, Item 5.a | RHR S/D Cooling Rx Water Level Low - Level 3 cal |
| 4.3.2.1, Table 4.3.2.1-1, Item 6.b | Sec. Cont. Isolation - Drywell Press High channel cal |
| 4.3.2.3[(a)] | Isolation Actuation Inst. System Response Time |
| 4.3.3.1, Table 4.3.3.1-1, Item 1.b | CS Drywell Press High Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 2.b | LPCI Drywell Press High Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 2.f | LPCI Riser Differential Pressure High Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 2.g | LPCI Recirc. Pump Differential Pressure High Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 3.b | HPCI Drywell Press High Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 3.f | HPCI Manual Initiation |
| 4.3.3.1, Table 4.3.3.1-1, Item 4.f | ADS RPV Low Level 3 Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 4.i | ADS Manual Inhibit Functional Test |
| 4.3.4, Table 4.3.4-1, Item 2 | RPV Press High Cal (ATWS) |
| 4.3.7.4.1, Table 4.3.7.4.-1, Item 1 | RPV Press Cal - Remote Shutdown |
| 4.3.7.5, Table 4.3.7.5-1, Item 1 | RPV Press Cal Accident Mon. |
| 4.3.7.5, Table 4.3.7.5-1, Item 11 | SRV Position Indic Cal Accident Mon. |
| 4.3.7.5, Table 4.3.7.5-1, Item 12 | CTMT High Range Rad Monitoring Cal Accident Mon. |
| 4.3.7.5, Table 4.3.7.5-1, Item 2.a | RPV Fuel Zone Level Cal Accident Mon |
| 4.3.7.10.c | Loose Part Detection System Cal |
| 4.3.9.1, Table 4.3.9.1-1, Item a | RPV High Water Level 8 Cal FW/Main Turbine Trip |
| 4.3.9.2 | FW/Main Turbine Trip LSFT |
| 4.3.11.1, Table 4.3.11.1-1, Item 7 | Alt S/D system Rx Water Level instrument operability |
| 4.3.11.1, Table 4.3.11.1-1, Item 8 | Alt S/D system Rx Press instrument operability |
| 4.4.2.1.1 | SRV Tail Pipe Pressure Switch Cal |
| 4.4.2.1.2 | SRV lift set point test |
| 4.4.2.2.b | SRV Low Low Set Pressure setpoint Cal and LSFT |
| 4.4.3.1.b | Drywell Sump Flow/Lvl Monitoring Cal |
| 4.4.3.2.2.a | RCS Pressure Isol Valve Leak Test |
| 4.5.1.d.2.a | ADS System Functional Test |
| 4.6.1.4.d.3 | MSIV LCS Press Inst. Cal and DP Calibration |

## TABLE 4.0.2-1

## SURVEILLANCE TEST INTERVALS EXTENDED TO OCTOBER 5, 1996 Cont'd

| SURVEILLANCE REQUIREMENT | DESCRIPTION |
|---|---|
| 4.6.2.1.e | Suppression Chamber operability (visual inspection) |
| 4.6.2.1.h | Suppression Chamber operability DW to torus bypass leak test |
| 4.6.3.4 | Instr. Excess Flow Check operability |
| 4.6.3.5.b | TIP Explosive Squib operability test |
| 4.6.4.1.b.2.a | Torus/Drywell vacuum breaker setpoint operability |
| 4.6.4.1.b.2.b | Torus/Drywell vacuum breaker position indication cal |
| 4.6.4.1.b.2.c | Torus/Drywell vacuum breaker switch opening gap |
| 4.6.4.2.b.2.a | RB/Torus Vacuum Breaker operability (setpoint) |
| 4.6.4.2.b.2.b | RB/Torus Vacuum Breaker operability (visual) |
| 4.6.4.2.b.2.c | RB/Torus Vacuum Breaker position indication operability |
| 4.7.11.4 | Alternative Shutdown Control Circuit Functional Test |
| 4.8.4.2.a.1.a | Primary Containment 4160 Volt Penetration Protective Relay Cal |
| 4.8.4.2.a.1.b | Primary Containment 4160 Volt Penetration Protective Device Integrated Functional Test |

## TABLE NOTATIONS

(a) The surveillance interval of channels within the same trip system required to be tested at least once every N times 18 months, where N is the total number of channels in the trip system, may be based upon the performance of the surveillance during the fifth refueling outage.



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## TABLE 4.0.2-2

## SURVEILLANCE TEST INTERVALS EXTENDED TO END OF REFUELING OUTAGE 5

| SURVEILLANCE REQUIREMENT | DESCRIPTION |
|---|---|
| 4.1.3.5.b.2 | CR Accumulator Integrity Test (Check Valve Leakage) |
| 4.1.5.d.1 | SLCS operability Manual Initiation |
| 4.1.5.d.2 | SLCS pump Relief Valve operability |
| 4.1.5.d.3 | SLCS flow path demonstration |
| 4.3.1.1, Table 4.3.1.1-1, Item 11 | RPS Rx Mode Switch shutdown position functional |
| 4.3.1.2 | RPS Logic System Function Test |
| 4.3.2.1, Table 4.3.2.1-1, Item 1.h | Pri Cont Isolation Actuation Manual Initiation Functional |
| 4.3.2.1, Table 4.3.2.1-1, Item 2.d | RWCU - SLCS initiation channel functional test |
| 4.3.2.1, Table 4.3.2.1-1, Item 5.c | RHR S/D Cooling Rx manual initiation functional test |
| 4.3.2.1, Table 4.3.2.1-1, Item 6.a | Sec. Cont. Isolation - Rx Water Low Level - Level 2 cal |
| 4.3.2.2 | Isolation Actuation Inst. LSFT |
| 4.3.3.1, Table 4.3.3.1-1, Item 1.a | CS RPV Low Level 1 Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 1.c | CS Rx Steam Dome Press Low Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 1.d | CS Manual Initiation |
| 4.3.3.1, Table 4.3.3.1-1, Item 2.a | LPCI RPV Low Level 1 Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 2.c | LPCI Rx Steam Dome Press Low Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 2.d | LPCI Rx Low Level 2 Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 2.e | LPCI Rx Steam Dome Press Low Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 2.h | LPCI Manual Initiation |
| 4.3.3.1, Table 4.3.3.1-1, Item 3.a | HPCI RPV Low Level 2 Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 3.e | HPCI RPV High Level 8 Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 4.a | ADS RPV Low Level 1 Cal |
| 4.3.3.1, Table 4.3.3.1-1, Item 4.h | ADS Drywell Pressure High Bypass Timer |
| 4.3.3.2 | ECCS Logic System Functional Tests |
| 4.3.3.3[a] | ECCS Response Time Tests |
| 4.3.4, Table 4.3.4-1, Item 1 | RPV Low Water Level 2 Cal (ATWS) |
| 4.3.4.2 | ATWS Logic System Functional Test |
| 4.3.5.1, Table 4.3.5.1-1, Item a | RPV Low Level 2 Cal (RCIC) |
| 4.3.5.1, Table 4.3.5.1-1, Item b | RPV High Level 8 Cal (RCIC) |
| 4.3.5.2 | RCIC Logic System Functional Test |
| 4.3.6, Table 4.3.6-1, Item 5.b | Scram Disc. Vol. Trip Bypass Funct. Test |
| 4.3.6, Table 4.3.6-1, Item 7 | Rx Mode Switch Shutdown Pos. Rod Block Funct. Test |
| 4.3.7.4.1, Table 4.3.7.4.-1, Item 2 | RPV Level Cal - Remote Shutdown |
| 4.3.7.5, Table 4.3.7.5-1, Item 16 | CTMT Isolation Valve Position Cal Accident Mon |
| 4.3.7.5, Table 4.3.7.5-1, Item 2.b | RPV Wide Range Level Cal Accident Mon |
| 4.5.1.c.1 | ECCS System Functional Test |
| 4.6.3.2 | Primary Containment Isol Valve operability |
| 4.6.5.2.b | Secondary Containment Isolation Damper Actuation |
| 4.7.1.2.b | ECCW Automatic Actuation |
| 4.7.1.3.b | EESW Automatic Actuation |
| 4.7.1.4.b | EDG Cooling Water Pump Automatic Actuation |
| 4.7.2.1.c.1 | CR Ventilation Filter Penetration |
| 4.7.2.1.c.2 | CR Ventilation Filter Charcoal Laboratory Analysis |
| 4.7.2.1.c.3 | CR Emergency Filtration System Flowrate |
| 4.7.2.1.e.1 | CR Ventilation Filter Pressure Drop |
| 4.7.2.1.e.2 | CR Emergency Filtration System Operational Mode Actuation |
| 4.7.2.1.e.4 | CR Emergency Makeup Inlet Heater Dissipation |
| 4.7.5.e | Snubber Functional Test |
| 4.8.1.1.2.e.1 | EDG Inspection |
| 4.8.1.1.2.e.2 | EDG Load Rejection (1666 kW) |
| 4.8.1.1.2.e.3 | EDG Load Rejection (2850 kW) |
| 4.8.1.1.2.e.4.a | EDG LOP Load Shedding |
| 4.8.1.1.2.e.4.b | EDG LOP Auto Start and Load Sequencing |

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PRIMARY CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each primary containment air lock shall be OPERABLE with:

a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and

b. An overall air lock leakage rate of less than or equal to 0.05 $L_a$ at $P_a$, 56.5 psig.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2*, and 3.

ACTION:

a. With one primary containment air lock door inoperable:

1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.

2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be locked closed at least once per 31 days.

3. Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

b. With the primary containment air lock inoperable, except as a result of an inoperable air lock door, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

*See Special Test Exception 3.10.1.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each primary containment air lock shall be demonstrated OPERABLE:

a. Within 7 days following each closing, except when the air lock is being used for multiple entries, then at least once per 30 days, by verifying seal leakage rate less than or equal to 5 scf per hour when the gap between the door seals is pressurized to $P_a$, 56.5 psig.

b. Prior to establishing PRIMARY CONTAINMENT INTEGRITY when the air lock has been opened during periods when containment integrity was not required. The demonstration shall verify a seal leakage rate less than or equal to 5 scf per hour when the gap between the door seals is pressurized to $P_a$, 56.5 psig, unless the air lock is tested pursuant to Specification 4.6.1.3.c.2.

c. By conducting an overall air lock leakage test at $P_a$, 56.5 psig, and by verifying that the overall air lock leakage rate is within its limit:

1. Prior to initial fuel loading and at 30 months* intervals thereafter,

2. Prior to establishing PRIMARY CONTAINMENT INTEGRITY when the air lock has been opened during periods when containment integrity was not required, if maintenance which could affect the leak tight integrity of the doors has been performed since the last successful test pursuant to Specification 4.6.1.3.c.1.

d. At least once per 6 months by verifying that only one door in each air lock can be opened at a time.**

*The provisions of Specification 4.0.2 are not applicable.

**Except that the inner door need not be opened to verify interlock OPERABILITY when the primary containment is inerted, provided that the inner door interlock is tested within 8 hours after the primary containment has been deinerted.

MSIV LEAKAGE CONTROL SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.1.4 Two independent MSIV leakage control system (LCS) subsystems shall be OPERABLE with each subsystem comprised of a flow path from the associated control air division to the main steam lines.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

With one MSIV leakage control system subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.4 Each MSIV leakage control system subsystem shall be demonstrated OPERABLE:

a. At least once per 31 days by cycling each testable valve except the motor-operated MSIVs through at least one complete cycle of full travel.

b. During each COLD SHUTDOWN, if not performed within the previous 31 days, by cycling each valve including the motor-operated MSIVs not testable during operation through at least one complete cycle of full travel.

c. At least once per 18 months by performance of a functional test of the subsystem throughout its operating sequence, and verifying that each interlock operates as designed and each automatic valve actuates to its correct position.

d. By verifying the pressure control (pressure and $\Delta p$) instrumentation to be OPERABLE by performance of a:

   1. CHANNEL CHECK at least once per 24 hours,

   2. CHANNEL FUNCTIONAL TEST at least once per 92 days, and

   3. CHANNEL CALIBRATION at least once per 18 months.

PRIMARY CONTAINMENT STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.5 The structural integrity of the primary containment shall be maintained at a level consistent with the acceptance criteria in Specification 4.6.1.5.1.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

With the structural integrity of the primary containment not conforming to the above requirements, restore the structural integrity to within the limits within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.5.1 The structural integrity of the exposed accessible interior and exterior surfaces of the primary containment shall be determined during the shutdown for each Type A containment leakage rate test by a visual inspection of those surfaces. This inspection shall be performed prior to the Type A containment leakage rate test and during two other refueling outages before the next Type A test if the interval for the Type A test has been extended to 10 years to verify no apparent changes in appearance or other abnormal degradation.

4.6.1.5.2 Reports Any abnormal degradation of the primary containment structure detected during the above required inspections shall be reported in a Special Report to the Commission pursuant to Specification 6.9.2 within 30 days. This report shall include a description of the condition of the structure, the inspection procedure, the inspection criteria, and the corrective actions taken.

## DRYWELL AND SUPPRESSION CHAMBER INTERNAL PRESSURE

### LIMITING CONDITION FOR OPERATION

3.6.1.6 Drywell and suppression chamber internal pressure shall be maintained between -0.10 and +2.00 psig.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

With the drywell and/or suppression chamber internal pressure outside of the specified limits, restore the internal pressure to within the limit within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

### SURVEILLANCE REQUIREMENTS

4.6.1.6 The drywell and suppression chamber internal pressure shall be determined to be within the limits at least once per 12 hours.