Docket No. 50-341

Mr. Douglas R. Gipson Senior Vice President Nuclear Generation Detroit Edison Company 6400 North Dixie Highway Newport, Michigan 48166

Dear Mr. Gipson:

SUBJECT: FERMI 2 - CORRECTION TO AMENDMENT NO. 102, RESPECTIVELY, RE: FERMI 2 - GENERIC LETTER 91-08 - REMOVAL OF COMPONENT LISTS FROM TECHNICAL SPECIFICATIONS (TAC NO. M89445)

The above Amendment No. 102 to Facility Operating License No. NPF-43 for the Fermi 2 facility, was issued without Technical Specifications 3/4 6-5, 3/4 6-19, 3/4 6-48, 3/4 8-19, and B 3/4 6-5.

We have enclosed a copy of the amendment in its entirety. We regret any inconvenience this error may have caused.

Sincerely,

Original signed by

Timothy G. Colburn, Sr. Project Manager Project Directorate III-1 Division of Reactor Projects - III/IV Office of Nuclear Reactor Regulation

Enclosures:

1. Corrected Amendment No. 102 to NPF-43

2. Safety Evaluation

cc w/enclosures:
See next page

OFFICE	LA:PD31	PM: PD31 Vec	D:PD31 Am
NAME /	&Jamerson #	TColburn:gll	LBMarsh
DATE	08/11 /94	08/11/94	08/11/94

OFFICIAL RECORD COPY FILENAME: G:\WPDOCS\FERMI\FE89445.AMD

9408180047 940811 PDR ADOCK 05000341 PDR PDR

170102 THE FILE GENTER CONV

Qfo!

Docket No. 50-341

Mr. Douglas R. Gipson Senior Vice President Nuclear Generation Detroit Edison Company 6400 North Dixie Highway Newport, Michigan 48166

Dear Mr. Gipson:

SUBJECT: FERMI-2 - ISSUANCE OF AMENDMENT RE: FERMI 2 - GENERIC LETTER (GL) 91-08 - REMOVAL OF COMPONENT LISTS FROM TECHNICAL SPECIFICATIONS (TAC NO. M89445)

The Commission has issued the enclosed Amendment No. 102 to Facility Operating License No. NPF-43 for the Fermi-2 facility. The amendment consists of changes to the Technical Specifications (TS) in response to your letter dated May 10, 1994.

The amendment revises the Fermi-2 TS to remove the list of primary containment isolation valves (Table 3.6.3-1) and the motor-operated valves thermal overload protection list (Table 3.8.4.3-1) from TS. The amendment also modifies TS that reference these lists to reflect their removal. The lists will be incorporated into plant procedures which are subject to the administrative controls of TS 6.5.3 and 6.8 in accordance with the guidance of GL 91-08.

A copy of our Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly <u>Federal</u> <u>Register</u> notice.

Sincerely,

Original signed by

Timothy G. Colburn, Sr. Project Manager Project Directorate III-1 Division of Reactor Projects - III/IV Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 102 to NPF-43

2. Safety Evaluation

cc w/enclosures:

See next page

OFFICE	LA:PD31	PM:PD31 The	OGC SHO	D:PD31V
NAME	CJamasen	TColburn:gll	EHOLLEN	LBMarsh
DATE	07/	07/ 4/94	07/14/94	07/5 ⁹ /94
OFFICIAL RECORD COPY FILENAME: G:\WPDOCS\FERMI\FE89445.AMD				

Mr. Douglas R. Gipson Detroit Edison Company

cc:

John Flynn, Esquire Senior Attorney Detroit Edison Company 2000 Second Avenue Detroit, Michigan 48226

Nuclear Facilities and Environmental Monitoring Section Office Division of Radiological Health Department of Public Health 3423 N. Logan Street P. O. Box 30195 Lansing, Michigan 48909

U.S. Nuclear Regulatory Commission Resident Inspector Office 6450 W. Dixie Highway Newport, Michigan 48166

Monroe County Office of Civil Preparedness 963 South Raisinville Monroe, Michigan 48161

Regional Administrator, Region III U.S. Nuclear Regulatory Commission 801 Warrenville Road Lisle, Illinois 60532-4351

Ms. Lynne S. Goodman
Director - Nuclear Licensing
Detroit Edison Company
Fermi-2
6400 North Dixie Highway
Newport, Michigan 48166



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

DETROIT EDISON COMPANY

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 102 License No. NPF-43

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Detroit Edison Company (the licensee) dated May 10, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 102, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance with full implementation within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Marsh Director

Ledyard B. Marsh, Director

Project Directorate III-1

Division of Reactor Projects - III/IV Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: August 1, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 102

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

DEMOVE	<u>INSERT</u>
REMOVE	xiv
xiv	xxiv
xxiv	XXV
XXV	1-5
1-5	3/4 3-11
3/4 3-11	3/4 3-12
3/4 3-12	3/4 3-12
3/4 3-13	3/4 3-13
3/4 3-14	
3/4 3-14a	3/4 3-14a
3/4 6-1	3/4 6-1
3/4 6-2	3/4 6-2
3/4 6-3	3/4 6-3
3/4 6-5	3/4 6-5*
3/4 6-6	3/4 6-6
3/4 6-19	3/4 6-19*
3/4 6-20	3/4 6-20
3/4 6-21	3/4 6-21
3/4 6-22 through	3/4 6-22
3/4 6-48	3/4 6-48*
3/4 8-19	3/4 8-19*
3/4 8-20	3/4 8-20
3/4 8-21 through	3/4 8-21
3/4 8-24	
B 3/4 6-5	B 3/4 6-5*
B 3/4 6-6	B 3/4 6-6
, 	B 3/4 6-6a

^{*}Overleaf page provided to maintain document completeness. No changes contained on these pages.

INDEX

BASES		
<u>SECTION</u>		PAGE
3/4.4 REACTOR C	OOLANT SYSTEM (Continued)	
3/4.4.10	CORE THERMAL HYDRAULIC STABILITY	B 3/4 4-8
3/4.5 EMERGENCY	CORE COOLING SYSTEMS	
3/4.5.1/2	ECCS - OPERATING and SHUTDOWN	B 3/4 5-1
3/4.5.3	SUPPRESSION CHAMBER	B 3/4 5-2
3/4.6 CONTAINME	NT SYSTEMS	
3/4.6.1	PRIMARY CONTAINMENT	
	Primary Containment Integrity	B 3/4 6-1
	Primary Containment Leakage	B 3/4 6-1
	Primary Containment Air Locks	B 3/4 6-1a
	MSIV Leakage Control System	B 3/4 6-2
	Primary Containment Structural Integrity	B 3/4 6-2
	Drywell and Suppression Chamber Internal Pressure	B 3/4 6-2
	Drywell Average Air Temperature	B 3/4 6-2
	Drywell and Suppression Chamber Purge System	B 3/4 6-2
3/4.6.2	DEPRESSURIZATION SYSTEMS	B 3/4 6-3
3/4.6.3	PRIMARY CONTAINMENT ISOLATION VALVES	B 3/4 6-6
3/4.5.4	VACUUM RELIEF	B 3/4 6-6
3/4.6.5	SECONDARY CONTAINMENT	B 3/4 6-6a
3/4.6.6	PRIMARY CONTAINMENT ATMOSPHERE CONTROL	В 3/4 6-7

INDEX

LIST OF TABL	ES (Continued)	
TABLE		PAGE
3.3.7.9-1	DELETED	3/4 3-68
3.3.7.12-1	EXPLOSIVE GAS MONITORING INSTRUMENTATION	3/4 3-77
4.3.7.12-1 3.3.9-1	EXPLOSIVE GAS MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTSFEEDWATER/MAIN TURBINE TRIP SYSTEM ACTUATION	3/4 3-81
3.3.9-2	INSTRUMENTATIONFEEDWATER/MAIN TURBINE TRIP SYSTEM ACTUATION	3/4 3-87 3/4 3-88
4.3.9.1-1 3.3.11-1	INSTRUMENTATION SETPOINTS FEEDWATER/MAIN TURBINE TRIP SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS APPENDIX R ALTERNATIVE SHUTDOWN INSTRUMENTATION	3/4 3-89 3/4 3-91
4.3.11.1-1	APPENDIX R ALTERNATIVE SHUTDOWN INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-92
3.4.3.2-1	REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES REACTOR COOLANT SYSTEM INTERFACE VALVES LEAKAGE	3/4 4-12
3.4.3.2-2 3.4.4-1	PRESSURE MONITORS	3/4 4-12 3/4 4-15
4.4.5-1 4.4.6.1.3-1	ANALYSIS PROGRAM	3/4 4-18
	WITHDRAWAL SCHEDULE	3/4 4-22
4.6.1.1-1 3.6.3-1	PRIMARY CONTAINMENT ISOLATION VALVES/FLANGES LOCATED IN LOCKED HIGH RADIATION AREAS DELETED	3/4 6-1b 3/4 6-22
3.6.5.2-1	SECONDARY CONTAINMENT VENTILATION SYSTEM AUTOMATIC ISOLATION DAMPERS	3/4 6-53
4.7.2.1-1 3.7.3-1 4.7.5-1 3.7.7.5-1 3.7.7.6-1	CONTROL ROOM EMERGENCY FILTRATION SYSTEM DUCT LEAK TESTING SURVEILLANCE REQUIREMENTS SURVEY POINTS FOR SHORE BARRIER SNUBBER VISUAL INSPECTION INTERVAL DELETED DELETED	3/4 7-10b 3/4 7-12 3/4 7-20a 3/4 7-32 3/4 7-37
4.8.1.1.2-1	DIESEL GENERATOR TEST SCHEDULE	3/4 8-8

xxiv Amendment No. 49,89,82,82,84,88

INDEX

LIST OF TABLE	S (Continued)	
TABLE		<u>PAGE</u>
4.8.2.1-1	BATTERY SURVEILLANCE REQUIREMENTS	3/4 8-12
3.8.4.2-1	PRIMARY CONTAINMENT PENETRATION CONDUCTOR OVERCURRENT PROTECTIVE DEVICES	3/4 8-19
3.8.4.3-1	DELETED	3/4 8-21
3.8.4.5-1	STANDBY LIQUID CONTROL SYSTEM ASSOCIATED ISOLATION DEVICES 480 V MOTOR CONTROL CENTERS	3/4 8-27
5.7.1-1	COMPONENT CYCLIC OR TRANSIENT LIMITS	5-7
6.2.2-1	MINIMUM SHIFT CREW COMPOSITION	6-5

DEFINITIONS

- 2. Closed by at least one manual valve, blank flange, or deactivated automatic valve secured in its closed position, except for valves that are open under administrative control as permitted by Specification 3.6.3.
- b. All primary containment equipment hatches are closed and sealed.
- c. Each primary containment air lock is in compliance with the requirements of Specification 3.6.1.3.
- d. The primary containment leakage rates are within the limits of Specification 3.6.1.2.
- e. The suppression chamber is in compliance with the requirement of Specification 3.6.2.1.
- f. The sealing mechanism associated with each primary containment penetration, e.g., welds, bellows, or O-rings, is OPERABLE.
- g. The suppression chamber to reactor building vacuum breakers are in compliance with Specification 3.6.4.2.

THE PROCESS CONTROL PROGRAM

1.30 The PROCESS CONTROL PROGRAM (PCP) shall contain the current formulas, sampling, analyses, test, and determinations to be made to ensure that processing and packaging of solid radioactive wastes based on demonstrated processing of actual or simulated wet solid wastes will be accomplished in such a way as to assure compliance with 10 CFR Parts 20, 61, and 71, State regulations, burial ground requirements, and other requirements governing the disposal of solid radioactive waste.

PURGE - PURGING

1.31 PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

RATED THERMAL POWER

1.32 RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 3430 MWT.

TABLE 3.3.2-1

ISOLATION ACTUATION INSTRUMENTATION

TRIF	<u>FUNCT</u>	<u>ION</u>	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM ^(a)	APPLICABLE OPERATIONAL CONDITION	ACTION
1.	PRIMARY CONTAINMENT ISOLATION				
	a.	Reactor Vessel Low Water Level 1) Level 3 ## 2) Level 2 (d) 3) Level 1	2 2 2	1, 2, 3 1, 2, 3 1, 2, 3	20 20 20
	b.	Drywell Pressure - High ##	2	1, 2, 3	20
	c.	Main Steam Line			
		 Radiation - High## Pressure - Low Flow - High 	2 2 2	1, 2, 3 1 1, 2, 3	21 22 21
	d.	Main Steam Line Tunnel Temperature - High	2(c)	1, 2, 3	21
	e.	Condenser Pressure - High	2	1, 2**, 3**	21
	f.	Turbine Bldg. Area Temperature - High	2	1, 2, 3	21
	g.	Deleted			
	h.	Manual Initiation	1/valve	1, 2, 3	26

DATED: <u>August 1, 1994</u>

AMENDMENT NO. 102 TO FACILITY OPERATING LICENSE NO. NPF-43-FERMI-2

Docket File
NRC & Lecal PDRs
PDITI-1 Reading
J. Roe
J. Zwolinski
L. Marsh
C. Jamerson
T. Colburn
OGC-WF
D. Hagan
G. Hill, T-5C2
C. Grimes, O-11/F/23
W. Reckley
ACRS (10)
OPA
OC/LFDCB, T-4A43
L. Miller, R-III
SEDB

cc: Plant Service list