

May 28, 1998

Mr. Douglas R. Gipson
Senior Vice President
Nuclear Generation
Detroit Edison Company
6400 North Dixie Highway
Newport, MI 48166

SUBJECT: FERMI 2 - ISSUANCE OF AMENDMENT RE: EXTENSION OF THE LEAK RATE TEST SURVEILLANCE INTERVAL FOR THE PRESSURE ISOLATION VALVES (TAC NO. MA0719)

Dear Mr. Gipson:

The Commission has issued the enclosed Amendment No. 118 to Facility Operating License No. NPF-43 for the Fermi 2 facility. The amendment consists of changes to the Technical Specifications in response to your application dated January 28, 1998 (NRC-98-0006), as supplemented on March 10, 1998 (NRC-98-0036).

The amendment revises technical specification surveillance requirement 4.4.3.2.2.a for the leak rate test of the pressure isolation valves, extending it from the current 18-month interval to a 24-month interval.

A copy of our Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

ORIGINAL SIGNED BY

Andrew J. Kugler, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-341

Enclosures: 1. Amendment No. 118 to NPF-43
2. Safety Evaluation

cc w/encl: See next page

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DATE	05/ 7 /98		05/ 5 /98		05/ 7 /98	5/19/98	5/28/98

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 28, 1998

Mr. Douglas R. Gipson
Senior Vice President
Nuclear Generation
Detroit Edison Company
6400 North Dixie Highway
Newport, MI 48166

SUBJECT: FERMI 2 - ISSUANCE OF AMENDMENT RE: EXTENSION OF THE LEAK
RATE TEST SURVEILLANCE INTERVAL FOR THE PRESSURE ISOLATION
VALVES (TAC NO. MA0719)

Dear Mr. Gipson:

The Commission has issued the enclosed Amendment No. 118 to Facility Operating License No. NPF-43 for the Fermi 2 facility. The amendment consists of changes to the Technical Specifications in response to your application dated January 28, 1998 (NRC-98-0006), as supplemented on March 10, 1998 (NRC-98-0036).

The amendment revises technical specification surveillance requirement 4.4.3.2.2.a for the leak rate test of the pressure isolation valves, extending it from the current 18-month interval to a 24-month interval.

A copy of our Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Andrew J. Kugler".

Andrew J. Kugler, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-341

Enclosures: 1. Amendment No. 118 to NPF-43
2. Safety Evaluation

cc w/encl: See next page

Mr. Douglas R. Gipson
Detroit Edison Company

Fermi 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DETROIT EDISON COMPANY

DOCKET NO. 50-341

FERMI 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Detroit Edison Company (the licensee) dated January 28, 1998, as supplemented on March 10, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 118 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance with full implementation within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Andrew J. Kugler, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: May 28, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 118

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 4-11a
3/4 4-11b

INSERT

3/4 4-11a
3/4 4-11b*

*Overleaf page provided to maintain document completeness. No changes contained on this page.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

4.4.3.2.2 Each reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1 shall be demonstrated OPERABLE by leak testing pursuant to Specification 4.0.5 and verifying the leakage of each valve to be within the specified limit:

- a. At least once per 24 months, and
- b. Prior to returning the valve to service following maintenance, repair or replacement work on the valve which could affect its leakage rate.

The provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 3.

4.4.3.2.3 The high/low pressure interface valve leakage pressure monitors shall be demonstrated OPERABLE with alarm setpoints per Table 3.4.3.2-2 by performance of a:

- a. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
- b. CHANNEL CALIBRATION at least once per 18 months.

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FERMI - UNIT 2

3/4 4-11b

Amendment No. 89
MAY 26 1993



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. NPF-43

DETROIT EDISON COMPANY

FERMI 2

DOCKET NO. 50-341

1.0 INTRODUCTION

By letter dated January 28, 1998 (NRC-98-0006), as supplemented on March 10, 1998 (NRC-98-0036), the Detroit Edison Company (DECo or the licensee) requested an amendment to the Technical Specifications (TS) appended to Facility Operating License No. NPF-43 for Fermi 2. The proposed amendment would revise TS surveillance requirement 4.4.3.2.2.a for the leak rate test of the pressure isolation valves (PIVs), extending it from the current 18-month interval to a 24-month interval. This change is consistent with the requirements of the applicable version of the American Society of Mechanical Engineers (ASME) Code, Section XI. The March 10, 1998, supplement requested a change in the implementation period. This information was within the scope of the original *Federal Register* notice and did not change the staff's initial proposed no significant hazards considerations determination.

Current TS Section 4.4.3.2.2 states "Each reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1 shall be demonstrated OPERABLE by leak testing pursuant to Specification 4.0.5 and verifying the leakage of each valve to be within the specified limit: a. At least once per 18 months." This section would be revised by extending the surveillance frequency (shown underlined) from, "At least once per 18 months" to "At least once per 24 months." As described above, this is a change only to the surveillance frequency. It does not cause a change to any surveillance requirement nor does it change the way in which the surveillance is performed. Furthermore, the proposed change does not involve any physical changes to plant systems or components.

PIVs perform an isolation function between the high pressure and low pressure portions of systems connected to the reactor coolant system (RCS). There are several safety systems connected to the reactor coolant pressure boundary that have design pressures below the rated reactor system pressure. The low pressure portions of these systems are protected from overpressurization by the PIVs placed in series to form the interface between the high pressure and low pressure portions of these systems. The low pressure portions of these systems are also equipped with relief valves and alarms in the control room that alert the operator of PIV leakage. Leak tight integrity of the PIVs should be maintained at acceptable values to prevent overpressurization of the low pressure portions of systems connected to the RCS.

2.0 EVALUATION

Fermi 2 TS Section 4.0.5 specifies that inservice testing (IST) of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance with ASME Code Section XI as required by 10 CFR 50.55a. The current IST program for Fermi 2 nuclear plant was developed to the requirements of the 1980 Edition of the ASME Code, Section XI, up to and including the Winter 1980 Addenda. Paragraph IWV-3422 of this ASME Code Section XI requires that a seat leakage test be conducted at least once every 2 years. However, an earlier version of Section XI required these tests to be conducted at the same frequency as scheduled refueling outages, but not less than once every 2 years. The PIV leak tests are normally performed during extended plant shutdown conditions, and the 18-month refueling outage interval for Fermi 2 plant was therefore specified in TS Section 4.4.3.2.2.a. Due to unexpected shutdowns and extended outages, certain refueling intervals might extend beyond 18 months. In such cases, performance of the 18-month PIV leak tests would require a plant shutdown. If the leak tight integrity of the PIVs has been maintained at an acceptable level and has not exhibited increasing seat leakage from previous tests, shutting down the plant to perform valve leak tests would be a hardship to the licensee without compensating increase in the level of quality and safety.

Extending the test frequency from every 18 months to every 24 months would still meet the ASME Code Section XI requirements, and thus, would provide an adequate level of verification of the leak tight integrity for the affected PIVs. Furthermore, the 24-month testing frequency is consistent with the guidance provided in Table 3.2 of NUREG-1482, "Guidelines for Inservice Testing at Nuclear Power Plants," which recommends that leak tests for PIVs (Category "A" and "A/C" valves) be performed at a frequency of once every 2 years. Accordingly, the staff finds the proposed TS changes to be acceptable.

Based on the above evaluation, the staff concludes that the proposed TS change meets the requirements of ASME Code, Section XI, and is consistent with the guidance delineated in Table 3.2 of NUREG-1482. Performing leak tests in accordance with the requirements of ASME Code, Section XI, provides assurance of operational readiness for the PIVs and is, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Michigan State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (63 FR 9598). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR

51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: John Huang

Date: May 28, 1998

DATED: May 28, 1998

AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. NPF-43 - FERMI 2

Docket File (50-341)

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