

October 22, 1990

Docket No. 50-341

Mr. William S. Orser  
Senior Vice President - Nuclear  
Operations  
Detroit Edison Company  
6400 North Dixie Highway  
Newport, Michigan 48166

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DHagan

Dear Mr. Orser:

SUBJECT: AMENDMENT NO. 61 TO FACILITY OPERATING LICENSE NO. NPF-43:  
(TAC NO. 77512)

The Commission has issued the enclosed Amendment No. 61 to Facility Operating License No. NPF-43 for the Fermi-2 facility. This amendment consists of changes to the Plant Technical Specifications (TS) in response to your letter dated December 7, 1989 (NRC-89-0264). Your letter indicated that it also supersedes your December 22, 1988 application.

The amendment revises the Technical Specification by removing the existing footnote on the Limiting Condition for Operation to Section 3.6.4.1 of the TS which deals with Suppression Chamber Drywell Vacuum Breakers.

A copy of the Safety Evaluation supporting this amendment is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

John F. Stang, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 61 to NPF-43
- 2. Safety Evaluation

cc w/enclosures:  
See next page

\*See previous concurrence

LA/PD31:DRSP  
\*SMeador  
9/14/90

PM/PD31:DRSP  
\*JStang  
9/15/90

PD31:DRSP  
\*APerson  
9/19/90

NRR/SPLB  
\*CMCCRACKEN  
9/20/90

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10/1/90

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

October 22, 1990

Docket No. 50-341

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Senior Vice President - Nuclear  
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Sincerely,

A handwritten signature in cursive script that reads "John F. Stang".

John F. Stang, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 61 to NPF-43
2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. William Orser  
Detroit Edison Company

Fermi-2 Facility

cc:

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Supervisor - Licensing  
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Newport, Michigan 48166



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DETROIT EDISON COMPANY

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 61  
License No. NPF-43

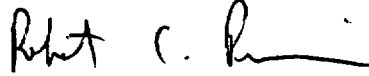
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Detroit Edison Company (the licensee) dated December 7, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 61, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert C. Pierson, Director  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 22, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 61

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 6-47\*

3/4 6-48

INSERT

3/4 6-47\*

3/4 6-48

-----  
\*Overleaf page provided to maintain document completeness. No changes contained on this page.

TABLE 3.3.3-1 (continued)

PRIMARY CONTAINMENT ISOLATION VALVES

TABLE NOTATIONS (Continued)

- (k) Will automatically close when a) RCIC Turbine Steam Stop Valve E51-F045 closes or b) RCIC Turbine Governor Trip and Throttle Valve E51-F059 closes.
- (l) Will automatically close as a result of the conditions listed in Note (k) above, as well as when RCIC flow is greater than 130 gpm.
- (m) These valves are actuated by remote manual key-locked switches and will cut the TIP cable and seal off the TIP guide tube when actuated. These valves are squib-fired.
- (n) May be closed remotely as a secondary actuation mode to reverse flow.
- (o) Valves realign automatically on a reactor scram signal.
- (p) Thermal relief valves.
- (q) Locked closed.
- (r) Not subject to Type C leakage tests.



## CONTAINMENT SYSTEMS

### 3/4.6.4 VACUUM RELIEF

#### SUPPRESSION CHAMBER - DRYWELL VACUUM BREAKERS

##### LIMITING CONDITION FOR OPERATION

---

3.6.4.1 All suppression chamber - drywell vacuum breakers shall be closed and at least 10 vacuum breakers shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

##### ACTION:

- a. With one of the above required vacuum breakers inoperable for opening but known to be closed, restore the inoperable vacuum breaker to OPERABLE status within 72 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With one or more suppression chamber - drywell vacuum breakers open, close the open vacuum breaker(s) within 2 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. With one of the closed position indicators of one or more suppression chamber - drywell vacuum breaker(s) inoperable, verify the associated vacuum breaker and all other breakers to be closed within 2 hours and:
  1. Verify the vacuum breaker(s) with the inoperable position indicator to be closed by demonstrating the other indicator to be OPERABLE within 2 hours and at least once per 14 days thereafter, or
  2. Verify the vacuum breaker(s) with the inoperable position indicator to be closed by conducting a test which demonstrates that the drywell-to-suppression chamber is maintained at greater than or equal to 0.5 psi for one hour without makeup within 24 hours and at least once per 14 day thereafter.

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

- d. With one of the closed position indicators of one or more suppression chamber - drywell vacuum breaker(s) indicating open and the redundant closed position indicator indicating closed after a suppression chamber - drywell vacuum breaker opening as a result of a steam release, within 24 hours, cycle the applicable valve(s) to determine which of the redundant indicators is OPERABLE.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 61 TO FACILITY OPERATING LICENSE NO. NPF-43

DETROIT EDISON COMPANY

FERMI-2

DOCKET NO. 50-341

1.0 INTRODUCTION

By letter dated December 7, 1989, The Detroit Edison Company (DECo or the licensee) requested amendment to the Technical Specifications (TS) appended to Facility Operating License No. NPF-43 for Fermi-2. The proposed amendment would revise the TS to remove an existing footnote associated with the Limiting Condition for Operation (LCO) of Section 3.6.4.1. This section of the TS specifies the operation of the Suppression Chamber Drywell Vacuum Breakers. The application supersedes a December 22, 1988 application dealing with the same subject.

2.0 EVALUATION

Vacuum breakers between the suppression chamber atmosphere and drywell provide vacuum relief to the drywell after a postulated Loss-of-Coolant Accident (LOCA). The TS requires that all suppression chamber-to-drywell vacuum breakers be closed (normal operating position) when manually opened for inerting the containment. The exception is called out in a footnote to the LCO in Section 3.6.4.1 of the TS which also requires that all vacuum breakers be closed within two hours after the inerting of the primary containment has been completed. The bases for the footnote in the current TS is to allow the opening of the vacuum breakers during containment inerting to facilitate nitrogen flow from the suppression chamber atmosphere through the vent headers.

The proposed change to the TS would delete the subject footnote. Manually opening of the vacuum breakers bypasses the pressure suppression feature of the BWR Mark I Primary Containment. The containment was not designed to contain the effects of a LOCA if these vacuum breakers are open prior to the start of a LOCA. The proposed change prevents manually opening the vacuum breakers by deleting the subject footnote.

The proposed change to the TS is more restrictive on plant operation because manual operation of the suppression chamber vacuum breakers is no longer allowed. Therefore, the staff finds the proposed change to be acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. We have determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents which may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: John Stang

Date: October 22, 1990