

May 16, 1991

Docket No. 50-341

Mr. William S. Orser  
Senior Vice President - Nuclear  
Operations  
Detroit Edison Company  
6400 North Dixie Highway  
Newport, Michigan 48166

DISTRIBUTION

Docket file ✓  
NRC & Local PDRs  
PD31 R/F  
BBoger  
JZwolinski  
PShuttleworth  
OGC  
JStang

EJordan  
GHill (4)  
Wanda Jones  
JCalvo  
ACRS (10)  
GPA/PA  
ARM/LFMB  
DHagan

Dear Mr. Orser:

SUBJECT: AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. NPF-43:  
(TAC NO. 79794)

The Commission has issued the enclosed Amendment No. 70 to Facility Operating License No. NPF-43 for the Fermi-2 facility. This amendment consists of changes to the Plant Technical Specifications (TS) in response to your letter dated January 28, 1991 (NRC-90-0182).

The amendment revises Technical Specification Table 3.6.3-1, Primary Containment Isolation Valves, by modifying the numbers for two valves.

A copy of the Safety Evaluation supporting this amendment is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

John F. Stang, Project Manager  
Project Directorate III-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 70 to NPF-43
2. Safety Evaluation

cc w/enclosures:

See next page

LA/PD31:DRP345  
PShuttleworth

4/15/91 *mur*  
*except AS*

PM/PD31:DRP345  
JStang

4/17/91

D/PD31:DRP345  
LMarsh

5/17/91

OFFICIAL RECORD COPY

*SUBJECT TO NOTES  
CHANGES  
OP 70 OF SER.  
9AB  
OGC  
EHOLLER  
5/10/91*

9105300275 910516  
PDR ADDCK 05000341  
P PDR

0 016

NRC FILE CENTER COPY

*CP-1  
QFOL  
111*

May 16, 1991

Docket No. 50-341

Mr. William S. Orser  
Senior Vice President - Nuclear  
Operations  
Detroit Edison Company  
6400 North Dixie Highway  
Newport, Michigan 48166

DISTRIBUTION

Docket file  
NRC & Local PDRs  
PD31 R/F  
BBoger  
JZwolinski  
PShuttleworth  
OGC  
JStang

EJordan  
GHill (4)  
Wanda Jones  
JCalvo  
ACRS (10)  
GPA/PA  
ARM/LFMB  
DHagan

Dear Mr. Orser:

SUBJECT: AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. NPF-43:  
(TAC NO. 79794)

The Commission has issued the enclosed Amendment No. 70 to Facility Operating License No. NPF-43 for the Fermi-2 facility. This amendment consists of changes to the Plant Technical Specifications (TS) in response to your letter dated January 28, 1991 (NRC-90-0182).

The amendment revises Technical Specification Table 3.6.3-1, Primary Containment Isolation Valves, by modifying the numbers for two valves.

A copy of the Safety Evaluation supporting this amendment is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/  
John F. Stang, Project Manager  
Project Directorate III-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 70 to NPF-43
- 2. Safety Evaluation

cc w/enclosures:  
See next page

LA/PD31:DRP345  
PShuttleworth  
4/15/91 *MLR*  
*except AS*

PM/PD31:DRP345  
JStang  
4/17/91

*Am*  
D/PD31:DRP345  
LMarsh  
5/17/91  
OFFICIAL RECORD COPY

*SUBJECT TO POINT  
CHANGES POINTED  
OUT TO OFSER.*  
OGC  
*EHOCCOR*  
5/10/91



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

May 16, 1991

Docket No. 50-341

Mr. William S. Orser  
Senior Vice President - Nuclear  
Operations  
Detroit Edison Company  
6400 North Dixie Highway  
Newport, Michigan 48166

Dear Mr. Orser:

SUBJECT: AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. NPF-43:  
(TAC NO. 79794)

The Commission has issued the enclosed Amendment No. 70 to Facility Operating License No. NPF-43 for the Fermi-2 facility. This amendment consists of changes to the Plant Technical Specifications (TS) in response to your letter dated January 28, 1991 (NRC-90-0182).

The amendment revises Technical Specification Table 3.6.3-1, Primary Containment Isolation Valves, by modifying the numbers for two valves.

A copy of the Safety Evaluation supporting this amendment is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "John F. Stang".

John F. Stang, Project Manager  
Project Directorate III-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 70 to NPF-43
2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. William Orser  
Detroit Edison Company

Fermi-2 Facility

cc:

John Flynn, Esq.  
Senior Attorney  
Detroit Edison Company  
2000 Second Avenue  
Detroit, Michigan 48226

Nuclear Facilities and Environmental  
Monitoring Section Office  
Division of Radiological Health  
P. O. Box 30195  
Lansing, Michigan 48909

Mr. Walt Rogers  
U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
6450 W. Dixie Highway  
Newport, Michigan 48166

Monroe County Office of Civil  
Preparedness  
963 South Raisinville  
Monroe, Michigan 48161

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Ms. Lynne Goodman  
Director - Nuclear Licensing  
Detroit Edison Company  
Fermi Unit 2  
6400 North Dixie Highway  
Newport, Michigan 48166



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

DETROIT EDISON COMPANY

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70  
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Detroit Edison Company (the licensee) dated January 28, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 70, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

9105300281 910516  
PDR ADDCK 05000341  
P PDR

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



L. B. Marsh, Director  
Project Directorate III-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 16, 1991



TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

MAXIMUM  
ISOLATION TIME  
(Seconds)

VALVE FUNCTION AND NUMBER

B. Remote-Manual Isolation Valves <sup>(e)</sup> (Continued)

19. Primary Containment Monitoring System Torus Suction Isolation Valves

NA

Division I: T50-F407A  
Division II: T50-F407B

20. Drywell Atmosphere Sample Isolation Valves

NA

Division I: T50-F401A  
T50-F402A  
T50-F403A  
T50-F404A  
T50-F405A

Division II: T50-F401B  
T50-F402B  
T50-F403B  
T50-F404B  
T50-F405B

21. Drywell to Suppression Chamber Vacuum Breakers N, Supply Isolation Valves

NA

T48-F416  
T48-F417  
T48-F418  
T48-F419  
T48-F420  
T48-F421  
T48-F422  
T48-F423  
T48-F424  
T48-F425  
T48-F426  
T48-F427

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
B. <u>Remote-Manual Valves</u> <sup>(e)</sup> (Continued)	
22. <u>Drywell Pressure Instrumentation Isolation Valves</u>	NA
Division I: T50-F420A	
Division II: T50-F420B	
23. <u>Suppression Pool Level Instrumentation Isolation Valves</u>	NA
Division I: E41-F401 <sup>(b)</sup> T50-F412A <sup>(b)</sup> E41-F400	
Division II: E41-F403 <sup>(b)</sup> T50-F412B <sup>(b)</sup> E41-F402	
24. <u>EECW Supply to Drywell Equipment Isolation Valves</u>	NA
Division I: P44-F606A	
Division II: P44-F606B	
25. <u>EECW Return from Drywell Equipment Isolation Valves</u>	NA
Division I: P44-F607A P44-F616	
Division II: P44-F607B P44-F615	



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. NPF-43

DETROIT EDISON COMPANY

FERMI-2

DOCKET NO. 50-341

1.0 INTRODUCTION

By letter dated January 28, 1991, The Detroit Edison Company (DECo or the licensee) requested amendment to the Technical Specifications (TS) appended to Facility Operating License No. NPF-43 for Fermi-2. The proposed amendment changes TS Table 3.6.3-1, Primary Containment Isolation Valves, by modifying the numbers for two valves.

2.0 EVALUATION

TS Table 3.6.3-1, Primary Containment Isolation Valves, contains a listing of isolation valves which are required to be operational, and includes the valve function and the valve number. The proposed amendment would change Table 3.6.3-1 to be consistent with a planned change in valve numbering for two isolation valves. The two affected valves are: The Division I Emergency Equipment Cooling Water (EECW) Supply to Drywell Equipment Isolation Valve and the Division I EECW Return from Drywell Equipment Isolation Valve each of which will be renumbered.

The renumbering of these valves is planned in conjunction with the resolution of Human Engineering Discrepancy (HED) 1150, which resulted from the Fermi-2 Detailed Control Room Design Review. HED 1150 deals with improving the layout of EECW and Reactor Building Closed Cooling Water (RBCCW) controls in the Control Room. The new layout will be more logical in arrangement and consistent between the two divisions, which are on different Control Room panels.

The proposed amendment is strictly administrative in nature. The functions of each valve, including the containment isolation function, are not affected and the change has no physical effect except for plant labelling. As indicated above, the renumbering enhances the Control Room design by implementing a human engineering improvement which will improve the man/machine interface by eliminating the valve numbering inconsistency. Based on the above, the staff finds the proposed change is administrative in nature and, therefore, is acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Michigan State official was notified of the proposed issuance of the amendment. The State official has no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (56 FR 11778). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: John Stang

Date: May 16, 1991