

July 27, 1988

Docket No. 50-341

Mr. B. Ralph Sylvia
Group Vice President - Nuclear
Operations
Detroit Edison Company
6400 North Dixie Highway
Newport, Michigan 48166

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Dear Mr. Sylvia:

SUBJECT: AMENDMENT NO.22 TO FACILITY OPERATING LICENSE NO. NPF-43: REMOVAL OF SLCS CIRCUIT BREAKER POSITIONS (TAC NO. 66660)

The Commission has issued the enclosed Amendment No. 22 to Facility Operating License No. NPF-43 for the Fermi-2 facility. This amendment consists of changes to the Plant Technical Specifications in response to your letter dated November 19, 1987 (NRC-87-0201).

The amendment corrects errors in the Technical Specifications by removing from Table 3.8.4.5-1 Motor Control Center circuit breakers that do not exist in the plant.

A copy of the Safety Evaluation supporting this amendment is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Theodore R. Quay, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosures:

1. Amendment No. 22 to NPF-43
2. Safety Evaluation

cc w/enclosures:

See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
July 27, 1988

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Group Vice President - Nuclear
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Detroit Edison Company
6400 North Dixie Highway
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2. Safety Evaluation

cc w/enclosures:
See next page

Mr. B. Ralph Sylvia
Detroit Edison Company

Fermi-2 Facility

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DETROIT EDISON COMPANY

WOLVERINE POWER SUPPLY COOPERATIVE, INCORPORATED

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 22
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Detroit Edison Company (the licensee) dated November 19, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

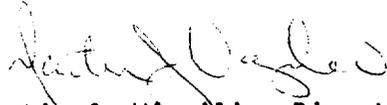
Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 22, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Martin J. Virgilio, Director
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 27, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 22

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

REMOVE

3/4 8-27

INSERT

3/4 8-27

TABLE 3.8.4.5-1

STANDBY LIQUID CONTROL SYSTEM ASSOCIATED ISOLATION DEVICES
480 V MOTOR CONTROL CENTERS

MCC 72B-4A

Position 2F

SLC Heat Trace A

MCC 72B-4C

Position 2AR

SLC Pump A

MCC 72C-4A

Position 5C

SLC Heater A

MCC 72E-4B

Position 1AR

SLC Heat Trace B

MCC 72E-5B

Position 2B

SLC Pump B

Position 2CR

SLC Heater B



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 22 TO FACILITY OPERATING LICENSE NO. NPF-43

DETROIT EDISON COMPANY

WOLVERINE POWER SUPPLY COOPERATIVE, INCORPORATED

FERMI-2

DOCKET NO. 50-341

1.0 INTRODUCTION AND BACKGROUND

By letter dated November 19, 1987, the Detroit Edison Company (DECo or the licensee) requested amendment to the Technical Specifications (TSs) appended to Facility Operating License No. NPF-43 for Fermi-2. The proposed amendment would correct errors in the TSs by deleting from Section 3/4.8.4.5, "Standby Liquid Control System Associated Isolation Devices," Table 3.8.4.5-1, the specification of Motor Control Center (MCC) 72B-4C, Position 1A circuit breaker, and MCC 72E-5B, Position 1C circuit breaker, for required periodic testing. The circuit breakers have been found not to exist in the plant, and as such, the proposed amendment would correct the TSs to be consistent with the as-built plant design. The breakers that perform the isolation function are still in the Table.

The Fermi-2 Standby Liquid Control System (SLCS) is a special capability backup system provided by the licensee to satisfy the requirements of General Design Criterion 26 of Appendix A to 10 CFR Part 50. The system is independent of, and diverse to, the control rods for shutting down the reactor in the event that multiple failures prevent the insertion of the control rods. Portions of the system are redundant. Accordingly, this manually operated system is subject to a single failure. Although the SLCS is not fully redundant, power buses, pumps, and explosive operated injection valves are redundant so that a single component may be removed for maintenance during plant operations.

The NRC staff's initial acceptance of the Fermi-2 SLCS design was documented in the Fermi-2 Safety Evaluation Report (SER) (NUREG-0798), issued in July 1981. However, subsequent to the staff's initial review findings, concerns were raised that the Fermi-2 SLCS design did not conform with NRC standards for safety-related systems and that the Fermi-2 SLCS components were not covered by the licensee's Quality Assurance Program. These concerns were documented in Supplement Number 5 to the Fermi-2 SER and were discussed with the licensee, resulting in the licensee's agreement to implement a program to confirm the quality of the Fermi-2 SLCS design and construction as a licensing basis commitment.

The licensee, in keeping with its commitments, subsequently implemented TS 3/4.8.4.5 to ensure the continued reliability and availability of the SLCS. This TS specifies the periodic surveillance testing of the non-Class IE circuit breakers associated with the SLCS. These non-Class IE circuit

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breakers, actuated by fault currents, are used as the isolation devices for protecting equipment associated with the SLCS. The operability requirements for these circuit breakers, listed in TS Table 3.8.4.5-1, ensures that the SLCS equipment is protected in the event of faults in the loads powered by the MCCs in which those circuit breakers are connected.

2.0 EVALUATION

As a result of the licensee's TS improvement program, the licensee discovered that TS 3/4.8.4.5, Table 3.8.4.5-1, lists two circuit breaker positions, MCC 72B-4C, Position 1A, and MCC 72E-5B, Position 1C, which do not exist. This has been verified by a review of the system design documents and by a system walkdown. Accordingly, the licensee's letter dated November 19, 1987 (NRC-87-0201), requested deletion to these two items from the TSs.

The NRC staff has reviewed the licensee's request as well as the licensee's response to the staff identified concerns as discussed above. The staff's review considered electrical drawings and the schematics reflecting the SLCS design and the system's associated isolation devices and has confirmed that the two circuit breaker positions do not exist. The breakers that perform the isolation function are still in the table. Therefore, the staff concludes that the proposed change is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents which may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Quay

Dated: July 27, 1988

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