

September 27, 1990

Docket No. 50-341

Mr. William S. Orser
Senior Vice President - Nuclear
Operations
Detroit Edison Company
6400 North Dixie Highway
Newport, Michigan 48166

Dear Mr. Orser:

SUBJECT: ERRATA FOR AMENDMENTS 56 AND 57 TO FACILITY OPERATING LICENSE NO.
NPF-43: (TAC NOS. 71227 AND 75143)

The changes to plant Technical Specifications (TS) implemented by License Amendment Nos. 56 and 57 to Facility Operating License No. NPF-43 for Fermi-2, which were transmitted to you by letters dated September 10 and September 13, 1990, have been found to contain text errors. The enclosed errata to the TS changes implemented by License Amendment No. 56 and the corrected page to the Safety Evaluation associated with Amendment No. 57 are hereby transmitted and should replace the pages previously transmitted.

Sincerely,

/s/
John F. Stang, Project Manager
Project Directorate III-1
Division of Reactor Projects - III,
IV, V & Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script that reads "John F. Stang".

John F. Stang, Project Manager
Project Directorate III-1
Division of Reactor Projects - III,
IV, V & Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: See next page

Mr. William Orser
Detroit Edison Company

Fermi-2 Facility

cc:

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Newport, Michigan 48166

TABLE 3.3.7.5-1
ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	REQUIRED NUMBER OF CHANNELS	MINIMUM CHANNELS OPERABLE	APPLICABLE OPERATIONAL CONDITIONS	ACTION
1. Reactor Vessel Pressure	2	1	1, 2	80
2. Reactor Vessel Water Level				
a. Fuel Zone	2	1	1, 2	80
b. Wide Range	2	1	1, 2	80
3. Suppression Chamber Water Level	2	1	1, 2	80
4. Suppression Chamber Water Temperature	2	1	1, 2	80
5. Suppression Chamber Air Temperature	2	1	1, 2	80
6. Suppression Chamber Pressure	2	1	1, 2	80
7. Drywell Pressure, Wide Range	2	1	1, 2	80
8. Drywell Air Temperature	2	1	1, 2	80
9. Drywell Oxygen Concentration	2	1	1, 2	80
10. Drywell Hydrogen Concentration	2	1	1, 2	80
11. Safety/Relief Valve Position Indicators	1*/valve	1*/valve	1, 2	80
12. Containment High Range Radiation Monitor	2	2	1, 2, 3	81
13. Standby Gas Treatment System Radiation Monitors**	2	2	1, 2, 3	81
14. Neutron Flux	2	1	1, 2	80
15. Deleted				
16. Primary Containment Isolation Valve Position	1/valve	1/valve	1, 2, 3	82

*Pressure switch

**High (accident) range noble gas monitors (one channel per flow path)

containment is lost under accident conditions and can be reopened under the conditions that would prevail when the valve reopening is appropriate. The valve will be normally open and will have fail as-is logic. Position 1.d of Regulatory Guide 1.11 requires that components up to and including the isolation valve should be designed and installed to the quality at least equivalent to the containment. The sensing line piping and valves up to and including the remote manual isolation valve and second test connection valve will be designed, fabricated and installed per ASME Code, Section III, Class 2. The remainder of the sensing line will be installed in an accessible area approximately eight feet above the Reactor Building third floor which will provide for visual inspection and testing. The piping assembly will be installed to preclude any failure of one line inducing failure of another. Position 1.e of Regulatory Guide 1.11 requires that instrument lines penetrating primary containment should not be so restricted by components in the line. The in-line components of the proposed sensing line will be selected to minimize the restriction presented to the transmission of drywell pressure to the transmitter diaphragm.

The installation of the new sensing line at penetration X-27f will be conducted in two phases. The first phase was completed during the recent refueling outage (December 1989) which installed the manual in-line valve and the test connection and associated valves. A qualified welded cap was used to seal closed the line where the remote manual valve will be installed later. The penetration, including the in-line manual valve, was leak tested per the requirements of 10 CFR Part 50, Appendix J prior to unit restart. The second installation phase will take place with the unit in power operation. The welded cap will be removed and remaining installation of the remote manual valve, associated wiring and the blind flange assembly for the pressure transmitter sensing diaphragm, will be completed. When the welded cap is removed from the penetration for phase two of the installation, the licensee will enter into the Action Statement a. 3 of TS Section 3.6.3. The Action Statement allows continued operation of the plant with the one locked closed manual containment isolation valve until the plant is shut down.

Following the installation, the penetration will be leak tested per the requirements of 10 CFR Part 50, Appendix J, the remote-manual valve operation will be tested, and the new TS requirements will be implemented.

Based on the above evaluation the staff finds the modifications to the plant to resolve HED 462 are acceptable and the addition of the new manual-remote containment isolation valve to Section 3/4.6.3, Table 3.6.3-1 is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. We have determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents which may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public