

April 15, 1988

Docket No. 50-341

Mr. B. Ralph Sylvia  
Group Vice President - Nuclear  
Operations  
Detroit Edison Company  
6400 North Dixie Highway  
Newport, Michigan 48166

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Dear Mr. Sylvia:

SUBJECT: AMENDMENT NO. 18 TO FACILITY OPERATING LICENSE NO. NPF-43: PRIMARY CONTAINMENT LEAKAGE (TAC NO. 67866)

The Commission has issued the enclosed Amendment No. 18 to Facility Operating License No. NPF-43 for the Fermi-2 facility. This amendment consists of changes to the Plant Technical Specifications in response to your letter dated April 11, 1988.

The amendment revises Technical Specification 4.6.1.2.b to provide a one-time extension of the 24-month surveillance interval for Type C Local Leak Rate Tests of three Residual Heat Removal shutdown cooling isolation valves.

A copy of the Safety Evaluation supporting this amendment is also enclosed. Notice of Issuance and Final Determination of No Significant Hazards Consideration and Opportunity for Hearing will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Theodore R. Quay, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III, IV, V  
& Special Projects

Enclosures:

1. Amendment No. 18 to NPF-43
2. Safety Evaluation

cc w/enclosures:  
See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

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Mr. B. Ralph Sylvia  
Detroit Edison Company

Fermi-2 Facility

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DETROIT EDISON COMPANY  
WOLVERINE POWER SUPPLY COOPERATIVE, INCORPORATED

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 18  
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Detroit Edison Company (the licensee) dated April 11, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 18, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Theodore R. Sway for*

Martin J. Virgilio, Director  
Project Directorate III-1  
Division of Reactor Projects - III, IV, V  
& Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 15, 1988

ATTACHMENT TO LICENSE AMENDMENT NO.18

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE

3/4 6-6

INSERT

3/4 6-6

## CONTAINMENT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

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- (c) Dewpoint temperature sensors shall have an accuracy of  $\pm 1^{\circ}\text{F}$  or better over the dewpoint temperature range expected during the test  $\pm 20^{\circ}\text{F}$  and a repeatability of at least  $\pm 0.5^{\circ}\text{F}$ .
  - (d) Pressure sensors should have a range such that  $P_a$  is between 25 and 75% of full scale. Accuracy shall be at least 0.015% of full scale with resolution and repeatability of 0.001% of full scale.
  - (e) The number and location of temperature and dewpoint sensors shall be determined prior to each Type A test based on a temperature survey of the containment.
  - (f) A sufficient number of dry bulb temperature sensors must be functioning properly during the test such that no sensor contributes more than 10% to the calculated temperature.
  - (g) At least two-thirds of the dewpoint temperature sensors shall be functioning properly during the test. However, if data recorded over the last 5 hours indicate that dewpoint temperatures have stabilized and any changes are not of an order to cause error in leak rate calculations, then malfunction of any or all but three of the dewpoint sensors shall not require aborting the test.
  - (h) At least one precision pressure gauge shall be functioning properly during the test.
  - (i) Prior to each Type A test and following the failure of any sensor, an instrument error analysis shall be performed using the Instrument Selection Guide (ISG) formula of ANSI/ANS-56.8-1981. The ISG shall not exceed  $0.25 L_a$  at the end of a test except as noted in (g) above.
8. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at  $40 \pm 10$  month intervals during shutdown at  $P_a$ , 56.5 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.
9. If any periodic Type A test fails to meet  $0.75 L_a$ , the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet  $0.75 L_a$ , a type A test shall be performed at least every 18 months until two consecutive Type A tests meet  $0.75 L_a$ , at which time the above test schedule may be resumed.

SURVEILLANCE REQUIREMENTS (Continued)

10. The accuracy of each Type A test shall be verified by a supplemental test which:
- (a) Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within  $0.25 L_a$ .
  - (b) Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
  - (c) Requires that the rate of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 75% but not more than 125% of  $L_a$  at  $P_a$ , 56.5 psig,
- b. Type B and C tests shall be conducted with gas at  $P_a$ , 56.5 psig\*, at intervals no greater than 24 months\*\* except for tests involving:
    1. Air locks,
    2. Main steam line isolation valves,
    3. Penetrations using continuous leakage monitoring systems,
    4. Valves pressurized with fluid from a seal system,
    5. ECCS and RCIC containment isolation valves in hydrostatically tested lines which penetrate the primary containment, and
    6. Purge supply and exhaust isolation valves with resilient material seals.
  - c. Air locks shall be tested and demonstrated OPERABLE per Specification 4.6.1.3.
  - d. Main steam line isolation valves shall be leak tested at least once per 18 months.
  - e. Type B tests for penetrations employing a continuous leakage monitoring system shall be conducted at  $P_a$ , 56.5 psig, at intervals no greater than once per 3 years.
  - f. Leakage from isolation valves that are sealed with fluid from a seal system may be excluded, subject to the provisions of Appendix J, Section III.C.3, when determining the combined leakage rate provided the seal system and valves are pressurized to at least  $1.10 P_a$ , 62.2 psig, and the seal system capacity is adequate to maintain system pressure for at least 30 days.

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\*Unless a hydrostatic test is required per Table 3.6.3-1.

\*\*With the exception of valves E11-F009, E11-F408 and E11-F608 for which the surveillance interval has been extended until startup from the first refueling outage late in 1989.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 18 TO FACILITY OPERATING LICENSE NO. NPF-43

DETROIT EDISON COMPANY

WOLVERINE POWER SUPPLY COOPERATIVE, INCORPORATED

FERMI-2

DOCKET NO. 50-341

1.0 INTRODUCTION

By letter dated April 11, 1988, the Detroit Edison Company (DECo or the licensee) requested an emergency Technical Specification (TS) change to make the Fermi-2 TSs consistent with an exemption that had been requested in an earlier letter dated February 22, 1988.

2.0 EVALUATION

The February 22, 1988 letter from DECo to the Commission requested a one-time exemption from 10 CFR Part 50, Appendix J, for three inboard isolation valves for the Residual Heat Removal System. This exemption was approved by the Commission on April 15, 1988. The requested emergency change to TS 3/4.6.1.2 would make the Fermi-2 TSs consistent with the one-time scheduler exemption already approved by the Commission. The scheduler exemption from 10 CFR Part 50, Appendix J, Section III.D.3, concerning the Type C Local Leak Rate Testing discusses the safety of the exemption and is fully applicable to the proposed change. Consequently, the staff finds that the requested change is acceptable.

3.0 EMERGENCY CIRCUMSTANCES

In its April 11, 1988 letter, the licensee requested that this amendment be treated as an emergency because insufficient time exists for the Commission's usual 30-day notice without extending the current outage.

Technical Specification 3/4.6.1.2 requires that the valves covered by the one-time exemption be tested once every two years. Without the TS modification, the TSs would not permit the plant to be restarted.

In accordance with 10 CFR 50.91(a)(5), the licensee could not have avoided this emergency situation because of oversight on the part of the staff that a TS amendment was required. The Commission has determined that emergency circumstances exist in that swift action is necessary to avoid a delay in startup not related to safety.

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#### 4.0 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The Commission's regulations in 10 CFR 50.92 state that the Commission may make a final determination that a license amendment involves no significant hazards considerations if operation of the facility, in accordance with the amendment, would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from an accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

The Commission has determined that the proposed TS change: (1) does not involve a significant increase in the probability or consequences of an accident previously evaluated as the change will make the TSs consistent with an approved one-time exemption; (2) does not create the possibility of a new or different kind of accident from any accident previously evaluated since the change is consistent with the approved exemption; and (3) does not involve a significant reduction in a margin of safety since, as stated in (1) and (2) above, the change is consistent with an approved exemption.

Therefore, based on these considerations and the three criteria given above, the Commission has made a final determination that the amendment request involves no significant hazards consideration.

#### 5.0 STATE CONSULTATION

In accordance with the Commission's regulations, efforts were made to contact the Michigan representative. The state representative was contacted and had no comments.

#### 6.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has made a final no significant hazards consideration finding with respect to this amendment. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

7.0 CONCLUSION

We have concluded, based on the considerations discussed above, that:  
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Quay, NRR

Dated: April 15, 1988