



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 3, 1993

Docket No. 50-302

Mr. Percy M. Beard, Jr.
Senior Vice President,
Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
Crystal River Energy Complex
15760 W Power Line Street
Crystal River, Florida 34428-6708

Dear Mr. Beard:

SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF AMENDMENT RE: CONTAINMENT
TENDON SURVEILLANCE (TAC NO. M74563)

The Commission has issued the enclosed Amendment No. 148 to Facility Operating License No. DPR-72 for the Crystal River Unit 3. The amendment consists of changes to the existing Technical Specifications (TS) in response to your letter dated October 25, 1993, in which you requested expedited issuance of the portion of the Improved Technical Specification (ITS) dealing with containment integrity, necessary to support containment tendon surveillance testing.

Containment tendon surveillance testing was scheduled to begin on November 1, 1993, to prevent exceeding the TS surveillance interval which expires on January 10, 1994. Issuance of the ITS is likely in this time interval, which creates a conflict of requirements, since the ITS and the current TS differ in this area. We are in agreement that it is preferable to perform the containment tendon testing to the currently accepted NRC standards contained in the ITS.

Please note that the action statement for TS 3.6.1.6 has not been changed by this amendment. We discussed this with your staff on November 23, 1993.

Your letter of August 25, 1989 transmitted a request to replace the existing TS in its entirety with the ITS (including containment integrity). A notice was placed in the Federal Register (54 FR 46998) covering this request, and providing an opportunity for hearing. No request for hearing has been received. The staff's review of the remainder of that request is continuing at this time.

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Mr. Percy M. Beard, Jr.

- 2 -

A copy of the Safety Evaluation is enclosed. Also enclosed is the Notice of Issuance which has been forwarded to the Office of the Federal Register for publication.

Sincerely,
(Original Signed By)

Harley Silver, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 148 to DPR-72
2. Safety Evaluation
3. Federal Register notice

cc w/enclosures: See next page

AMENDMENT NO. 148 TO FACILITY OPERATING LICENSE NO. DPR-72-CRYSTAL RIVER UNIT 3

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* See previous concurrence

DATED: December 3, 1993

AMENDMENT NO. 148 TO FACILITY OPERATING LICENSE NO. DPR-72-CRYSTAL RIVER UNIT 3

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D. Hagan, 3302 MNBB

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Mr. Percy M. Beard
Florida Power Corporation

Crystal River Unit No.3
Generating Plant

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CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 148
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated August 25, 1989, as supplemented October 25, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

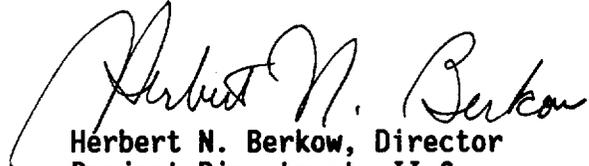
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 148, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 3, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 148

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove

3/4 6-1

3/4 6-8

3/4 6-9

3/4 6-9a

B 3/4 6-2

6-22

Insert

3/4 6-1

3/4 6-8

3/4 6-9

3/4 6-9a

B 3/4 6-2

6-22

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

a. At least once per 31 days by verifying that:

1. All penetrations* not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except those valves that may be opened under administrative controls per Specification 3.6.3.1, and
2. All equipment hatches are closed and sealed.

b. By verifying that each containment air lock is OPERABLE per Specification 3.6.1.3.

4.6.1.1.2 The structural integrity of the exposed accessible interior and exterior surfaces of the containment, including the liner plate, shall be determined during the shutdown for each Type A containment leakage rate test (Specification 4.6.1.2) by a visual inspection of these surfaces. This inspection shall be performed prior to the Type A containment leakage rate test to verify no apparent changes in appearance or other abnormal degradation.

*Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked, sealed, or otherwise secured in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that verification of these penetrations being closed need not be performed more often than once per 92 days.

CONTAINMENT SYSTEMS

AIR TEMPERATURE

LIMITING CONDITION FOR OPERATION

3.6.1.5 Primary containment average air temperature shall not exceed 130°F.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the containment average air temperature > 130°F, reduce the average air temperature to within the limit within 8 hours, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.5 The primary containment average air temperature shall be the arithmetical average of the temperatures at the following locations and shall be determined at least once per 24 hours:

Location

- a. Column RB-320, elevation 100'
- b. Column RB-320, elevation 125'
- c. Outside secondary shield wall, elevation 180'
- d. Crane access platform, elevation 235'

CONTAINMENT SYSTEMS

CONTAINMENT STRUCTURAL INTEGRITY

LIMITING CONDITIONS FOR OPERATION

3.6.1.6 The structural integrity of the containment shall be maintained at a level consistent with the acceptance criteria in Specification 4.6.1.6.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With the structural integrity of the containment not conforming to the above requirements, restore the structural integrity to within the limits within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.6.1 Verify containment structural integrity in accordance with the Containment Tendon Surveillance Program. The frequency shall be in accordance with the Containment Tendon Surveillance Program.

4.6.1.6.2 Any abnormal degradation of the containment structure detected during the tests required by the Containment Tendon Surveillance Program shall be reported to the Commission within 30 days. The report shall include a description of the tendon condition, the condition of the concrete (especially at tendon anchorages), the inspection procedures, the tolerances on cracking, and the corrective action taken.

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3/4.6 CONTAINMENT SYSTEMS

BASES

3/4.6.1 PRIMARY CONTAINMENT

3/4.6.1.1 CONTAINMENT INTEGRITY

Primary CONTAINMENT INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the safety analyses. This restriction, in conjunction with the leakage rate limitation, will limit the site boundary radiation doses to within the limits of 10 CFR 100 during accident conditions.

3/4.6.1.2 CONTAINMENT LEAKAGE

The limitations on containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the safety analyses at the peak accident pressure of 53.3 psig, P_a . As an added conservatism, the measured overall integrated leakage rate is further limited to $\leq 0.75 L_a$ during performance of the periodic tests to account for possible degradation of the containment leakage barriers between leakage tests.

The surveillance testing for measuring leakage rates are consistent with the requirements of Appendix "J" of 10 CFR 50.

3/4.6.1.3 CONTAINMENT AIR LOCKS

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on CONTAINMENT INTEGRITY and containment leak rate given. Surveillance testing of the air lock seals provide assurance that the overall air lock leakage will not become excessive due to seal damage during the intervals between air lock leakage tests.

CONTAINMENT SYSTEMS

BASES

3/4.6.1.4 INTERNAL PRESSURE

The limitations on containment internal pressure ensure that 1) the containment structure is prevented from exceeding its design negative pressure differential with respect to the outside atmosphere of 3.0 psig and 2) the containment peak pressure does not exceed the design pressure of 54.6 psig during LOCA conditions.

The maximum peak pressure obtained from a LOCA event is 50.3 psig. The limit of 3 psig for initial positive containment pressure will limit the total maximum peak pressure to 53.3 psig which is less than the design pressure and is consistent with the safety analysis.

3/4.6.1.5 AIR TEMPERATURE

The limitations on containment average air temperature ensure that the overall containment average air temperature does not exceed the initial temperature condition assumed in the accident analysis for a LOCA.

3/4.6.1.6 CONTAINMENT STRUCTURAL INTEGRITY

This limitation ensures that the structural integrity of the containment will be maintained comparable to the original design standards for the life of the facility. Structural integrity is required to ensure that the containment will withstand the maximum pressure of 53.3 psig in the event of a LOCA. The testing addressed within Regulatory Guide 1.35, Revision 3, is sufficient to demonstrate this capability.

ADMINISTRATIVE CONTROLS

6.17 POST-ACCIDENT SAMPLING AND AUTOMATED ISOTOPIC MONITORING SYSTEMS

6.17.1 Procedures shall be established, implemented, and maintained to obtain and analyze reactor coolant and containment atmosphere samples. Additional procedures shall address obtaining and analyzing radioactive iodines and particulates in plant gaseous effluents. The procedures shall include the following:

1. Sampling and Analysis, and
2. Maintenance of Sampling and Analysis Equipment.

Personnel shall be trained in the use of the above systems.

6.18 CONTAINMENT TENDON SURVEILLANCE PROGRAM

6.18.1 This program provides controls for monitoring any tendon degradation in concrete containments, including effectiveness of its corrosion protection medium, to ensure containment structural integrity. The program shall include baseline measurements prior to initial operations. The Containment Tendon Surveillance Program, inspection frequencies, and acceptance criteria shall be in accordance with Regulatory Guide 1.35, Revision 3, 1989.

The provisions of Specification 4.0.2 are applicable to the Containment Tendon Surveillance Program inspection frequencies.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 148 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET. AL.
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

1.0 INTRODUCTION

By letter dated August 25, 1989, the Florida Power Corporation (FPC) proposed to amend Appendix A of Operating License No. DPR-72 to replace, in its entirety, the Crystal River Unit 3 (CR-3) Technical Specifications (TS) with Improved TS (ITS). NRC's proposed action on the amendment request was published in the Federal Register (54 FR 46998) on November 8, 1989.

By letter dated October 25, 1993, FPC requested expedited issuance of the containment section of the ITS necessary to support containment tendon testing. Containment tendon surveillance testing was scheduled to begin on November 1, 1993, to prevent exceeding the surveillance interval which expires on January 10, 1994. Issuance of the ITS is likely in this time interval, which would create a conflict of requirements since the ITS and the current TS differ in this area. FPC considers it preferable to perform the containment tendon testing to the currently accepted NRC standards contained in the ITS. This safety evaluation (SE) covers only the containment portion of the ITS.

FPC's proposal to revise the CR-3 TS in its entirety was based on a Babcock & Wilcox (B&W) Topical Report, "B&WOG Revised Standard Technical Specifications" (BAW-2076). During 1989 to 1992, B&W, FPC, other reactor vendor Owners Groups (OG) and the NRC staff developed Standard Technical Specifications (NUREG-1430) for use by B&W plants.

NUREG-1430, the B&W Standard Technical Specifications (STS), became the basis for the CR-3 ITS. Portions of the existing TS were also used as a basis for the ITS. Plant-specific issues, including plant-unique design features, plant-unique requirements, and plant-unique operating practices, were discussed with the licensee during a series of meetings concluding on May 26, 1993.

On July 22, 1993, the Commission published in the Federal Register (58 FR 39132) the Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors (the Policy Statement). The Policy Statement reflects modifications resulting from public comments on the Interim Policy Statement and from experience gained in developing the improved STS, including the B&W STS, NUREG-1430. The Policy Statement states that the Commission encourages licensees to use the improved STS as the basis for plant-specific TS, i.e., a

TS conversion. The Policy Statement also lists the criteria for inclusion of items in TS.

In some instances, changes in the licensee's proposed TS resulted from discussion with the licensee during our review. These changes clarify the TS with respect to the guidance of NUREG-1430 and do not affect the intent or safety significance of the specifications. Therefore, the changes are within the scope of the action described in the Federal Register (54 FR 46998) on November 8, 1989.

2.0 PROPOSED TECHNICAL SPECIFICATION CHANGES

The current TS specify that the structural integrity of the containment shall be maintained at a level consistent with the acceptance criteria specified in the surveillance requirements. If structural integrity is not met, it must be restored within 24 hours or the plant must be taken to cold shutdown. The surveillance requirements then specify the details of the testing.

The proposed TS state that the containment structural integrity must be maintained at a level consistent with the acceptance criteria specified in the surveillance requirements or restore the integrity to within the limits. The proposed TS surveillance state that the containment structural integrity shall be maintained in accordance with the containment tendon surveillance program. Details of the program are not located in the TS; however, the proposed TS state that the containment tendon surveillance program, inspection frequency, and acceptance criteria shall be in accordance with Regulatory Guide 1.35, Revision 3, 1989. The proposed TS also require reporting of any abnormal degradation of the containment structure detected during these tests to the NRC within 30 days.

3.0 DISCUSSION AND EVALUATION

3.1. Technical Specifications Change

The FPC proposal is in accordance with the current staff position regarding tendon surveillance testing since it is based on Regulatory Guide 1.35, Revision 3, 1989.

The proposed relocation of the details on containment tendon surveillance in the CR-3 TS to appropriate plant-specific programs, procedures, and TS Bases follows the guidance of the B&W STS, NUREG-1430. Once these details have been relocated by removing them from the TS to other licensee controlled documents, the licensee may revise them under the provisions of 10 CFR 50.59. This provides appropriate procedural means to control changes. The tendon surveillance program is not totally removed from TS since the TS refer to Regulatory Guide 1.35, Revision 3, 1989 in the administrative control section of the CR-3 TS.

The staff determined that the action statement associated with 3.6.1.6 should not be changed. A clear action statement regarding shutdown is considered necessary rather than relying on application of other action statements. The action statement of TS 3.6.1.6 was therefore not changed by this amendment.

The staff has reviewed the proposed relocation of the details of the containment tendon surveillance program and has concluded that the scope of the Policy Statement criteria is met, and that review supports removal of those details from TS. Therefore, the above relocation is acceptable. The containment section and administrative control section are consistent with NUREG-1430 and reflect the current staff position on containment tendon surveillance. This change provides a programmatic approach to the requirements relating to containment tendon surveillances and inspections. This portion is considered an administrative change in the location of the safety objective within the TS. For the above reasons, the proposed change is technically acceptable.

3.2. Deferral of Regulatory Guide 1.35, Revision 3, Inspections

In the October 25, 1993, letter, FPC committed to follow Regulatory Guide 1.35, Revision 3, except for the timing of testing tendons deferred as part of the fifth CR-3 tendon surveillance.

Per Regulatory Guide 1.35, Revision 3, tendons that are randomly selected but cannot be tested (due to plant conditions at the time) should be inspected during the following plant shutdown. In the case of the fifth CR-3 tendon surveillance this would require testing during the next refueling outage in April 1994, in addition to the scheduled testing which began on November 1, 1993.

Five randomly selected tendons were inaccessible during the November 1993 testing since the testing was performed with the plant on line. Alternate tendons were selected and tested during the November 1993 testing. FPC proposed to perform testing of the inaccessible tendons during the next tendon surveillance currently scheduled for Refuel Outage 10 in April 1996. This is approximately 2 years later than the April 1994 outage, when the testing would normally be conducted per the Regulatory Guide.

FPC stated that previous tendon testing in the area of these five tendons has met the TS requirements. The licensee concluded that operating history indicates that deferral of the testing does not increase the potential for undetected degradation during the time interval. The additional costs and limited availability of the vendor performing the tests make testing in April 1994 impractical for FPC.

Based our review, the staff considers deferral of testing of the inaccessible five tendons for 2 years acceptable.

4.0 SUMMARY

The changes will not affect the capability of the containment to perform its design function. The staff finds the licensee's proposed revision to TS and deferral of testing acceptable.

5.0 STATE CONSULTATION

Based upon written notice of the proposed amendment, the Florida State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATIONS

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact have been prepared and published in the Federal Register on November 16, 1993 (58 FR 60468). Based upon the environmental assessment, the staff has determined that the issuance of the amendment will not have a significant effect on the quality of the human environment.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. P. Croteau

Date: December 3, 1993

UNITED STATES NUCLEAR REGULATORY COMMISSIONFLORIDA POWER CORPORATIONDOCKET NO. 50-302NOTICE OF ISSUANCE OF AMENDMENT TOFACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 148 to Facility Operating License No. DPR-72 issued to Florida Power Corporation (the licensee), which revised the Technical Specifications for operation of Crystal River, Unit No. 3, located in Citrus County, Florida. The amendment is effective as of the date of issuance.

The amendment revises the containment integrity section 3/4.6.1 of the existing Technical Specifications. It is comprised of that portion of the Improved Technical Specifications requested by an application dated August 25, 1989. Separate issuance of this portion was requested by the licensee in a letter dated October 25, 1993.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

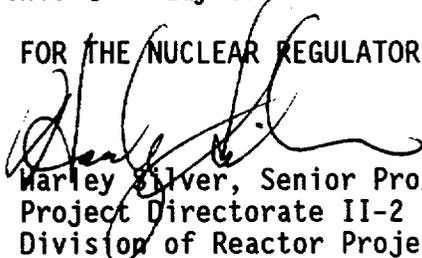
Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER on November 8, 1989 (54 FR 46998). No request for hearing or petition for leave to intervene was filed following this notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an Environmental Impact Statement. Based upon the Environmental Assessment, the Commission has concluded that the issuance of this amendment will not have a significant effect on the quality of the human environment (58 FR 60468).

For further details with respect to the action see (1) the application for amendment dated August 25, 1989, as supplemented October 25, 1993, (2) Amendment No. 148 to License No. DPR-72, (3) the Commission's related Safety Evaluation, and (4) the Commission's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC 20555, and at the local public document room located at the Coastal Region Library, 8619 W. Crystal Street, Crystal River, Florida 32629.

Dated at Rockville, Maryland this 3rd day of December 1993.

FOR THE NUCLEAR REGULATORY COMMISSION


Marley Silver, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II