

March 26, 1990

Docket No. 50-341

Mr. B. Ralph Sylvia
Senior Vice President - Nuclear
Operations
Detroit Edison Company
6400 North Dixie Highway
Newport, Michigan 48166

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Dear Mr. Sylvia:

SUBJECT: AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. NPF-43:
(TAC NO. 72052)

The Commission has issued the enclosed Amendment No. 52 to Facility Operating License No. NPF-43 for the Fermi-2 facility. This amendment consists of changes to the Plant Technical Specifications (TSS) in response to your letter dated December 22, 1988 as supplemented May 10, 1989.

The amendment revises the TSS Section 4.0.5 to incorporate the NRC staff positions on the inspection schedule, methods and personnel, and sample expansion for piping identified in accordance with Generic Letter 88-01.

A copy of the Safety Evaluation supporting this amendment is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

John F. Stang, Project Manager
Project Directorate III-1
Division of Reactor Projects - III,
IV, V & Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 52 to NPF-43
2. Safety Evaluation

cc w/enclosures:
See next page

*See previous concurrence

*LA/PD31:DRSP
PShuttleworth
11/29/89

*PM/PD31:DRSP
JStang
11/29/89

(A) ⁵⁹ PD31:DRSP
JThoma
11/ /89

*OGC
02/12/89

NRR/EMTB
CY Cheng
12/4/89

02/28/90

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John F. Stang, Project Manager
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Senior Vice President - Nuclear
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Detroit Edison Company
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The amendment revises the TSs Section 4.0.5 to incorporate the NRC staff positions on the inspection schedule, methods and personnel, and sample expansion for piping identified in accordance with Generic Letter 88-01.

A copy of the Safety Evaluation supporting this amendment is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "John F. Stang".

John F. Stang, Project Manager
Project Directorate III-1
Division of Reactor Projects - III,
IV, V & Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 52 to NPF-43
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. B. Ralph Sylvia
Detroit Edison Company

Fermi-2 Facility

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DETROIT EDISON COMPANY

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Detroit Edison Company (the licensee) dated December 22, 1988 as supplemented May 10, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 52, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John O. Thoma

John O. Thoma, Acting Director
Project Directorate III-1
Division of Reactor Projects - III,
IV, V & Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 26, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 52

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

3/4 0-3

B 3/4 0-3

3/4 4-25

3/4 4-26*

INSERT

3/4 0-3

B 3/4 0-3

3/4 4-25

3/4 4-26*

* Overleaf page provided to maintain document completeness. No changes contained in these pages.

APPLICABILITY

SURVEILLANCE REQUIREMENTS (Continued)

<u>ASME Boiler and Pressure Vessel Code and applicable Addenda terminology for inservice inspection and testing activities</u>	<u>Required frequencies for performing inservice inspection and testing activities</u>
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Every 9 months	At least once per 276 days
Yearly or annually	At least once per 366 days

- c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities.
- d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.
- e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.
- f. The Inservice Inspection (NDE) Program for piping identified in NRC Generic Letter 88-01, dated January 25, 1988, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping", shall be performed in accordance with the staff positions on schedule, methods and personnel, and sample expansion included in this generic letter.

APPLICABILITY

BASES

4.0.5 This specification ensures that inservice inspection of AMSE Code Class 1, 2 and 3 components and inservice testing of ASME Code Class 1, 2 and 3 pumps and valves will be performed in accordance with a periodically updated version of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda as required by 10 CFR 50, Section 50.55a. Relief from any of the above requirements has been provided in writing by the Commission and is not a part of these Technical Specifications.

This specification includes a clarification of the frequencies of performing the inservice inspection and testing activities required by Section XI of the ASME Boiler and Pressure Code and applicable Addenda. This clarification is provided to ensure consistency in surveillance intervals throughout these Technical Specifications and to remove any ambiguities relative to the frequencies for performing the required inservice inspection and testing activities.

Under the terms of this specification, the more restrictive requirements of the Technical Specifications take precedence over the ASME Boiler and Pressure Vessel Code and applicable Addenda. For example, the surveillance activities prior to entry into an OPERATIONAL CONDITION or other specified applicability condition takes precedence over the ASME Boiler and Pressure Vessel Code provision which allows pumps to be tested up to one week after return to normal operation. And for example, the Technical Specification definition of OPERABLE does not grant a grace period before a device that is not capable of performing its specified function is declared inoperable and takes precedence over the ASME Boiler and Pressure Vessel provision which allows a valve to be incapable of performing its specified function for up to 24 hours before being declared inoperable.

NRC Generic Letter 88-01 identified NRC's position on intergranular stress corrosion cracking (IGSCC) in BWR Stainless Steel weldments. This letter establishes requirements to inspect these weldments to a special schedule and by procedures, personnel and equipment that are qualified under a formal program approved by the NRC.

REACTOR COOLANT SYSTEM

3/4.4.8 STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.4.8 The structural integrity of ASME Code Class 1, 2, and 3 components shall be maintained in accordance with Specification 4.4.8.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, 4, and 5.

ACTION:

- a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the reactor coolant system temperature more than 50°F above the minimum temperature required by NDT considerations.
- b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the reactor coolant system temperature above 200°F.
- c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) from service.
- d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.8 No requirements other than Specification 4.0.5.

REACTOR COOLANT SYSTEM

3/4.4.9 RESIDUAL HEAT REMOVAL

HOT SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.4.9.1 Two# shutdown cooling mode loops of the residual heat removal (RHR) system shall be OPERABLE and, at least one recirculation pump shall be in operation or, at least one shutdown cooling mode loop shall be in operation## with each loop consisting of at least:

- a. One OPERABLE RHR pump, and
- b. One OPERABLE RHR heat exchanger.

APPLICABILITY: OPERATIONAL CONDITION 3, with reactor vessel pressure less than the RHR cut-in permissive setpoint.

ACTION:

- a. With less than the above required RHR shutdown cooling mode loops OPERABLE, immediately initiate corrective action to return the required loops to OPERABLE status as soon as possible. Within 1 hour and at least once per 24 hours thereafter, verify the OPERABILITY of at least one alternate method capable of decay heat removal for each inoperable RHR shutdown cooling mode loop. Be in at least COLD SHUTDOWN within 24 hours.**
- b. With neither a recirculation pump nor an RHR shutdown cooling mode loop in operation, immediately initiate corrective action to return either at least one recirculation pump or at least one RHR shutdown cooling mode loop to operation as soon as possible. Within 1 hour establish reactor coolant circulation by an alternate method and monitor reactor coolant temperature and pressure at least once per hour.
- c. The provisions of Specification 3.0.4 are not applicable for up to 4 hours for the purpose of establishing the RHR system in the shutdown cooling mode once the reactor vessel pressure is less than the RHR cut-in permissive setpoint.

SURVEILLANCE REQUIREMENTS

4.4.9.1.1 At least one shutdown cooling mode loop of the residual heat removal system or at least one recirculation pump shall be determined to be in operation and circulating reactor coolant at least once per 12 hours.

#One RHR shutdown cooling mode loop may be inoperable for up to 2 hours for surveillance testing.

*The shutdown cooling pump may be removed from operation for up to 2 hours per 8-hour period.

##The RHR shutdown cooling mode loop may be removed from operation during hydrostatic testing.

**Whenever both RHR shutdown cooling mode loops are inoperable, if unable to attain COLD SHUTDOWN as required by this ACTION, maintain reactor coolant temperature as low as practical by use of alternate heat removal methods.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. NPF-43

DETROIT EDISON COMPANY

FERMI-2

DOCKET NO. 50-341

1.0 INTRODUCTION

By letter dated December 22, 1988 as supplemented May 10, 1989, the Detroit Edison Company (DECo or the licensee) requested amendment to the Technical Specifications (TS) appended to Facility Operating License No. NPF-43 for Fermi-2. The proposed amendment would revise the TS to incorporate the NRC staff positions on the inspection schedule, methods and personnel, and sample expansion for piping identified in accordance with Generic Letter (GL) 88-01.

2.0 EVALUATION

Generic Letter 88-01, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping," provided industry with the NRC staff positions on BWR piping made of austenitic stainless steel that is four inches or larger in nominal diameter or contains reactor coolant at a temperature above 200°F during power operation regardless of Code classification. The GL also applies to reactor vessel attachments and appurtenance such as jet pump instrumentation penetration assemblies and head spray and vent components. The GL requested licensees to change Section 4.0.5 of their TS to incorporate the staff positions in the GL concerning schedule, methods and personnel, and sample expansion. The GL included a sample TS. The December 22, 1988, application requested a change to TS Section 4.0.5 incorporating the above mentioned positions. The proposed change was as requested by GL 88-01 and is, therefore, acceptable.

By letter dated May 10, 1989, the licensee supplemented the December 22, 1988, application. The May 10, 1989, letter proposed a change to Section 3/4.4.8, Structural Integrity to be consistent with the change proposed to Section 4.0.5 of the TS in the December 22, 1988, submittal. Specification 3/4.4.8, Structural Integrity, also provides requirements concerning ASME Code Class 1, 2 and 3 components. Surveillance Requirement 4.4.8 includes reference to NUREG-0313, Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping, Revision 1, which was published in July 1980. Revision 2 of NUREG-0313 was developed and issued by the NRC in conjunction with GL 88-01 in January 1988.

The May 10, 1989, submittal proposes to eliminate the reference to the superseded document, NUREG-0313, Revision 1. The replacement document (NUREG-0313, Revision 2) does not in itself impose new requirements.

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NUREG-0313, Revision 2 provides the technical bases for GL 88-01. As mentioned above, the licensee is fully implementing the staff positions of GL 88-01 (in regards to structural integrity) in the change to Specification 4.0.5. Therefore, it is not necessary to replace the eliminated reference with any alternative active document since all requirements are implemented in Specification 4.0.5.

The new Surveillance Requirement 4.4.8 will read:

No requirements other than Specification 4.0.5

This wording brings Fermi-2 TS into conformance with BWR Standard TS.

In summary, the proposed change to eliminate the reference to NUREG-0313, Revision 1 from Surveillance Requirement 4.4.8 is not a safety concern because:

- ° NUREG-0313, Revision 1 has been superseded.
- ° The staff positions regarding structural integrity developed from NUREG-0313, Revision 2 and implemented in GL 88-01 are included in the proposed change to Specification 4.0.5. Further, Specification 4.0.5 is referenced in Surveillance Requirement 4.4.8.
- ° The change is in conformance with BWR Standard TS.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. We have determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents which may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: John Stang

Dated: March 26, 1990