

September 1, 1989

Docket No. 50-341

Mr. B. Ralph Sylvia  
Senior Vice President - Nuclear  
Operations  
Detroit Edison Company  
6400 North Dixie Highway  
Newport, Michigan 48166

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Dear Mr. Sylvia:

SUBJECT: AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. NPF-43:  
(TAC NO. 72755)

The Commission has issued the enclosed Amendment No. 40 to Facility Operating License No. NPF-43 for the Fermi-2 facility. This amendment consists of changes to the Plant Technical Specifications (TS) in response to your letter dated March 10, 1989.

The amendment revises the TS by removing the Traversing Incore Probe (TIP) Purge System Primary Containment Isolation Valve from the TS. The subject penetration will be sealed closed with a qualified welded cap and leak tested per 10 CFR Part 50, Appendix J during the first refueling outage.

A copy of the Safety Evaluation and Notice of Issuance are enclosed.

Sincerely,

~~Original signed by~~

John F. Stang, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 40 to NPF-43
2. Safety Evaluation
3. Notice

cc w/enclosures:  
See next page

*PM*  
LA/PD31:DRSP  
PShuttleworth  
08/10/89

*JStang*  
PM/PD31:DRSP  
JStang  
08/11/89

*J07*  
(A)B/PD31:DRSP  
LYandell J. Thomas  
08/29/89

*C.R.*  
BC/SPLB  
CMcCracken  
08/16/89

*OGC*  
OGC  
08/15/89

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555  
September 1, 1989

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Senior Vice President - Nuclear  
Operations  
Detroit Edison Company  
6400 North Dixie Highway  
Newport, Michigan 48166

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A copy of the Safety Evaluation and Notice of Issuance are enclosed.

Sincerely,

A handwritten signature in dark ink, appearing to read "John F. Stang".

John F. Stang, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

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2. Safety Evaluation
3. Notice

cc w/enclosures:  
See next page

Mr. B. Ralph Sylvia  
Detroit Edison Company

Fermi-2 Facility

cc:

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Michigan Public Service Commission  
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Nuclear Facilities and Environmental  
Monitoring Section Office  
Division of Radiological Health  
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Mr. Walt Rogers  
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Newport, Michigan 48166

Monroe County Office of Civil  
Preparedness  
963 South Raisinville  
Monroe, Michigan 48161

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DETROIT EDISON COMPANY  
WOLVERINE POWER SUPPLY COOPERATIVE, INCORPORATED  
DOCKET NO. 50-341  
FERMI-2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 40  
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Detroit Edison Company (the licensee) dated March 10, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 40, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This license amendment is effective as of its date of issuance with full implementation within 90 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*John O. Thoma*

John O. Thoma, Acting Director  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 1, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 40

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE

3/4 6-43

INSERT

3/4 6-43

TABLE 3.6.3-1 (Continued)  
PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
D. <u>Other Isolation Valves (Continued)</u>	
22. (DELETED)	
23. <u>Control Rod Drive System Insert and Withdrawal Lines</u> (r)	NA
C11-F115	
C11-F138	
24. <u>Control Rod Drive Scram Discharge Volume</u>	NA
C11-F010	
C11-F011	
C11-F180	
C11-F181	

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

TABLE NOTATIONS

(a) The following is a summary of the parameters which will automatically actuate the Primary Containment Isolation Valve Groups. The instrumentation associated with these parameters is described in Specification 3.3.2.

1. Group 1 - Main Steam System

Reactor Vessel Low Water Level - Level 1  
Main Steam Line Radiation - High  
Main Steam Line Flow - High  
Main Steam Line Tunnel Temperature - High  
Main Steam Line Pressure - Low  
Condenser Pressure - High  
Turbine Building Area Temperature - High

2. Group 2 - Reactor Water Sample System

Reactor Vessel Low Water Level - Level 2  
Drywell Pressure - High  
Main Steam Line Radiation - High

3. Group 3 - Residual Heat Removal (RHR) System

Reactor Vessel Low Water Level - Level 1  
Drywell Pressure - High

4. Group 4 - Residual Heat Removal Shutdown Cooling and Head Spray

Reactor Vessel Low Water Level - Level 3  
Reactor Vessel Pressure - High, Shutdown Cooling Interlock

5. Group 5 - Core Spray System

Reactor Vessel Low Water Level - Level 1  
Drywell Pressure - High

6. Group 6 - High Pressure Coolant Injection (HPCI) System

HPCI Steam Line Flow - High  
HPCI Steam Supply Pressure - Low  
HPCI Turbine Exhaust Diaphragm Pressure - High  
HPCI Equipment Room Temperature - High

7. Group 7 - High Pressure Coolant Injection (HPCI) Vacuum Breakers

Drywell Pressure - High with simultaneous HPCI  
Steam Supply Pressure - Low





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. NPF-43

DETROIT EDISON COMPANY

WOLVERINE POWER SUPPLY COOPERATIVE, INCORPORATED

FERMI-2

DOCKET NO. 50-341

1.0 INTRODUCTION

By letter dated March 10, 1989, the Detroit Edison Company (DECo or the licensee) requested amendment to the Technical Specifications appended to Facility Operating License No. NPF-43 for Fermi-2. The proposed amendment would revise the Technical Specification (TS) to remove the Traversing Incore Probe (TIP) Purge System Primary Containment Isolation Valve from the TS.

2.0 EVALUATION

By letter dated July 21, 1986 the NRC granted an exemption from the requirements of General Design Criterion (GDC) 56 of Appendix A to 10 CFR Part 50 for the TIP Purge System penetration (Penetration X-356) until the first refueling outage. In order to resolve future compliance with GDC 56 the licensee will relocate the TIP purge panel inside containment. Penetration X-356 will be sealed with a qualified welded cap and leak tested per 10 CFR Part 50, Appendix J. The modification will be completed during the first refueling outage.

The elimination of X-356 penetration containment isolation valves and sealing the penetration with a qualified welded cap will reduce the probability of leakage during an accident from this potential leakage path. In addition, a reduction in man-rem exposure can be expected because of lower maintenance, inspection and testing requirements of the new design.

Based on the above evaluation the staff finds the proposed changes to the TS to be acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32 and 51.35, an environmental assessment and finding of no significant impact has been prepared and published in the Federal Register on August 31, 1989 (54FR36070). Accordingly, based upon the environmental assessment, we have determined that the issuance of this amendment will not have a significant effect on the quality of the human environment.

#### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: John Stang

Date: September 1, 1989

UNITED STATES NUCLEAR REGULATORY COMMISSION  
DETROIT EDISON COMPANY  
WOLVERINE POWER SUPPLY COOPERATIVE, INCORPORATED  
DOCKET NO. 50-341  
NOTICE OF ISSUANCE OF AMENDMENT TO  
FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 40 to Facility Operating License No. NPF-43, issued to Detroit Edison Company (the licensee) which revised the Technical Specifications (TS) for operations of Fermi-2 located in Monroe County, Michigan.

The amendment is effective as of the date of issuance.

The amendment revised the TS by removing the Traversing Incore Probe (TIP) Primary Containment Isolation Valve from the TS. The subject penetration will be sealed closed with a qualified welded cap and leak tested per 10 CFR Part 50, Appendix J during the first refueling outage.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity of Hearing in connection with this action was published in the FEDERAL REGISTER on August 1, 1989 (54 FR 31750). No request for a hearing or petition for leave to intervene was filed following this notice.

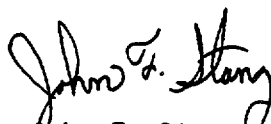
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The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of this amendment will not have a significant effect on the quality of the human environment.

For further details with respect to the action see (1) the application for amendment dated March 10, 1989, (2) Amendment No. 40 to License No. NPF-43, (3) the Commission's related Safety Evaluation, and (4) the Commission's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, N.W., Washington, D.C. and at the Local Public Document Room. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Projects III, IV, V and Special Projects.

Dated at Rockville, Maryland this 1st day of September 1989.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stang, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation