

December 3, 1993

Docket No. 50-302

Mr. Percy M. Beard, Jr.
Senior Vice President,
Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear
Licensing
Crystal River Energy Complex
15760 W Power Line Street
Crystal River, Florida 34428-6708

Dear Mr. Beard:

SUBJECT: CRYSTAL RIVER UNIT 3 - ENVIRONMENTAL ASSESSMENT AND FINDING OF NO SIGNIFICANT IMPACT (TAC NO. M74563)

Enclosed is a copy of the Environmental Assessment and Finding of No Significant Impact related to your application for amendment dated August 25, 1989. The amendment will replace the existing Technical Specifications in their entirety with the Improved Technical Specifications.

This assessment is being forwarded to the Office of the Federal Register for publication.

Sincerely,

Original signed by:

Harley Silver, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

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Mr. Percy M. Beard
Florida Power Corporation

Crystal River Unit No.3
Generating Plant

cc:

Mr. Gerald A. Williams
Corporate Counsel
Florida Power Corporation
MAC-A5A
P. O. Box 14042
St. Petersburg, Florida 33733

Mr. Joe Myers, Director
Div. of Emergency Preparedness
Department of Community Affairs
2740 Centerview Drive
Tallahassee, Florida 32399-2100

Mr. Bruce J. Hickie, Director
Nuclear Plant Operations
Florida Power Corporation
Crystal River Energy Complex
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Chairman
Board of County Commissioners
Citrus County
110 North Apopka Avenue
Inverness, Florida 32650

Mr. Robert B. Borsum
B&W Nuclear Technologies
1700 Rockville Pike, Suite 525
Rockville, Maryland 20852

Mr. Rolf C. Widell, Director
Nuclear Operations Site Support
Florida Power Corporation
Crystal River Energy Complex
15760 W Power Line Street
Crystal River, Florida 34428-6708

Regional Administrator, Region II
U. S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, Georgia 30323

Senior Resident Inspector
Crystal River Unit 3
U.S. Nuclear Regulatory
Commission
6745 N. Tallahassee Road
Crystal River, Florida 34428

Mr. Bill Passetti
Office of Radiation Control
Department of Health and
Rehabilitative Services
1317 Winewood Blvd.
Tallahassee, Florida 32399-0700

Mr. Gary Boldt
Vice President - Nuclear
Production
Florida Power Corporation
Crystal River Energy Complex
15760 W Power Line Street
Crystal River, Florida 34428-6708

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

UNITED STATES NUCLEAR REGULATORY COMMISSION
FLORIDA POWER CORPORATION CRYSTAL RIVER UNIT 3

DOCKET NO. 50-302

ENVIRONMENTAL ASSESSMENT AND FINDING OF NO SIGNIFICANT IMPACT

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-72, issued to Florida Power Corporation (FPC, the licensee), for operation of Crystal River, Unit 3, (CR-3) located in Citrus County, Florida.

ENVIRONMENTAL ASSESSMENT

Identification of the Proposed Action:

The proposed amendment will replace the existing Technical Specifications (TS) in their entirety with the Improved Technical Specifications (ITS). The proposed action is in accordance with the licensee's amendment request dated August 25, 1989.

The Need for the Proposed Action:

It has been recognized that nuclear safety in all plants would benefit from improvement and standardization of TS. The "NRC Interim Policy Statement on Technical Specification Improvements for Nuclear Power Reactors," (FEDERAL REGISTER 52 FR 3788, February 6, 1987) and later the Final Policy Statement, formalized this need. To facilitate the development of individual ITS, each reactor vendor owners group (OG) and the NRC staff developed standard Technical Specifications. For Babcock & Wilcox (B&W) plants, the standard TS (STS) is NUREG-1430, which formed the basis of the CR-3 ITS. The NRC Committee to Review Generic Requirements (CRGR) reviewed the STS and made note

of the safety merits of the STS and indicated its support of conversion by operating plants to the STS.

Description of the Proposed Change:

The proposed revision to the TS is based on NUREG-1430 and on guidance provided in the Policy Statement. Its objective is to completely rewrite, reformat, and streamline the existing TS. Emphasis is placed on human factors principles to improve clarity and understanding. The Bases section has been significantly expanded to clarify and better explain the purpose and foundation of each specification. In addition to NUREG-1430, portions of the existing TS were also used as the basis for the ITS. Plant-specific issues (unique design features, requirements, and operating practices) were discussed at length with the licensee, and generic matters with the licensee and the B&W and other OGs.

The proposed changes from the existing TS can be grouped into four general categories, as follows:

Non-Technical (administrative) changes were intended to make the ITS easier to use for plant operations personnel. They are purely editorial in nature or involve the movement or reformat of requirements without affecting technical content. Every section of the CR-3 TS has undergone these types of changes. In order to ensure consistency, the NRC staff and FPC have used NRC-1430 as guidance to reformat and make other administrative changes.

Relocation of requirements includes items that were in the existing CR-3 TS, but did not meet the criteria set forth in the Policy Statement for inclusion in TS. In general, the proposed relocation of items in the CR-3 TS to the Final Safety Analysis Report (FSAR), appropriate plant-specific programs,

procedures, and ITS Bases follows the guidance of the B&W STS, NUREG-1430. Once these items have been relocated by removing them from the TS to other licensee-controlled documents, the licensee may revise them under the provisions of 10 CFR 50.59 or other NRC staff-approved control mechanisms which provide appropriate procedural means to control changes.

More restrictive requirements are those proposed CR-3 ITS items that are either more conservative than corresponding requirements in the existing CR-3 TS, or are additional restrictions which are not in the existing CR-3 TS, but are contained in NUREG-1430. Examples of more restrictive requirements include: placing a Limiting Condition of Operation (LCO) on plant equipment which is not required by the present TS to be operable; more restrictive requirements to restore inoperable equipment; and more restrictive surveillance requirements.

Less restrictive requirements are relaxations of corresponding requirements in the existing CR-3 TS which provided little or no safety benefit or placed unnecessary burden on the licensee. These relaxations were the result of generic NRC action or other analyses. They have been justified on a case-by-case basis for CR-3 as described in the Safety Evaluation to be issued with the license amendment, which will be noticed in the Federal Register.

Examples of such requirements are: modification of the Axial Power Imbalance TS to be consistent with NUREG-1430 and the current staff position; and modification of action requirements on quadrant power tilt to achieve consistency between the FSAR and the TS.

Environmental Impacts of the Proposed Action:

The Commission has completed its evaluation of the proposed revision to the TS. Changes which are administrative in nature have been found to have no effect on technical content of the TS, and are acceptable. The increased clarity and understanding these changes bring to the TS are expected to improve the operator's control of the plant in normal and accident conditions.

Relocation of requirements to other licensee-controlled documents does not change the requirements themselves. Future changes to these requirements may be made by the licensee under 10 CFR 50.59 or other NRC-approved control mechanisms, which assure continued maintenance of adequate requirements. All such relocations have been found to be in conformance with the guidelines of NUREG-1430 and the Policy Statement, and, therefore, to be acceptable.

Changes involving more restrictive requirements have been found to be acceptable.

Changes involving less restrictive requirements have been reviewed individually. When requirements have been shown to provide little or no safety benefit or to place unnecessary burden on the licensee, their removal from the TS was justified. In most cases, relaxations previously granted to individual plants on a plant-specific basis were the result of a generic NRC action, or of agreements reached during discussions with the OG and found to be acceptable for CR-3. Generic relaxations contained in NUREG-1430 have also been reviewed by the NRC staff and have been found to be acceptable.

In summary, the proposed revision to the TS was found to provide control of plant operations such that reasonable assurance will be provided that the health and safety of the public will be adequately protected.

These TS changes will not increase the probability or consequences of accidents, no changes are being made in the types of any effluent that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Therefore, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed TS amendment.

With regard to potential non-radiological impacts, the proposed amendment involves features located entirely within the restricted areas as defined in 10 CFR Part 20. It does not affect non-radiological plant effluent and has no other environmental impact. Therefore, the Commission concludes that there are no significant non-radiological environmental impacts associated with the proposed amendment.

Alternatives to the Proposed Action:

Since the Commission has concluded there is no measurable environmental impact associated with the proposed amendment, any alternatives with equal or greater environmental impact need not be evaluated. The principal alternative to the amendment would be to deny the amendment request. Such action would not enhance the protection of the environment.

Alternative Use of Resources:

This action does not involve the use of resources not considered previously in the Final Environmental Statement for Crystal River, Unit 3.

Agencies and Persons Consulted:

The NRC staff consulted with the State of Florida regarding the environmental impact of the proposed action.

FINDING OF NO SIGNIFICANT IMPACT

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed amendment.

For further details with respect to this proposed action, see the licensee's letter dated August 25, 1989. This letter is available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC 20555, and at the local public document room located at the Coastal Region Library, 8619 W. Crystal Street, Crystal River, Florida 32629.

Dated at Rockville, Maryland this 3rd day of December 1993.

FOR THE NUCLEAR REGULATORY COMMISSION

Bart C. Buckley

Bart C. Buckley, Acting Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation