

September 30, 1991

Docket No. 50-302

DISTRIBUTION
See attached sheet

Mr. Percy M. Beard, Jr.
Senior Vice President,
Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Operations
Licensing
P. O. Box 219-NA-2I
Crystal River, Florida 32629

Dear Mr. Beard:

SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF AMENDMENT RE: CONTAINMENT
PENETRATION AND CONTAINMENT ISOLATION VALVE LOCAL LEAK RATE
TESTING (TAC NO. 80865)

The Commission has issued the enclosed Amendment No. 137 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TS) in response to your application dated June 20, 1991.

This amendment revises TS Section 4.6.1.2 to permit a one-time extension of the surveillance interval for Type B and C tests until the eighth refueling outage, currently scheduled to begin April 30, 1992.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

This completes our efforts on TAC No. 80865.

Sincerely,
(Original Signed By)

Harley Silver, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 137 to DPR-72
- 2. Safety Evaluation

cc w/enclosures:
See next page

LA:PDII-2
DMiller
9/16/91

INA:PDII-2
FTalbot
9/16/91

PN:PDII-2
HSilver:kdj
9/16/91

D:PDII-2
HBerkow
for 9/26/91

OGC
E.HOLLON
9/17/91

BC: SPLB
C.A.
9/24/91

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PDR

NRC FILE CENTER COPY

Mr. Percy M. Beard, Jr.
Florida Power Corporation

Crystal River Unit No. 3 Nuclear
Generating Plant

cc:

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Department of Environmental Regulation
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Tallahassee, Florida 32301

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

DATED: September 30, 1991

AMENDMENT NO. 137 TO FACILITY OPERATING LICENSE NO. DPR-72-CRYSTAL RIVER UNIT 3

Docket File

NRC & Local PDRs

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J. Miller, 11/F/23

ACRS (10)

GPA/PA

OC/LFMB

M. Sinkule, R-II

cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 137
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated June 20, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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P PDR

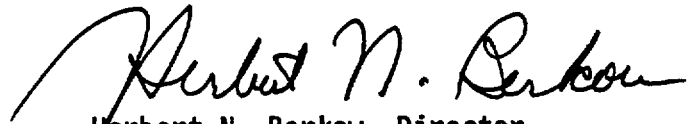
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 137, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 30, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 137

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove
3/4 6-3

Insert
3/4 6-3

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- b. If any periodic Type A test fails to meet $.75 L_a$ the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet $.75 L_a$ a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet $.75 L_a$ at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
 - 1. Confirms the accuracy of the Type A test by verifying that the difference between supplemental and Type A test data is within $0.25 L_a$.
 - 2. Had a duration sufficient to establish accurately the change in leakage between the Type A test and the supplemental test.
 - 3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage rate at $\geq P_a$, 53.3 psig.
- d. Type B and C tests shall be conducted with gas at $\geq P_a$, 53.3 psig at intervals no greater than 24 *months except for tests involving:
 - 1. Air locks, and
 - 2. Penetrations using continuous leakage monitoring systems.
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.

*The 24 month interval may be extended until the end of Refuel 8 currently scheduled to start in April 30, 1992.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- f. Deleted.
- g. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- f. Deleted.
- g. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 137 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET AL.
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

1.0 INTRODUCTION

By letter dated June 20, 1991, Florida Power Corporation (FPC, the licensee) requested an amendment to Facility Operating License No. DPR-72 for the Crystal River Unit 3 Nuclear Generating Station (CR-3). The proposed amendment would extend the interval for Type B and C local leak rate testing (LLRT) on a one-time basis for containment penetrations and containment isolation valves (CIVs) until the eighth refueling outage, currently scheduled to begin April 30, 1992. The LLRTs for these containment penetrations and valves would begin to become overdue in March 1992. By a separate letter dated January 31, 1991, as supplemented on May 16, 1991, the licensee also requested a one-time exemption from the schedular requirements of Sections III.D.2(a) and III.D.3 of Appendix J to 10 CFR Part 50 concerning LLRT testing intervals for 114 CIVs and 49 containment penetrations. The exemption would defer testing of the valves and penetrations until the eighth refueling outage.

2.0 DISCUSSION

Technical Specification (TS) 4.6.1.2.d and Sections III.D.2(a) and III.D.3 of 10 CFR Part 50, Appendix J require that Type B and C tests be performed at intervals no greater than 24 months. Historically, FPC Type B and C LLRT leakage has not been a source of significant "as-found" leakage as verified by the staff's review of FPC's 1989/1990 leak rate test results of containment penetrations and CIVs.

The TS change and exemption to Appendix J are necessary to avoid an otherwise unnecessary extension of the midcycle shutdown and to allow for leak rate tests to be performed during the eighth refueling outage (Refuel 8), scheduled to begin April 30, 1992. At that time, the containment penetrations and CIVs would exceed the 2-year testing interval requirement by approximately 2 months. Therefore, FPC requests a footnote change to the TS and an exemption to allow for this extension for leak rate testing of containment penetrations and CIVs.

3.0 EVALUATION

TS 4.6.1.2(d) requires LLRTs (Type B and C) on primary CIVs and containment penetrations to be performed at intervals no greater than 24 months. The Commission's regulations (10 CFR Part 50, Appendix J, Sections III.D.2(a) and III.D.3) require LLRTs (Type B and C tests) to be performed during each reactor shutdown for refueling, but in no case at intervals greater than 2 years. The licensee has requested that the 24-month testing interval for electrical penetrations and 114 CIVs and their associated containment penetrations identified in FPC letter dated May 16, 1991 be extended on a one-time basis until the eighth refueling outage, presently scheduled to begin April 30, 1992. These valves would otherwise become overdue for testing between March 1992 and May 1992.

The staff reviewed the 1989/1990 leak rate testing results of CIVs and their containment penetrations at CR-3 submitted with the licensee's letter of May 16, 1991. The results of this review indicate that the "as found" and "as left" leak rate conditions of CIVs and containment penetrations were significantly below the leak rate limit acceptance criteria. The review of leak rate test results also indicated that FPC performed preventive maintenance to further reduce the leakage rate of CIVs and containment penetrations in the "as left" condition.

The 24-month interval requirement for Type B and C penetrations is intended to be often enough to prevent significant deterioration from occurring and long enough to permit LLRTs to be performed during plant outages. The requested one-time extension would add approximately 2 months to the testing interval for penetrations and CIVs. The condition of the components is not expected to change significantly during this short extension period. Testing prior to the refueling outage would require an additional outage for this purpose only, or significant extension of the mid-cycle maintenance outage.

4.0 SUMMARY

Based on the short one-time extension requested, the previous satisfactory leak rate test results, and the small likelihood of significant degradation during the extension period, the NRC staff concludes that extending the surveillance testing interval for Type B and C tests until Refuel 8, as requested, provides reasonable assurance that the proposed changes will present no undue risk to public health and safety, and is, therefore, acceptable.

5.0 STATE CONSULTATION

Based upon the written notice of the proposed amendment, the Florida State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

This amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has

been no public comment on such finding (56 FR 33956). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: F. Talbot
H. Silver

Date: September 30, 1991