

June 8, 1998

Mr. John Paul Cowan, Vice President
Nuclear Operations (NA2E)
Florida Power Corporation
Crystal River Energy Complex
15760 W. Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 - STAFF EVALUATION AND ISSUANCE OF
AMENDMENT RE: REACTOR COOLANT PUMP FLYWHEEL INSPECTION
SCHEDULE (TAC NO. MA1494)

Dear Mr. Cowan:

The Commission has issued the enclosed Amendment No. 167 to Facility Operating License No. DPR-72 for the Crystal River Unit 3. This amendment is in response to your request dated March 20, 1998, in which you proposed to revise Improved Technical Specification 5.6.2.8 to reflect the current schedule for performing the required reactor coolant pump flywheel inspection.

The amendment approves the requested change to reflect the current refueling outage schedule. A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Leonard A. Wiens, Senior Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures: 1. Amendment No.167 to DPR-72
2. Safety Evaluation

cc w/enclosures: See next page

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Mr. John Paul Cowan
Florida Power Corporation

CRYSTAL RIVER UNIT NO. 3

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION.
CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 167
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated March 20, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 167, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Frederick J. Hebdon

Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Date of Issuance: June 8, 1998

ATTACHMENT TO LICENSE AMENDMENT NO.167

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Revise the Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page is identified by the captioned amendment number and contains marginal lines indicating the area of change.

REMOVE

INSERT

5.0-11

5.0-11

5.6 Procedures, Programs and Manuals

5.6.2.6 Post Accident Sampling (continued)

- c. Provisions for maintenance of sampling and analysis equipment.

5.6.2.7 Containment Tendon Surveillance Program

This program provides controls for monitoring any tendon degradation in concrete containments, including effectiveness of its corrosion protection medium, to ensure containment structural integrity. The program shall include baseline measurements prior to initial operations. The Containment Tendon Surveillance Program, inspection frequencies, and acceptance criteria shall be in accordance with Regulatory Guide 1.35, Revision 3, 1990.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Containment Tendon Surveillance Program inspection frequencies.

5.6.2.8 Inservice Inspection Program

This program provides controls for inservice inspection of ASME Code Class 1, 2, and 3 components, including applicable supports. The program shall include the following:

- a. Provisions that inservice inspection of ASME Code Class 1, 2, and 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50.55a;
- b. The provisions of SR 3.0.2 are applicable to the frequencies for performing inservice inspection activities;
- c. Inspection of each reactor coolant pump flywheel per the recommendation of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975, except, the volumetric examinations and complete surface examination of the flywheels for the Second ISI Interval may be delayed one cycle to coincide with Refueling Outage 11R; and
- d. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any TS.

(continued)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 167 TO FACILITY OPERATING LICENSE NO. DPR-72

FLORIDA POWER CORPORATION

CRYSTAL RIVER UNIT 3

DOCKET NO. 50-302

1.0 INTRODUCTION

The Inservice Inspection Program described in Improved Technical Specification (ITS) 5.6.2.8 requires inspection of the reactor coolant pump (RCP) motor flywheels in accordance with the recommendations of Regulatory Guide 1.14, Revision 1, dated August 1975. Amendment No. 153 to the Crystal River Unit 3 (CR-3) Operating License, issued February 15, 1996, added a one-time deferral of the RCP flywheels inspection until "the Spring 1998 refueling outage," which at that time was the scheduled date for Refueling Outage 11.

Shortly after issuance of Amendment No. 153, CR-3 entered into a 17-month extended, non-refueling outage. As a result of this extended outage, Florida Power Corporation (FPC) deferred Refueling Outage 11 until Fall 1999. By letter dated March 20, 1998, FPC submitted a request to replace the wording "Spring 1998 refueling outage" with "Refueling Outage 11R." Additionally, this request corrects typographical errors in this ITS.

2.0 EVALUATION

In November of 1995, CR-3 submitted a Technical Specification Change Request requesting that the reactor coolant pump flywheel inspections for the second 10-year interval (which were scheduled to be performed during the CR-3 Spring 1996 refueling outage) be postponed until the Spring 1998 refueling outage. This postponement was based upon the number and results of previous CR-3 flywheel inspections, as well as the low probability of flywheel failure as demonstrated by industry experience. In its safety evaluation, the U.S. Nuclear Regulatory Commission staff concluded, based on the favorable results from previous volumetric examinations and the low probability of flywheel failure based on industry records, that the inspection could be deferred one operating cycle (from refueling outage 10 until refueling outage 11) without affecting the structural integrity of the flywheels or increase the failure probability of the flywheels significantly.

Because of the extended maintenance outage, CR-3 is still in the operating cycle which will conclude with refueling outage 11 in the Fall 1999. During the maintenance outage, the RCPs accumulated no significant operating time, since the plant was in cold shutdown conditions for

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nearly the entire outage. Therefore, the basis for deferring the RCP flywheel inspection until refueling outage 11 remains valid, and the staff finds this change acceptable.

3.0 STATE CONSULTATION

Based upon written notice of the proposed amendment, the Florida State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATIONS

The amendment changes requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (63 FR 25110). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

Based on its evaluation, the staff concludes that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Len Wiens

Dated: June 8, 1998