

July 20, 1998

Mr. John Paul Cowan, Vice President
Nuclear Operations (NA2E)
Florida Power Corporation
Crystal River Energy Complex
15760 W. Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 - STAFF EVALUATION AND ISSUANCE OF
AMENDMENT RE: ORGANIZATIONAL TITLE CHANGES (TAC NO. MA1493)

Dear Mr. Cowan:

The Commission has issued the enclosed Amendment No. 168 to Facility Operating License No. DPR-72 for the Crystal River Unit 3. This amendment is in response to your request dated March 20, 1998, and revised by letter dated May 22, 1998, in which you proposed to revise Improved Technical Specification Safety Limits and Administrative Controls to replace the titles of the Senior Vice President, Nuclear Operations and the Vice President, Nuclear Production with the position of Chief Nuclear Officer.

The amendment approves the requested change to reflect the current refueling outage schedule. A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Leonard A. Wiens, Senior Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures: 1. Amendment No. 168 to DPR-72
2. Safety Evaluation

cc w/enclosures: See next page

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Mr. John Paul Cowan
Florida Power Corporation

CRYSTAL RIVER UNIT NO. 3

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION,
CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 168
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated March 20, 1998, as supplemented May 22, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 168 , are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY
COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 20, 1998

ATTACHMENT TO LICENSE AMENDMENT NO.168

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

2.0-2

2.0-2

B2.0-11

B2.0-11

5.0-2

5.0-2

2.0 SLs

2.2 SL Violations (continued)

- 2.2.3 In MODES 3, 4, and 5, if SL 2.1.2 is violated, restore compliance within 5 minutes.
- 2.2.4 With any SL violation, within 1 hour, notify the NRC Operations Center, in accordance with 10 CFR 50.72.
- 2.2.5 With any SL violation, within 24 hours, notify the appropriate Nuclear Operations senior management and the offsite reviewers specified in the Quality Assurance Plan.
- 2.2.6 With any SL violation, within 30 days, a Licensee Event Report (LER) shall be prepared pursuant to 10 CFR 50.73. The LER shall be submitted to the NRC, the offsite reviewers specified in the Quality Assurance Plan, the Director, Nuclear Plant Operations and the Chief Nuclear Officer.
- 2.2.7 With any SL violation, operation of the plant shall not be resumed until authorized by the NRC.
-
-

BASES

SAFETY LIMIT
VIOLATIONS
(continued)

2.2.6

If the RCS pressure SL is violated, a Licensee Event Report shall be prepared and submitted within 30 days to the NRC in accordance with 10 CFR 50.73 (Ref. 9). A copy of the report shall also be provided to the NGRC, the Director, Nuclear Plant Operations, and the Chief Nuclear Officer.

The 10 CFR 50.73 part against which a Safety Limit violation would be reported is: 1) completion of a plant shutdown required by Technical Specifications, (10 CFR 50.73(a)(2)(i)(A)), 2) an event which resulted in an unanalyzed condition that significantly compromised plant safety, (10 CFR 50.73(a)(2)(iv)).

2.2.7

If the RCS pressure SL is violated, operation of the plant shall not be resumed until authorized by the NRC. This requirement ensures the NRC that all necessary reviews, analyses, and actions are completed by establishing limitations on ascending MODES or other specified conditions in the Applicability until the NRC review is complete.

REFERENCES

1. FSAR, Section 1.4.
2. ASME Boiler and Pressure Vessel Code, Section III, Article NB-7000.
3. ASME Boiler and Pressure Vessel Code, Section XI, Articles IWA-5000 and IWB-5000.
4. BAW-10043, May 1972.
5. FSAR, Section 14.
6. ASME USAS B31.7, Code for Pressure Piping, Nuclear Power Piping, February 1968 Draft Edition.

(continued)

5.0 ADMINISTRATIVE CONTROLS

5.2 Organization

5.2.1 Onsite and Offsite Organizations

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions responsible for activities affecting safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communications shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of department responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These shall be documented in the FSAR;
- b. The Chief Nuclear Officer shall be an officer of the company and shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety. The Chief Nuclear Officer shall be responsible for the overall safe operation of the plant and shall have control over those onsite activities necessary for the safe operation and maintenance of the plant; and
- c. The individuals who train the operating staff, carry out health physics or perform quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

5.2.2 Unit Staff

The unit staff organization shall include the following:

- a. One auxiliary nuclear operator shall be assigned to the operating shift any time there is fuel in the reactor and

(continued)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 168 TO FACILITY OPERATING LICENSE NO. DPR-72

FLORIDA POWER CORPORATION

CRYSTAL RIVER UNIT 3

DOCKET NO. 50-302

1.0 INTRODUCTION

By letter dated March 20, 1998, and revised by letter dated May 22, 1998, Florida Power Corporation (FPC), the licensee for Crystal River Unit 3 (CR-3) requested an amendment to Facility Operating License No. DPR-72. The licensee proposed editorial changes to the Improved Technical Specifications (ITS) Safety Limits and Administrative Controls to replace the titles of the Senior Vice President, Nuclear Operations (SVPNO) and the Vice President, Nuclear Production (VPNP) with the position of Chief Nuclear Officer (CNO). The CNO combines the duties of the SVPNO and VPNP as currently described in ITS. The May 22, 1998, revision did not affect the original no significant hazards determination.

2.0 EVALUATION

The current ITS 2.2.5 requires that the Senior Vice President, Nuclear Operations be notified of any safety limit violation within 24 hours. The proposed change would revise the requirement to read that appropriate Nuclear Operations senior management be notified of safety violations within 24 hours. The proposed change would make the ITS consistent with the associated ITS Bases statement. Since senior licensee management officials would continue to be notified in a timely fashion of safety limit violations, the staff finds this change acceptable.

ITS 2.2.6 currently requires a copy of any Licensee Event Report (LER) prepared as a result of a safety limit violation be provided to the Senior Vice President, Nuclear Operations. The proposed revision of ITS 2.2.6 would require that copies of LERs prepared for safety limit violations be provided to the Chief Nuclear Officer instead of the Senior Vice President, Nuclear Operations. Since the Chief Nuclear Officer combines the duties formerly assigned to the positions of the Senior Vice President, Nuclear Operations and the Vice President, Nuclear Production, an appropriate level of licensee management would continue to receive copies of such LERs. Therefore this change is acceptable.

Since the Chief Nuclear Officer combines the duties formerly assigned to the positions of the Senior Vice President, Nuclear Operations and the Vice President, Nuclear Production, the title "Senior Vice President, Nuclear Operations" in ITS 5.2.1.b. is changed to "Chief Nuclear

Officer." Current ITS 5.2.1.c., which specifies responsibilities of the Vice President, Nuclear Production, for site operations, is proposed to be deleted and the description of these duties is combined with ITS 5.2.1.b. Since a senior licensee manager would continue to be assigned the responsibilities for overall safe operation of the plant and would have control over those corporate and site activities necessary for safe operation and maintenance of the plant, these changes are acceptable.

3.0 STATE CONSULTATION

Based upon written notice of the proposed amendment, the Florida State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATIONS

These amendments relate to changes in record keeping or administrative procedures or requirements. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (63 FR 25109). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

Based on the review of the licensee's submittal, the staff finds that the proposed ITS changes due to the change of the titles of the licensee management for the Crystal River Unit 3 does not alter the intent, meaning or substance of the ITS. Therefore, the staff finds the proposed ITS changes acceptable.

The Commission has concluded, based on considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Len Wiens

Dated: July 20, 1998