

Mr. John Paul Cowan
 Vice President, Nuclear Operations
 Florida Power Corporation
 ATTN: Manager, Nuclear Licensing (NA1B)
 Crystal River Energy Complex
 15760 W. Power Line Street
 Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF AMENDMENT REGARDING POST-ACCIDENT MONITORING INSTRUMENTATION (TAC NO. MA3612)

Dear Mr. Cowan:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. to Facility Operating License No. DPR-72 for Crystal River Unit 3. This amendment is in response to a Florida Power Company (FPC) request dated August 31, 1998, in which FPC proposed to add additional instrumentation variables to Improved Technical Specification Table 3.3.17-1, Post-Accident Monitoring Instrumentation.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

L. Wiens, Senior Project Manager, Section 2
 Project Directorate II
 Division of Licensing Project Management
 Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures: 1. Amendment No. to DPR-72
 2. Safety Evaluation

NRC FILE CENTER COPY

cc w/enclosures: See next page

Distribution:	ACRS	CNorsworthy(EMail SE)	LWert, RII
Docket File	BClayton	LWiens	SPeterson
PUBLIC	ABryant	GHill (2)	HBerkow
CR-3 r/f	OGC	WBeckner	

DOCUMENT NAME: G:\CRYSTAL\AMDA3612.WPD:

To receive a copy of this document, indicate in the box: "C" = Copy without attachment/enclosure "E" = Copy with attachment/enclosure "N" = No copy

OFFICE	PDII-2/PM	<input checked="" type="checkbox"/>	PDII-2/LA	<input checked="" type="checkbox"/>	OGC	<input checked="" type="checkbox"/>	PDII-2/SC	<input checked="" type="checkbox"/>	PDII/D	<input checked="" type="checkbox"/>
NAME	LWiens		BClayton		R Bachmann		SPeterson		HBerkow	
DATE	05/11/99		05/11/99		05/11/99		05/21/99		04/1/99	5/21/99

OFFICIAL RECORD COPY

9906080224 990603
 PDR ADOCK 05000302
 P PDR

DF011



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 3, 1999

Mr. John Paul Cowan
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing (NA1B)
Crystal River Energy Complex
15760 W. Power Line Street
Crystal River, Florida 34428-6708


SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF AMENDMENT REGARDING POST-ACCIDENT MONITORING INSTRUMENTATION (TAC NO. MA3612)

Dear Mr. Cowan:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 177 to Facility Operating License No. DPR-72 for Crystal River Unit 3. This amendment is in response to a Florida Power Company (FPC) request dated August 31, 1998, in which FPC proposed to add additional instrumentation variables to Improved Technical Specification Table 3.3.17-1, Post-Accident Monitoring Instrumentation. As discussed with your nuclear licensing staff on May 27, 1999, the amendment implementation was revised to prior to commencing cycle 12 operation.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,


for L. Wiens, Senior Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures: 1. Amendment No. 177 to DPR-72
2. Safety Evaluation

cc w/enclosures: See next page



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION,
CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 177
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees), dated August 31, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

9906080229 990603
PDR ADOCK 05000302
P PDR

- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 177, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented prior to commencing cycle 12 operation.

FOR THE NUCLEAR REGULATORY COMMISSION



Sheri R. Peterson, Chief, Section 2
Project Directorate II
Division of Project Licensing Management
Office of Nuclear Reactor Regulation

Date of Issuance: June 3, 199

ATTACHMENT TO LICENSE AMENDMENT NO. 177

TO FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contains vertical lines indicating the area of change.

Remove Page

3.3-40
3.3-41
B 3.3-125B

B 3.3-143
B 3.3-144

Insert Page

3.3-40
3.3-41
B 3.3-125B
B 3.3-138C
B 3.3-138D
B 3.3-143
B 3.3-144

SURVEILLANCE REQUIREMENTS

-----NOTE-----
 These SRs apply to each PAM instrumentation Function in Table 3.3.17-1.

SURVEILLANCE	FREQUENCY
SR 3.3.17.1 -----NOTE----- Not required for Function 4. ----- Perform CHANNEL CHECK for each required instrumentation channel that is normally energized.	31 days
SR 3.3.17.2 -----NOTE----- Neutron detectors are excluded from CHANNEL CALIBRATION. ----- -----NOTE----- Not required for Functions 23 and 25. ----- Perform CHANNEL CALIBRATION.	-----NOTE----- The Frequency for Function 12 is 18 months. ----- 24 months
SR 3.3.17.3 -----NOTE----- Only required for Functions 23 and 25. ----- Perform CHANNEL FUNCTIONAL TEST.	24 months

Table 3.3.17-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

FUNCTION	REQUIRED CHANNELS	CONDITIONS REFERENCED FROM REQUIRED ACTION D.1
1. Wide Range Neutron Flux	2	E
2. RCS Hot Leg Temperature	2	E
3. RCS Pressure (Wide Range)	2	E
4. Reactor Coolant Inventory	2	F
5. Borated Water Storage Tank Level	2	E
6. High Pressure Injection Flow	2 per injection line	E
7. Containment Sump Water Level (Flood Level)	2	E
8. Containment Pressure (Expected Post-Accident Range)	2	E
9. Containment Pressure (Wide Range)	2	E
10. Containment Isolation Valve Position	2 per penetration (a)(b)	E
11. Containment Area Radiation (High Range)	2	F
12. Containment Hydrogen Concentration	2	E
13. Pressurizer Level	2	E
14. Steam Generator Water Level (Start-up Range)	2 per OTSG	E
15. Steam Generator Water Level (Operating Range)	2 per OTSG	E
16. Steam Generator Pressure	2 per OTSG	E
17. Emergency Feedwater Tank Level	2	E
18a. Core Exit Temperature (Thermocouple)	2 thermocouples per core quadrant	E
18b. Core Exit Temperature (Recorder)	2	E
19. Emergency Feedwater Flow	2 per OTSG	E
20. Low Pressure Injection Flow	2	E
21. Degrees of Subcooling	2	E
22. Emergency Diesel Generator kW Indication	2(c)	E
23. LPI Pump Run Status	2	E
24. DHV-42 and DHV-43 Open Position	2	E
25. HPI Pump Run Status	2	E
26. RCS Pressure (Low Range)	2	E

(a) Only one position indication is required for penetrations with one Control Room indicator.

(b) Not required for isolation valves whose associated penetration is isolated by at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(c) One indicator per EDG.

BASES

FUNCTION	CHANNEL A	CHANNEL B
15. Steam Generator Water Level (Operating Range)	OTSG A: SP-17-LI1 or SP-17-LIR OTSG B: SP-21-LI1 or SP-21-LIR	OTSG A: SP-18-LI1 OTSG B: SP-22-LI1
16. Steam Generator Pressure	OTSG A: MS-106-PI1 or MS-106-PIR, OTSG B: MS-110-PI1 or MS-110-PIR	OTSG A: MS-107-PI1 or MS-107-PIR OTSG B: MS-111-PI1 or MS-111-PIR
17. Emergency Feedwater Tank Level	EF-98-LI1	EF-99-LI1
18. Core Exit Temperature (Backup)	Three detectors from each of the following groups: Quadrant WX: IM-2G-TE /IM-5G-TE /IM-6C-TE /IM-7F-TE Quadrant XY: IM-9E-TE /IM-10C-TE/IM-11G-TE/IM-13G-TE Quadrant YZ: IM-9H-TE /IM-10M-TE/IM-100-TE/IM-13L-TE Quadrant ZM: IM-3L-TE /IM-4N-TE /IM-6L-TE /IM-6O-TE and Recorders RC-171-TR, RC-172-TR, RC-173-TR	
19. Emergency Feedwater Flow	OTSG A: EF-25-FI1 OTSG B: EF-23-FI1	OTSG A: EF-26-FI1 OTSG B: EF-24-FI1
20. Low Pressure Injection Flow	DHV-110 Hand/Auto station flow indication (DH-1-FK3-1)	DHV-111 Hand/Auto station flow indication (DH-1-FK4-1)
21. Degrees of Subcooling	RC-4-TI4 and	RC-4-TI5 and
	SPDS "A" or SPDS "B"	
22. Emergency Diesel Generator kW Indication	EGDG-1A Wattmeter SSF-AH Main control board indicator	EGDG-1B Wattmeter SSF-AX Main control board indicator
23. LPI Pump Run Status	ESFA-LX3 (Red Light) or ESFA-HU (ES Light Matrix "A")	ESFB-LX3 (Red Light) or ESFB-HU (ES Light Matrix "B")
24. DHV-42 and DHV-43 Open Position	ESFA-KN3 (Red Light)	ESFB-KN3 (Red Light)
25. HPI Pump Run Status	HPI Pump 1A: ESFA-MF7 (Red Light) or ESFA-AH (ES Light Matrix "A") OR HPI Pump 1B: ESFA-MN7 (Red Light) or ESFA-AJ (ES Light Matrix "A")	HPI Pump 1C: ESFB-MF7 (Red Light) or ESFB-AH (ES Light Matrix "B") OR HPI Pump 1B: ESFB-MV7 (Red Light) or ESFB-AJ (ES Light Matrix "B")
26. RCS Pressure (Low Range)	RC-147-PI1	RC-148-PI1

NOTES: For Function 18, OPERABILITY of only two detectors (and associated recorder) for any group constitutes entry into Condition A of LCO 3.3.17. Any group with only one OPERABLE detector/recorder combination constitutes entry into Condition C of LCO 3.3.17. Separate Condition entry is allowed for each group.

For Function 21, with both channels of SPDS inoperable, LCO Condition C and its associated Required Action are applicable.

For Function 25, OPERABILITY of indication is required only for the one ES selected HPI pump in each channel.

(continued)

BASES

LCO
(continued)

23. LPI Pump Run Status

LPI Pump Run Status is used by the operator during the ECCS Pump Suction Transfer to initiate the opening of DHV-11 and DHV-12, the "Piggyback" valves.

LPI Pump Run Status is a Type A variable because the opening of these valves to establish a suction source is a planned manual action and is necessary to maintain HPI in LOCAs where the RCS pressure has not decreased to the point that LPI will provide injection of coolant into the Reactor Coolant System and when single failures occur which result in only one train of LPI being available.

24. DHV-42 and DHV-43 Open Position

DHV-42 and DHV-43 Open Position, the suction valves from the RB sump to the LPI pumps, are used by the operator during the ECCS Pump Suction Transfer to initiate the closing of DHV-34 and DHV-35, the suction valves from the BWST to the LPI pumps.

DHV-42 and DHV-43 Open Position are Type A variables because the closing of DHV-34 and DHV-35 to manually isolate the BWST from the LPI and BS suction is necessary to prevent vortexing and consequential damage to the LPI and BS pumps, which are required for maintaining core and containment cooling.

25. HPI Pump Run Status

HPI Pump Run Status is used by the operator in determining the total HPI flow per pump. If the total flow decreases below the minimum pump flow limits, then a manual action is taken to open the recirculation valves in order to avoid low flow operation which could damage the pump and render it inoperable while it is being used to mitigate a design basis accident.

HPI Pump Run Status is a Type A variable because there are some SBLOCAs that would result in HPI flows of this small a magnitude. Since these actions are necessary to protect equipment that is necessary to mitigate the accident, these are Type A variables.

(continued)

BASES

LCO
(continued)

26. RCS Pressure (Low Range)

RCS pressure is measured by pressure transmitters with a span of 0-600 psig. Redundant monitoring capability is provided by two trains of instrumentation. The control room indications are the primary indications used by the operator during an accident. Therefore, the LCO deals specifically with this portion of the instrument string.

RCS Pressure (Low Range) is classified as a Type B Category 1 variable because none of the actions taken by the operator based on RCS pressure variables qualifies it as a Type A variable. It is included in Table 3.3.17-1 as it is a Category 1 variable and relied on by the operators to maintain RCS pressure for several contingency actions included in the EOPs. Those actions are: 1) determination that an ES actuation of the LPI pumps should have occurred, 2) selecting Low Temperature Overpressure Protection (LTOP), 3) opening of drop lines to establish Decay Heat Removal, 4) ensure adequate RCP NPSH at low RCS pressure, and 5) ensure NDT limits have not been exceeded.

(continued)

BASES

SURVEILLANCE
REQUIREMENTS

SR 3.3.17.1 (continued)

A note to the Surveillance excludes the performance of a CHANNEL CHECK on Function 4. FPC requested, and was granted, exception from performing a CHANNEL CHECK on this instrumentation as part of Amendment 124, dated October 17, 1989. The basis for not performing this SR is based on the design of the system. The system utilizes differential pressure (dp) measurements across vertical elevations of the hot leg and the reactor vessel when the RCPs are tripped. Performance of the SR with the RCPs in operation provides no meaningful information, such that a CHANNEL CHECK of this Function is not required.

SR 3.3.17.2

CHANNEL CALIBRATION is a complete check of the instrument channel, including the sensor, to verify the channel responds to the measured parameter(s) within the necessary range and accuracy.

For the Containment Area Radiation instrumentation, a CHANNEL CALIBRATION consists of an electronic calibration of the channel, not including the detector, for range decades above 10 R/hr. The calibration also provides a one point check of the detector below 10 R/hr using a gamma test source (Reference NUREG 0737, Table II.F.1-3).

The 24 month Frequency is based on the results of a review of instrument drift data conducted in accordance with NRC Generic Letter 91-04. The Frequency for the hydrogen monitors is 18 months based on operating experience and was originally selected to be consistent with the typical industry fuel cycle.

A Note clarifies that the neutron detectors are not required to be tested as part of the CHANNEL CALIBRATION. Adjustment of the detectors is unnecessary because they are passive devices and operating experience has shown them to exhibit minimal drift. Furthermore, there is no adjustment that can be made to the detectors.

A Note to the Surveillance excludes the performance of a CHANNEL CALIBRATION on Functions 23 and 25. FPC requested, and was granted, exception from performing a CHANNEL CALIBRATION on this instrumentation as part of Amendment [TBD], dated [TBD]. Since no adjustment of these on-off indications is possible, performance of the SR provides no meaningful information such that a CHANNEL CALIBRATION of this Function is not required. Instead, these indications are subjected to a CHANNEL FUNCTIONAL TEST as described in SR 3.3.17.3.

(continued)

BASES

SURVEILLANCE
REQUIREMENTS
(continued)

SR 3.3.17.3

CHANNEL FUNCTIONAL TEST is, for switch contacts, the injection of a simulated or actual signal into the channel as close to the sensor as practicable to verify OPERABILITY, including required alarm and trip functions.

For the LPI Pump Run Status, and the HPI Pump Run Status, the CHANNEL FUNCTIONAL TEST verifies the red indicating lights on the main control board and the ES Light Matrix indicators during testing of the pumps already required by other surveillance requirements.

The 24 month frequency is based on providing consistency between this surveillance testing and the other surveillance requirements for verifying operability of these specific pumps.

REFERENCES

1. FSAR, Table 7-12.
 2. Regulatory Guide 1.97, Revision 3.
 3. NUREG-0737, 1979.
 4. 32-1177256-00, "Technical Basis for Reactor Vessel Level Indication System (RVLIS) Action Statement," April 10, 1990.
-
-



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 177 TO FACILITY OPERATING LICENSE NO. DPR-72
TO ADD FOUR REGULATORY GUIDE 1.97 INSTRUMENTATION VARIABLES

FLORIDA POWER CORPORATION

CRYSTAL RIVER UNIT 3

DOCKET NO. 50-302

1.0 INTRODUCTION

By letter dated August 31, 1998, Florida Power Corporation, the licensee for Crystal River Unit 3 (CR-3), requested U.S. Nuclear Regulatory Commission (NRC) approval to implement an amendment to its operating license by incorporating modifications to the Technical Specifications (TS) for post-accident monitoring (PAM) instrumentation. The licensee proposes to add (1) low pressure injection (LPI) pump run status, (2) LPI suction from reactor building (RB) sump isolation valves DHV-42 and DHV-43 open position, and (3) high pressure injection (HPI) pump run status to Type A Category 1 PAM instrumentation variables. In addition, the licensee proposes to add reactor coolant system (RCS) low-range pressure to Type B Category 1 PAM instrumentation variables. The requested additions are a result of a licensee review of the emergency operating procedures (EOPs) and variables associated with Regulatory Guide (RG) 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident."

2.0 EVALUATION

2.1 LPI Pump Run Status

Emergency Operating Procedures instruct operators to verify LPI pump run status during several evolutions. During loss-of-coolant accidents (LOCA), the LPI pump is manually aligned to provide suction to an associated HPI pump. This arrangement is referred to as LPI/HPI piggyback operation. The EOPs direct operators to verify each LPI pump is operating prior to opening the appropriate valve between the LPI pump discharge and the associated HPI pump suction. In addition, LPI/HPI piggyback operation is required when single component failures result in one available LPI train.

The licensee proposes a monthly channel check for the main control board indicating light and the Engineered Safeguards (ES) Light Matrix indicator for each LPI pump. Since the indicating lights have no adjustment and provide only on-off indication, a channel calibration surveillance requirement (SR) is not meaningful. Instead, the licensee proposes to add SR 3.3.17.3, "Channel Functional Test." The new SR will be performed on a 24-month frequency. The Channel Functional Test involves injecting a signal into the channel as close to the sensor as practicable to verify operability of the switch contacts. The 24-month frequency is consistent with the SR for PAM Instrumentation Function 20, LPI Flow, in Table 3.3.17-1.

9906080233 990603
PDR ADDCK 05000302
P PDR

The licensee has classified and evaluated the LPI pump run status as a Type A Category 1 variable in accordance with guidance delineated in Regulatory Guide (RG) 1.97. The staff finds the licensee's evaluation consistent with RG 1.97 guidance, and is therefore acceptable.

2.2 DHV-42 and DHV-43 Open Position

During LPI/HPI piggyback operation, valves DHV-42 and DHV-43 are open to preserve operability of the LPI pump, building spray pump, and the HPI pump. Emergency Operating Procedures require operators to verify that valves DHV-42 and DHV-43 have been successfully opened to ensure suction from the RB sump prior to isolating suction from the borated water storage tank.

The licensee has determined that the DHV-42 and DHV-43 open position instrumentation will be subject to a monthly channel check and 24-month channel calibration as described in the existing SR 3.3.17.1. This SR is consistent with the SR for other PAM instrumentation sump isolation functions in Table 3.3.17-1.

The licensee has classified and evaluated the DHV-42 and DHV-43 open position as Type A Category 1 instrumentation variable in accordance with guidance delineated in RG 1.97. The staff finds the licensee's evaluation consistent with RG 1.97 guidance and is, therefore, acceptable.

2.3 HPI Pump Run Status

During emergency operations, two ES selected HPI pumps are designed to automatically start. The EOPs contain actions for operators to verify the HPI pump(s) operating status during several evolutions. During certain LOCA scenerios, the HPI pump run status is necessary to comply with required EOP actions for opening HPI pump recirculation valves to protect the necessary HPI pump(s) from damage due to flow conditions. The EOPs specify different minimum HPI pump flow rates for opening HPI pump recirculation valves depending on the number of operating HPI pumps.

The licensee proposed a monthly channel check for the main control board indicating light and the ES Light Matrix indicator for each pump. Since the indicating lights have no adjustment and provide only on-off indication, a channel calibration SR is not meaningful. Instead, the licensee proposed to add SR 3.3.17.3, "Channel Functional Test." The new SR will be performed on a 24-month frequency. The Channel Functional Test involves injecting a signal into the channel as close to the sensor as practicable to verify operability of the switch contacts. The 24-month frequency is consistent with the SR for PAM Instrumentation Function 6, HPI Flow, in Table 3.3.17-1.

The licensee has classified and evaluated the HPI pump run status as a Type A Category 1 variable in accordance with guidance delineated in RG 1.97. The staff finds the licensee's evaluation consistent with RG 1.97 guidance, and is therefore acceptable.

2.4 RCS Low Range Pressure

In several contingency actions, the EOPs use RCS pressure to verify automatic operation of emergency core cooling systems. Current PAM instrumentation for RCS pressure (Wide Range) covers the entire range from 0-3000 psig. During post-accident conditions, low pressure injection operation is more accurately verified by monitoring the RCS Pressure (Low Range) instrumentation that displays RCS pressures from 0-600 psig.

The licensee has determined that the RCS low range pressure instrumentation will be subject to a monthly channel check and 24-month channel calibration as described in the existing SR 3.3.17.1. This SR is the same as for PAM Instrumentation Function 3, RCS Pressure (Wide Range), Table 3.3.17-1.

The licensee has classified and evaluated the RCS low range pressure as Type B Category 1 variable in accordance with guidance delineated in RG 1.97. The staff finds the licensee's evaluation consistent with RG 1.97 guidance, and is therefore acceptable.

3.0 STATE CONSULTATION

Based upon a letter dated March 8, 1991, from Mary E. Clark of the State of Florida, Department of Health and Rehabilitative Services, to Deborah A. Miller, Licensing Assistant, U.S. NRC, the State of Florida does not desire notification of issuance of license amendments.

4.0 ENVIRONMENTAL CONSIDERATIONS

The amendment changes requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (63 FR 56250). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

Based on the above evaluation, the staff concludes that the proposed additions to the PAM instrumentation functions and corresponding TS changes are consistent with the guidance in RG 1.97, and are therefore acceptable.

Principal Contributor: A. Bryant

Date: June 3, 1999

Mr. John Paul Cowan
Florida Power Corporation

CRYSTAL RIVER UNIT NO. 3

cc:

Mr. R. Alexander Glenn
Corporate Counsel (MAC-BT15A)
Florida Power Corporation
P.O. Box 14042
St. Petersburg, Florida 33733-4042

Chairman
Board of County Commissioners
Citrus County
110 North Apopka Avenue
Inverness, Florida 34450-4245

Mr. Charles G. Pardee, Director
Nuclear Plant Operations (PA4A)
Florida Power Corporation
Crystal River Energy Complex
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Ms. Sherry L. Bernhoft, Director
Nuclear Regulatory Affairs (NA2H)
Florida Power Corporation
Crystal River Energy Complex
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Mr. Michael A. Schoppman
Framatome Technologies Inc.
1700 Rockville Pike, Suite 525
Rockville, Maryland 20852

Senior Resident Inspector
Crystal River Unit 3
U.S. Nuclear Regulatory Commission
6745 N. Tallahassee Road
Crystal River, Florida 34428

Mr. William A. Passetti, Chief
Department of Health
Bureau of Radiation Control
2020 Capital Circle, SE, Bin #C21
Tallahassee, Florida 32399-1741

Mr. Gregory H. Halnon
Director, Quality Programs (SA2C)
Florida Power Corporation
Crystal River Energy Complex
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Mr. Joe Myers, Director
Division of Emergency Preparedness
Department of Community Affairs
2740 Centerview Drive
Tallahassee, Florida 32399-2100