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U. S. Nuclear Regulatory Commission  
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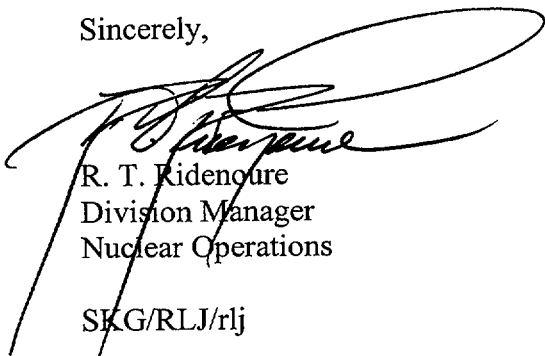
Reference: Docket No. 50-285

**SUBJECT: Transmittal of Fort Calhoun Station (FCS) Core Operating Limit Report (COLR) Revision 25**

Pursuant to FCS Unit No. 1 Technical Specification 5.9.5.c, Omaha Public Power District is transmitting Revision 25 (dated January 7, 2002) of the FCS COLR. The COLR was updated to allow for operation in the presence of a hot leg flow streaming event of up to 6% deviation.

Please contact me if you have any questions.

Sincerely,



R. T. Kidenoure  
Division Manager  
Nuclear Operations

SKG/RLJ/rlj

Attachment: TDB-VI – Technical Data Book – Core Operating Limit Report – Revision 25

- c: E. W. Merschoff, NRC Regional Administrator, Region IV
- A. B. Wang, NRC Project Manager
- W. C. Walker, NRC Senior Resident Inspector
- Winston & Strawn (w/o attachments)

*1001*

WP8

Fort Calhoun Station  
Unit No. 1

**TDB-VI**

TECHNICAL DATA BOOK

**Title:** CORE OPERATING LIMIT REPORT

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FC-68 Number: EC 29235

Reason for Change: To allow for operating in the presence of a hot leg flow streaming event of up to 6% deviation.

Requestor: Tom Heng

Preparer: Tom Heng

**FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE****TDB-VI  
PAGE 1 OF 19****Fort Calhoun Station, Unit 1****Cycle 20 Core Operating Limits Report**

Due to the critical aspects of the safety analysis inputs contained in this report, changes may not be made to this report without concurrence of the Nuclear Engineering Department.

**FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE****TDB-VI  
PAGE 2 OF 19****TABLE OF CONTENTS**

<u>Item</u>	<u>Description</u>	<u>Page</u>
1.0	Introduction	5
2.0	Core Operating Limits	5
3.0	TM/LP Limit	7
4.0	Maximum Core Inlet Temperature	7
5.0	Power Dependent Insertion Limit	8
6.0	Linear Heat Rate	8
7.0	Excore Monitoring of LHR	8
8.0	Peaking Factor ( $F_{RT}$ ) Limits	8
9.0	DNB Monitoring	8
10.0	$F_{RT}$ and Core Power Limitations	8
11.0	Refueling Boron Concentration	9
12.0	Axial Power Distribution	9
13.0	Shutdown Margin With $T_{cold} > 210^{\circ}F$	9
14.0	Most Negative Moderator Temperature Coefficient	9

**FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE****TDB-VI  
PAGE 3 OF 19****LIST OF TABLES**

<u>Table No.</u>	<u>Title</u>	<u>Page</u>
1	TM/LP Coefficients	7
2	Refueling Boron Concentrations	9

**FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE****TDB-VI  
PAGE 4 OF 19****LIST OF FIGURES**

<b><u>Figure No.</u></b>	<b><u>Title</u></b>	<b><u>Page</u></b>
1	Thermal Margin/Low Pressure for 4 Pump Operation	10
2	Power Dependent Insertion Limit	11
3	Allowable Peak Linear Heat Rate vs. Burnup	12
4	Excore Monitoring of LHR	13
5	DNB Monitoring	14
6	BASSS Allowable Power	15
7	$F_{RT}$ and Core Power Limitations	16
8	Axial Power Distribution LSSS for 4 Pump Operation	17
9	Axial Power Distribution Limits for 4 Pump Operation with Incores Inoperable	18
10	Minimum Boric Acid Storage Tank Level vs. Stored Boric Acid Storage Tank Concentration	19

FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE

TDB-VI  
PAGE 5 OF 19

## Cycle 20 Core Operating Limits Report

### 1.0 Introduction

This report provides the cycle-specific limits for operation of the Fort Calhoun Station Unit 1 for Cycle 20 operation. It includes limits for:

- TM/LP LSSS for 4 Pump Operation ( $P_{VAR}$ )
- Core Inlet Temperature ( $T_{IN}$ )
- Power Dependent Insertion Limit (PDIL)
- Allowable Peak Linear Heat Rate
- Excore Monitoring of LHR
- Integrated Radial Peaking Factor ( $F_{R^T}$ )
- DNB Monitoring
- $F_{R^T}$  versus Power Trade-off Curve
- Refueling Boron Concentration
- Axial Power Distribution
- Shutdown Margin With  $T_{cold} > 210^\circ\text{F}$
- Most Negative Moderator Temperature Coefficient

These limits are applicable for the duration of Cycle 20. For subsequent cycles the limits will be reviewed and revised as necessary. In addition, this report includes a number of cycle-specific coefficients used in the generation of certain reactor protective system trip setpoints or allowable increases in radial peaking factors.

### 2.0 Core Operating Limits

All values and limits in this TDB section apply to Cycle 20 operation. Cycle 20 must be operated within the bounds of these limits and all others specified in the Technical Specifications. This report has been prepared in accordance with the requirements of Technical Specification 5.9.5. The values and limits presented within this TDB section have been derived using the NRC approved methodologies listed below:

- OPPD-NA-8301, "Reload Core Analysis Methodology Overview," Rev. 6, dated December 1994. (TAC No. M89455)
- OPPD-NA-8302, "Reload Core Analysis Methodology, Neutronics Design Methods and Verification," Rev. 4, dated December 1994. (TAC No. M89456)
- OPPD-NA-8303, "Reload Core Analysis Methodology, Transient and Accident Methods and Verification," Rev. 4, dated January 1993. (TAC No. M85845)

**FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE****TDB-VI  
PAGE 6 OF 19**

- XN-75-32(P)(A) Supplements 1, 2, 3, & 4, "Computational Procedure for Evaluating Fuel Rod Bowing," October 1983.
- XN-NF-82-08(P)(A) and Supplements 2, 4, and 5, "Qualification of Exxon Nuclear Fuel for Extended Burnup," Revision 1, October 1986.
- XN-NF-85-92(P)(A), "Exxon Nuclear Uranium Dioxide/Gadolinia Irradiation Examination and Thermal Conductivity Results," August 1985.
- ANF-88-133(P)(A) and Supplement 1, "Qualification of Advanced Nuclear Fuels PWR Design Methodology for Rod Burnups of 62 GWd/MTU," December 1991.
- EMF-92-116(P)(A), "Generic Mechanical Design Criteria for PWR Fuel Designs," Revision 0, February 1999.
- XN-NF-78-44(P)(A), "A Generic Analysis of the Control Rod Ejection Transient for Pressurized Water Reactors," October 1983.
- XN-NF-82-21(P)(A), "Application of Exxon Nuclear Company PWR Thermal Margin Methodology to Mixed Core Configurations," Revision 1, September 1983.
- EMF-1961(P)(A), "Statistical Setpoint/Transient Methodology for Combustion Engineering Type Reactors," Revision 0, July 2000.
- XN-NF-621(P)(A), "Exxon Nuclear DNB Correlation for PWR Fuel Designs," Revision 1, September 1983.
- ANF-89-151(P)(A), "ANF-RELAP Methodology for Pressurized Water Reactors: Analysis of Non-LOCA Chapter 15 Events," Revision 0, May 1992.
- EMF-92-153(P)(A) and Supplement 1, "HTP: Departure from Nucleate Boiling Correlation for High Thermal Performance Fuel," March 1994.
- XN-NF-82-49(P)(A), Supplement 1, "Exxon Nuclear Company Evaluation Model Revised EXEM PWR Small Break Model," Revision 1, December 1994.
- EMF-2087(P)(A), "SEM/PWR-98: ECCS Evaluation Model for PWR LBLOCA Applications," Revision 0, June 1999.
- ANF-84-73 Appendix B (P)(A), "Advanced Nuclear Fuels Methodology for Pressurized Water Reactors: Analysis of Chapter 15 Events," Advanced Nuclear Fuels Corporation, Revision 5, July 1990.
- EMF-84-093(P)(A), "Steam Line Break Methodology for PWRs," Siemens Power Corporation, Revision 1, February 1999.



FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE

TDB-VI  
PAGE 7 OF 19

### 3.0 TM/LP Limit

The TM/LP coefficients for Cycle 20 are shown below:

**Table 1**  
**TM/LP Coefficients**

<u>Coefficient</u>	<u>Value</u>
$\alpha$	29.6
$\beta$	20.63
$\gamma$	-12372

The TM/LP setpoint is calculated by the  $P_{VAR}$  equation, shown below and in Figure 1:

$$P_{VAR} = 29.6 PF(B) A1(Y)B + 20.63T_{IN} - 12372$$

$$\begin{aligned} PF(B) &= 1.0 && \text{for } B \geq 100\% \\ &= -0.008(B) + 1.8 && \text{for } 50\% < B < 100\% \\ &= 1.4 && \text{for } B \leq 50\% \end{aligned}$$

$$\begin{aligned} A1(Y) &= -0.6666(Y_1) + 1.000 && \text{for } Y_1 \leq 0.00 \\ &= +0.3333(Y_1) + 1.000 && \text{for } Y_1 > 0.00 \end{aligned}$$

Where:

B = High Auctioneered thermal ( $\Delta T$ ) or Nuclear Power, % of rated power

Y = Axial Shape Index, asi

$T_{IN}$  = Core Inlet Temperature, °F

$P_{VAR}$  = Reactor Coolant System Pressure, psia

### 4.0 Maximum Core Inlet Temperature

The maximum core inlet temperature ( $T_{IN}$ ) for Cycle 20 shall not exceed **545°F**. This value includes instrumentation uncertainty of  $\pm 2^\circ\text{F}$  (Ref: FCS Calculation FC06292, 6/9/95).

This limit is not applicable during either a thermal power ramp in excess of 5% of rated thermal power per minute or a thermal power step greater than 10% of rated thermal power.

**FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE****TDB-VI  
PAGE 8 OF 19****5.0 Power Dependent Insertion Limit**

The power dependent insertion limit is defined in Figure 2.

**6.0 Linear Heat Rate**

The allowable peak linear heat rate for Cycle 20 is shown in Figure 3.

**7.0 Excore Monitoring of LHR**

The allowable operation for power versus axial shape index for monitoring of LHR with excore detectors for Cycle 20 is shown in Figure 4.

**8.0 Peaking Factor Limits**

The Cycle 20 maximum full power value for the integrated radial peaking factor ( $F_{R}^T$ ) is 1.732. An additional dotted line exists on Figure 7 to allow operation in the presence of a hot leg flow streaming event with less than a 6% power difference between actual power and  $\Delta T$  power indication.

**9.0 DNB Monitoring**

The core operating limits for monitoring of DNB are provided in Figures 5 and 6. These figures provide the allowable power versus axial shape index for Cycle 20 operation. When the Better Axial Shape Selection System (BASSS) is operable, the limits in Figure 6 apply. If BASSS becomes inoperable, the limits in Figure 5 apply. Note that for Cycle 20, the DNB limits are identical whether BASSS is operable or inoperable.

**10.0  $F_{R}^T$  and Core Power Limitations**

Core power limitations versus  $F_{R}^T$  are shown in Figure 7 for Cycle 20.

FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE

TDB-VI  
PAGE 9 OF 19

### 11.0 Refueling Boron Concentration

The refueling boron concentration is required to ensure a shutdown margin of not less than 5% with all CEAs withdrawn. The refueling boron concentration must be at least **1,900 ppm** through the end of Cycle 19 operation and is valid until the beginning of core reload for Cycle 20.

Listed below in Table 2 are the refueling boron concentration values for Cycle 20 operations:

**Table 2**  
**Refueling Boron Concentrations**

Cycle 20 Average Burnup (MWD/MTU)	Refueling Boron Concentration (ppm)
BOC	2,155
≥ 2,000	2,023
≥ 4,000	1,900

### 12.0 Axial Power Distribution

The axial power trip is provided to ensure that excessive axial peaking will not cause fuel damage. The Axial Shape Index is determined from the axially split excore detectors. The setpoint functions, shown in Figure 8 ensure that neither a DNBR of less than the minimum DNBR safety limit nor a maximum linear heat rate of more than 22 kW/ft (deposited in the fuel) will exist as a consequence of axial power maldistributions. Allowances have been made for instrumentation inaccuracies and uncertainties associated with the excore symmetric offset – incore axial peaking relationship. Figure 9 combines the LHR LCO tent from Figure 4, the DNB LCO tent from Figure 5, and the APD LSSS tent from Figure 8 into one figure for a visual comparison of the different limits.

### 13.0 Shutdown Margin With $T_{cold} > 210^{\circ}\text{F}$

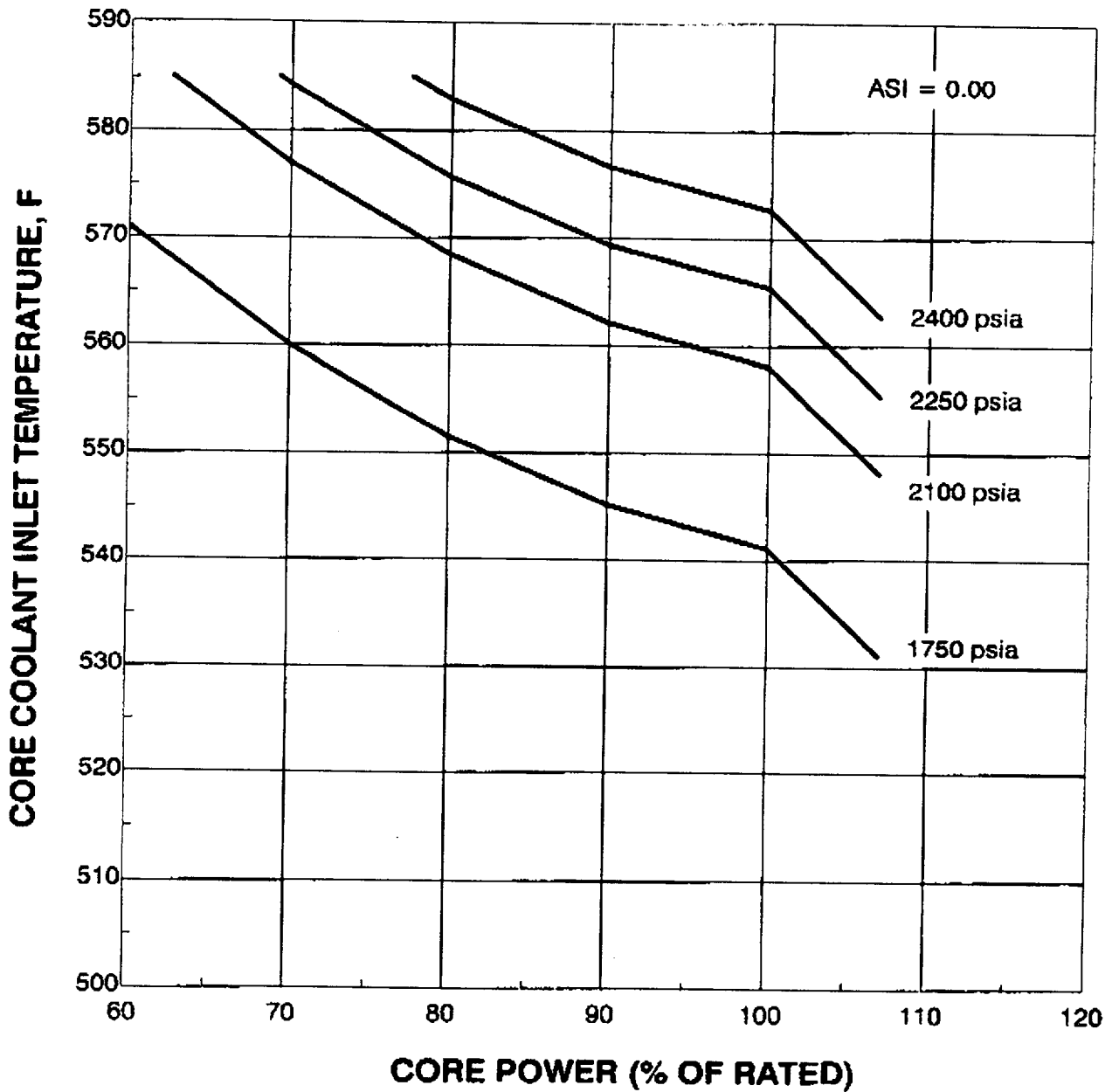
Whenever the reactor is in hot shutdown, hot standby or power operation conditions, the shutdown margin shall be  $\geq 4.0\% \Delta k/k$ . With the shutdown margin  $< 4.0\% \Delta k/k$ , initiate and continue boration until the required shutdown margin is achieved.

### 14.0 Most Negative Moderator Temperature Coefficient

The moderator temperature coefficient (MTC) shall be more positive than  $-3.05 \times 10^{-4} \Delta\rho/^{\circ}\text{F}$ , including uncertainties, at rated power.

FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE

TDB-VI  
PAGE 10 OF 19



$$P_{VAR} = 29.6PF(B)A1(Y)B + 20.63T_{IN} - 12372$$

PF(B) = 1.0	B ≥ 100%	A1(Y) = -0.6666Y <sub>1</sub> + 1.000	Y <sub>1</sub> ≤ 0.00
= -.008B + 1.8	50% < B < 100%	= +0.3333Y <sub>1</sub> + 1.000	Y <sub>1</sub> > 0.00
= 1.4	B ≤ 50%		

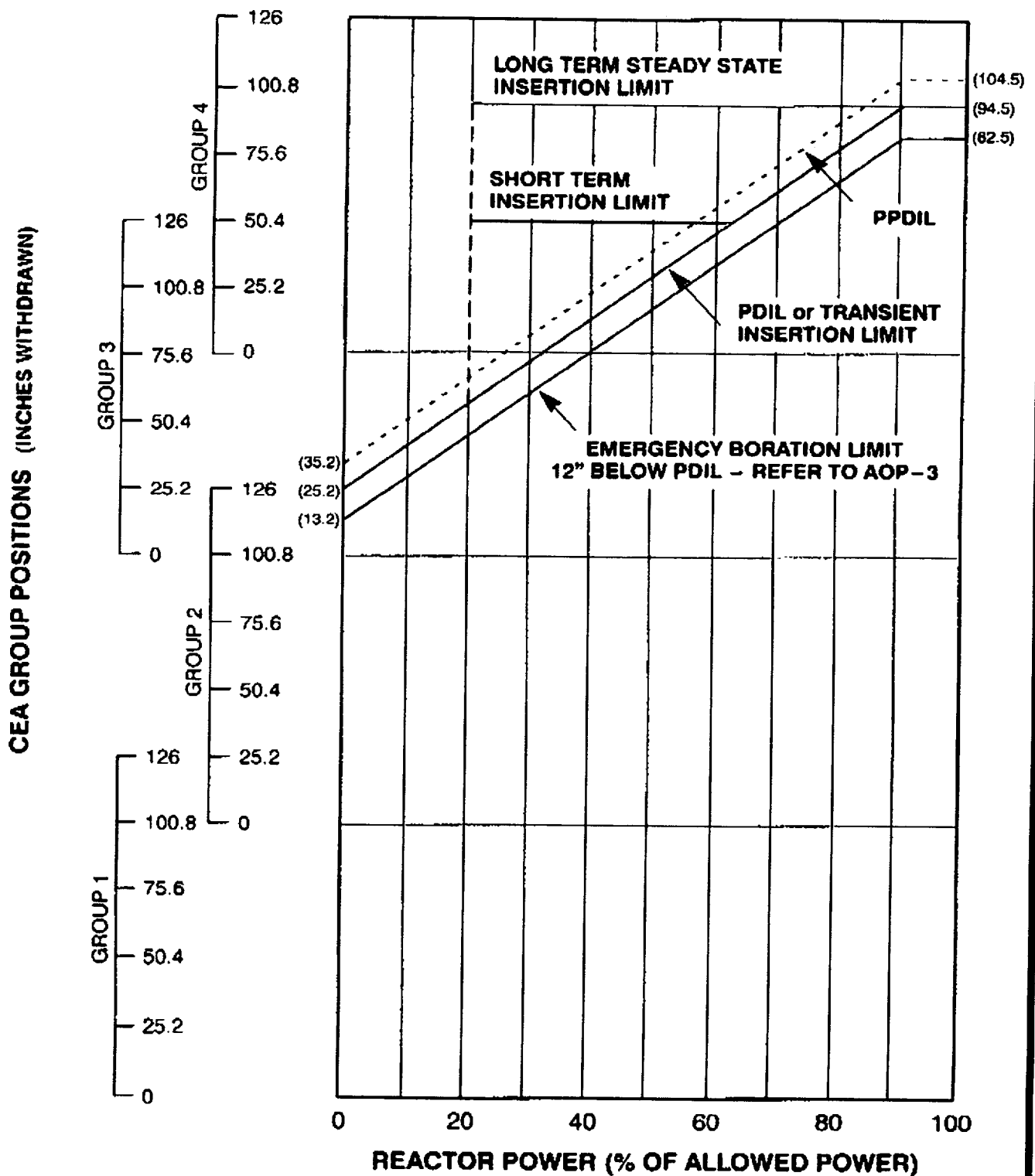
CYCLE 20  
COLR

THERMAL MARGIN / LOW PRESSURE  
FOR 4 PUMP OPERATION

FIGURE  
1

FORT CALHOUN STATION  
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TDB-VI  
PAGE 11 OF 19



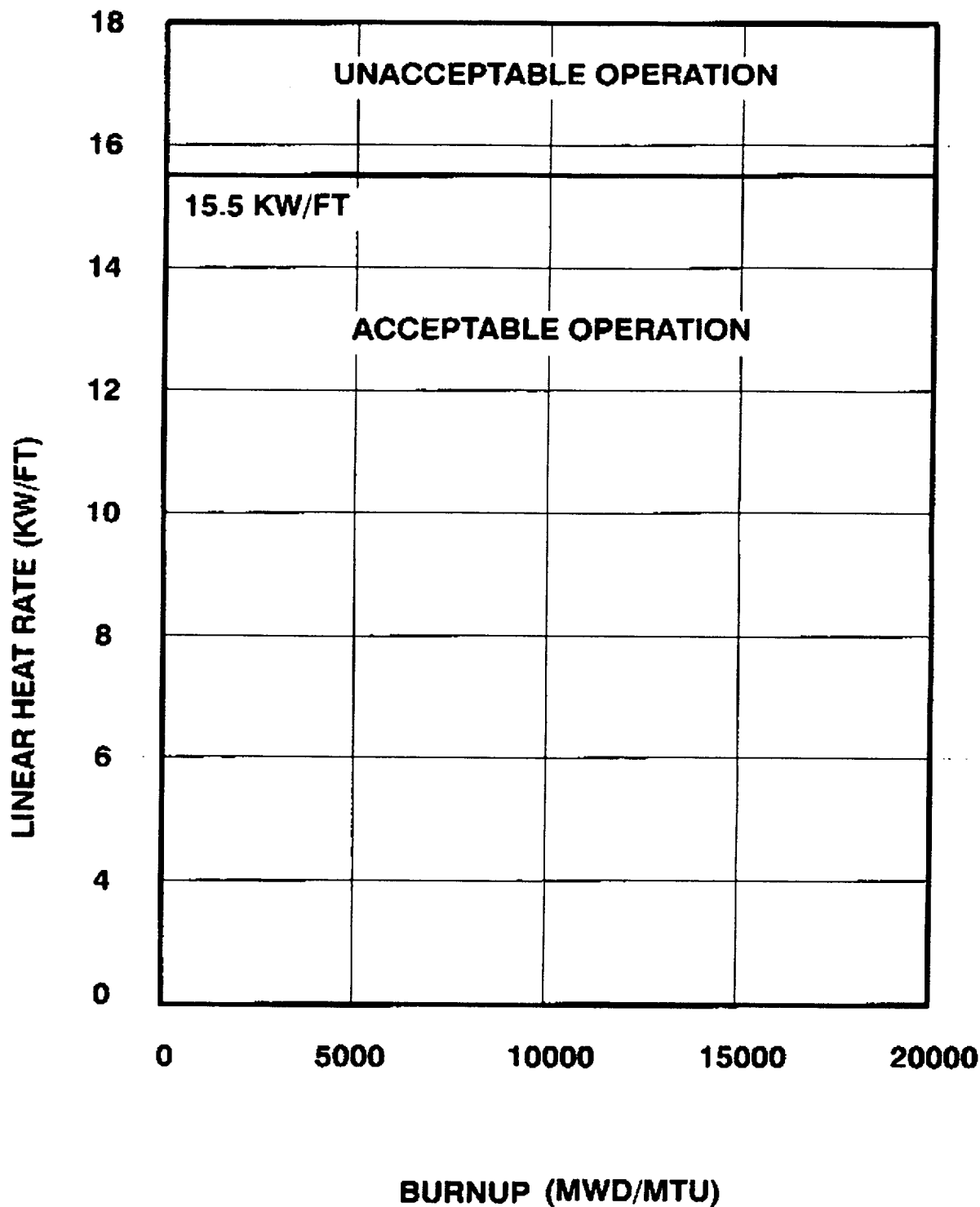
CYCLE 20  
COLR

POWER DEPENDENT INSERTION LIMIT

FIGURE  
2

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TECHNICAL DATA BOOK PROCEDURE

TDB-VI  
PAGE 12 OF 19



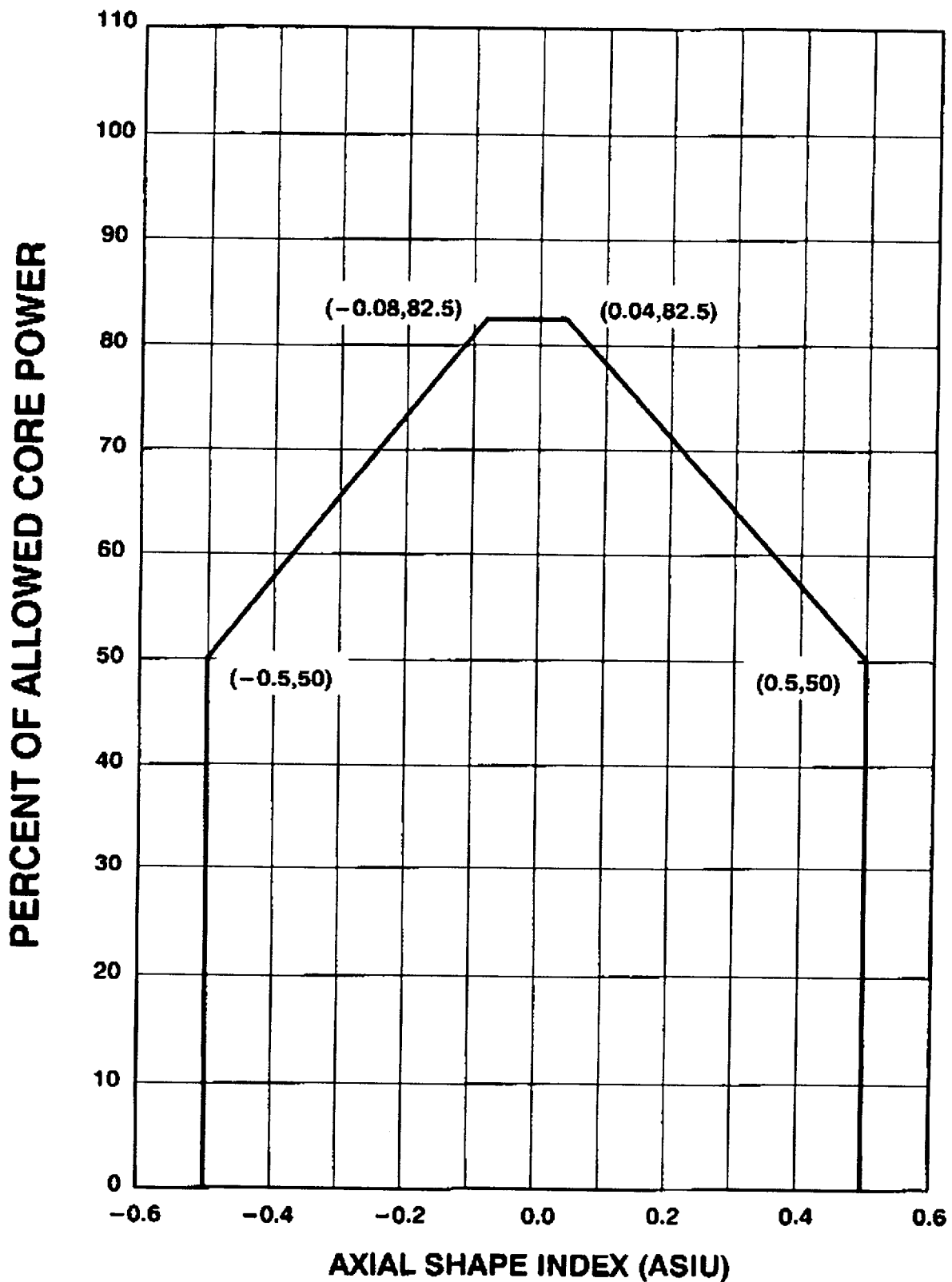
CYCLE 20  
COLR

ALLOWABLE PEAK LINEAR HEAT RATE  
VS. BURNUP

FIGURE  
3

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TECHNICAL DATA BOOK PROCEDURE

TDB-VI  
PAGE 13 OF 19



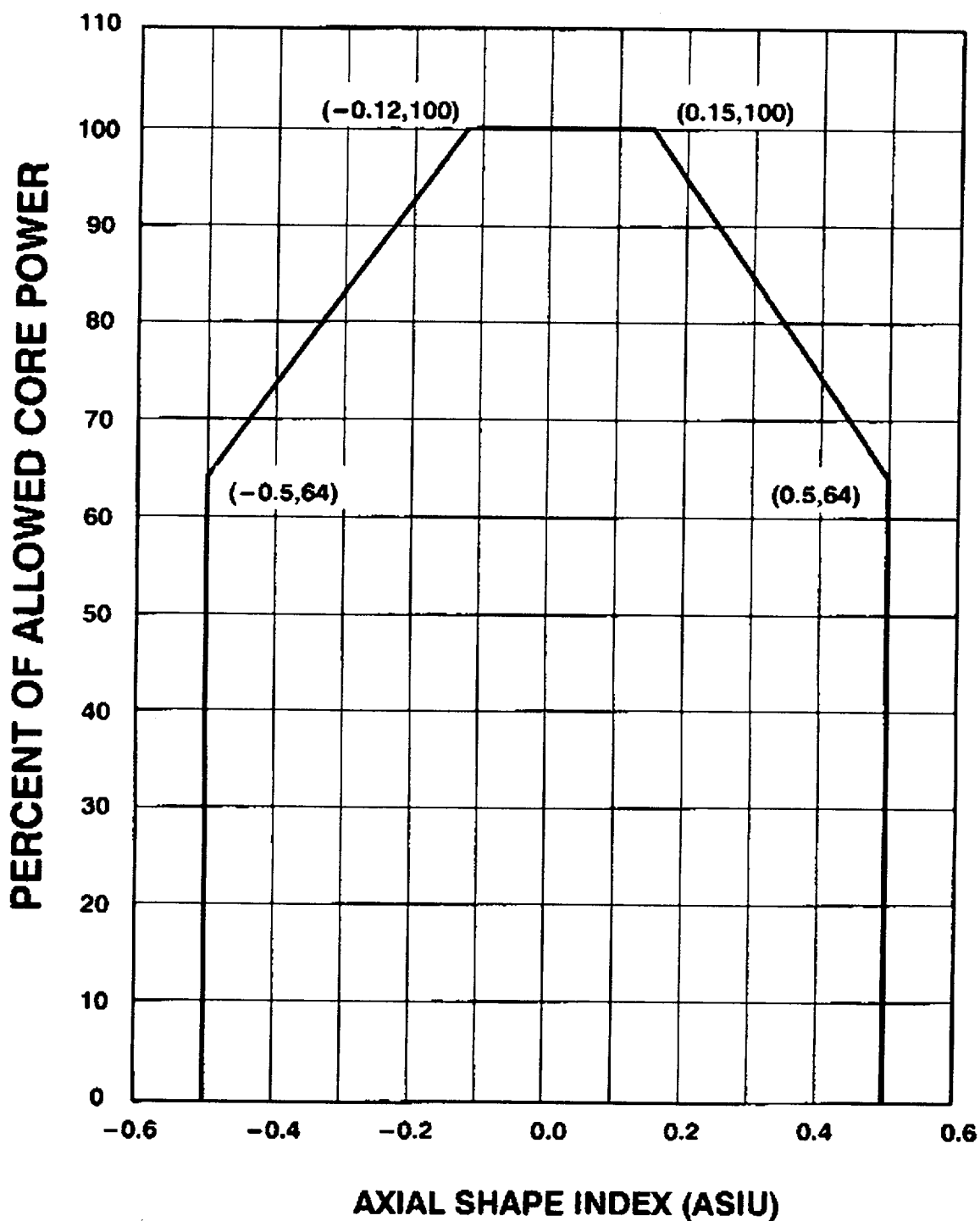
CYCLE 20  
COLR

EXCORE MONITORING OF LHR

FIGURE  
4

FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE

TDB-VI  
PAGE 14 OF 19



**NOTE: WHEN BASSS IS OPERABLE, THIS FIGURE IS SUPERCEDED BY FIGURE 6 – BASSS DNB ALLOWABLE POWER**

CYCLE 20  
COLR

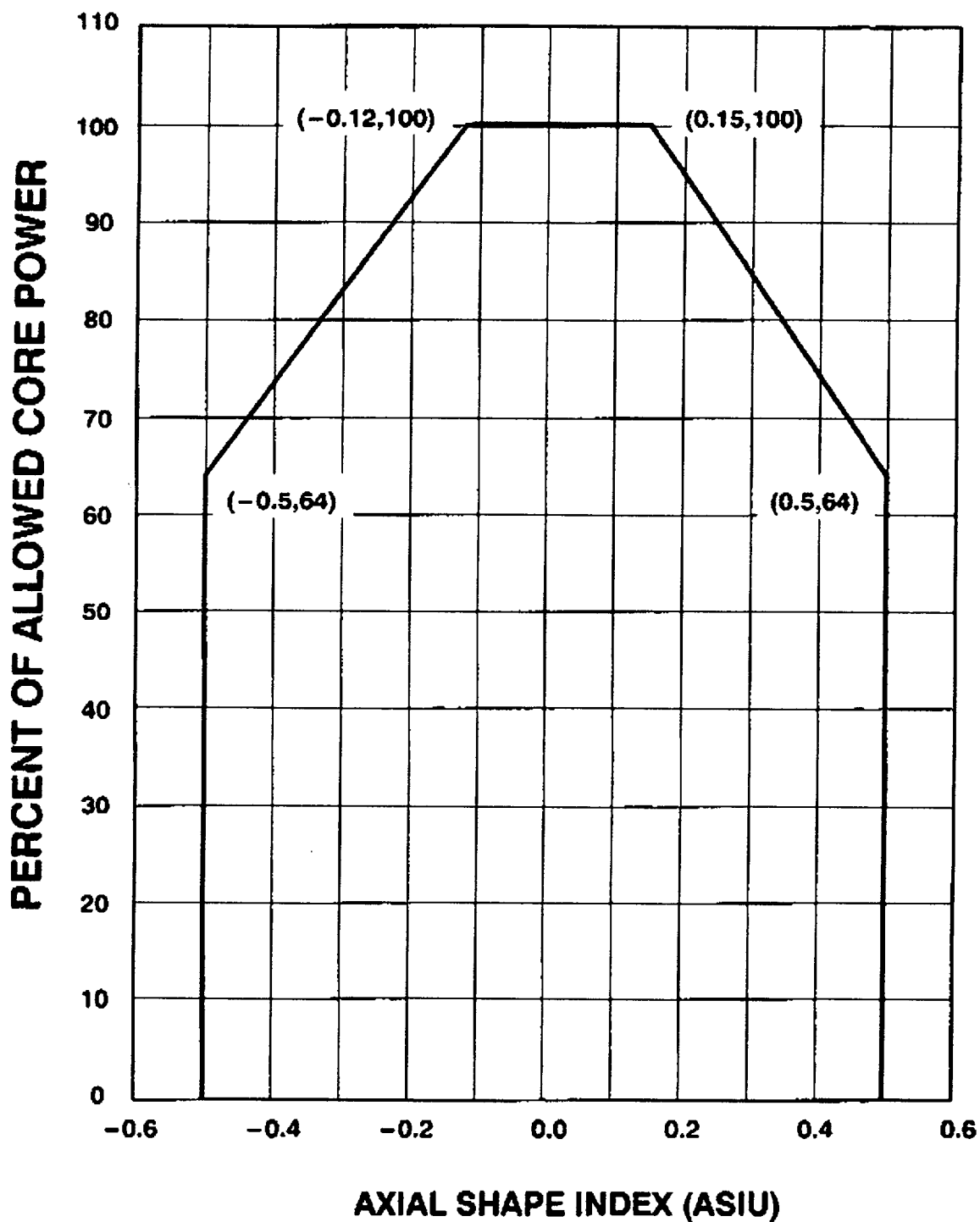
DNB MONITORING

FIGURE  
5



FORT CALHOUN STATION  
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TDB-VI  
PAGE 15 OF 19



CYCLE 20  
COLR

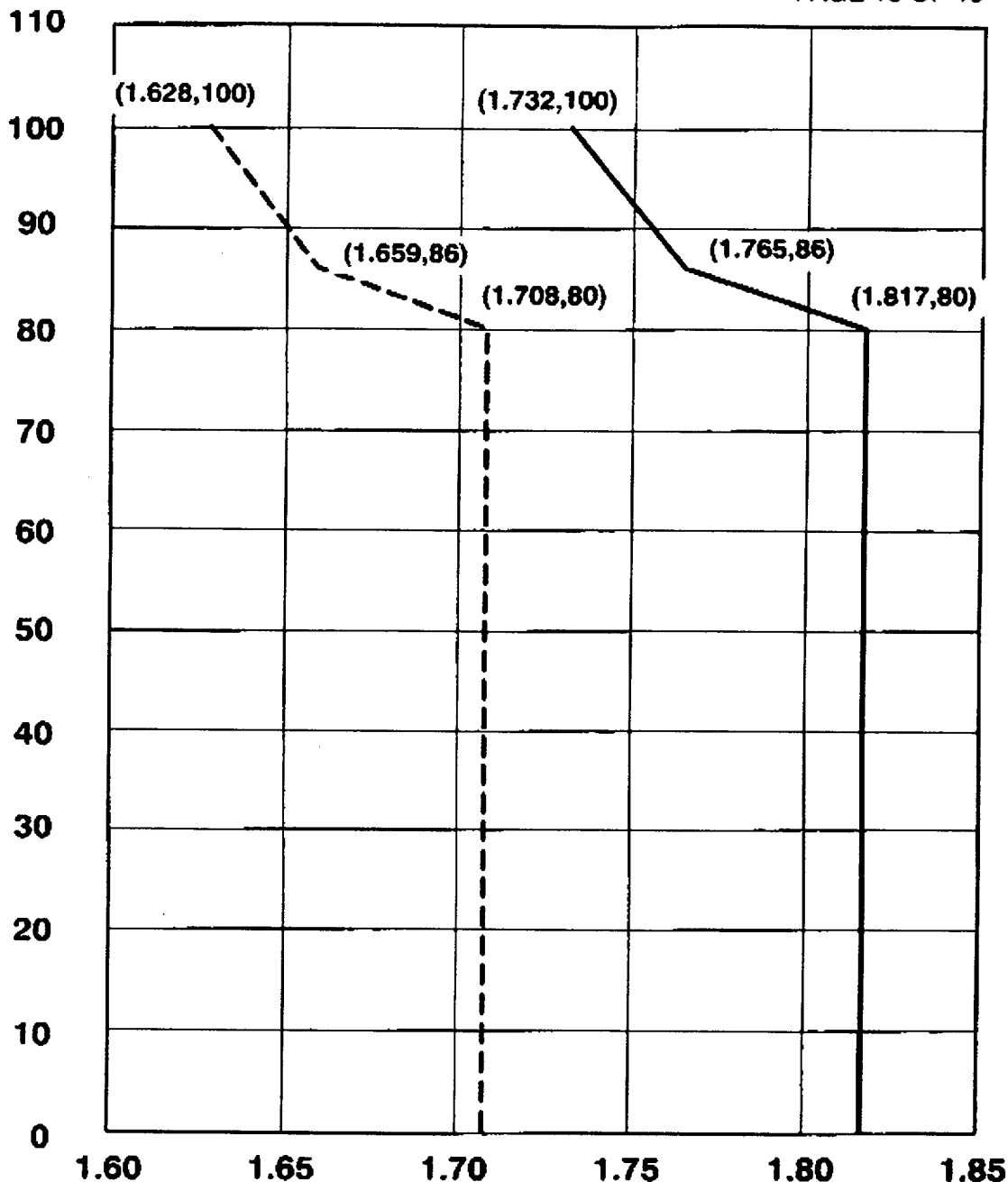
BASSS DNB ALLOWABLE POWER LCO

FIGURE  
6

FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE

TDB-VI  
PAGE 16 OF 19

PERCENT OF ALLOWED CORE POWER



PEAKING FACTOR

- $F_R^T$  Limit with no hot leg flow streaming event in progress
- - - - -  $F_R^T$  Limit if a hot leg flow streaming event is in progress with differences between actual power and  $\Delta T$  power indication of less than 6% power

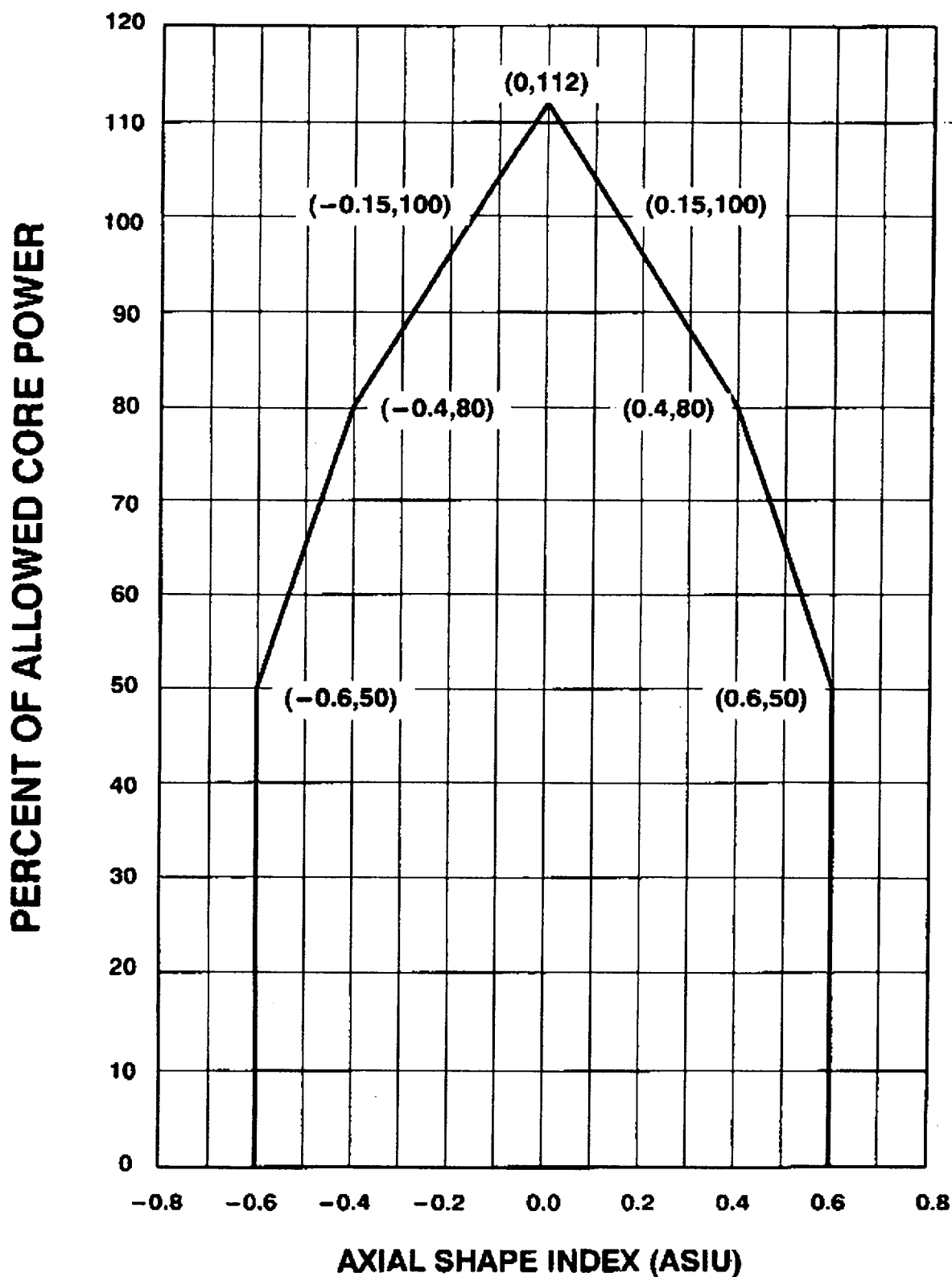
CYCLE 20  
COLR

$F_R^T$  AND CORE POWER LIMITATIONS

FIGURE  
7

FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE

TDB-VI  
PAGE 17 OF 19



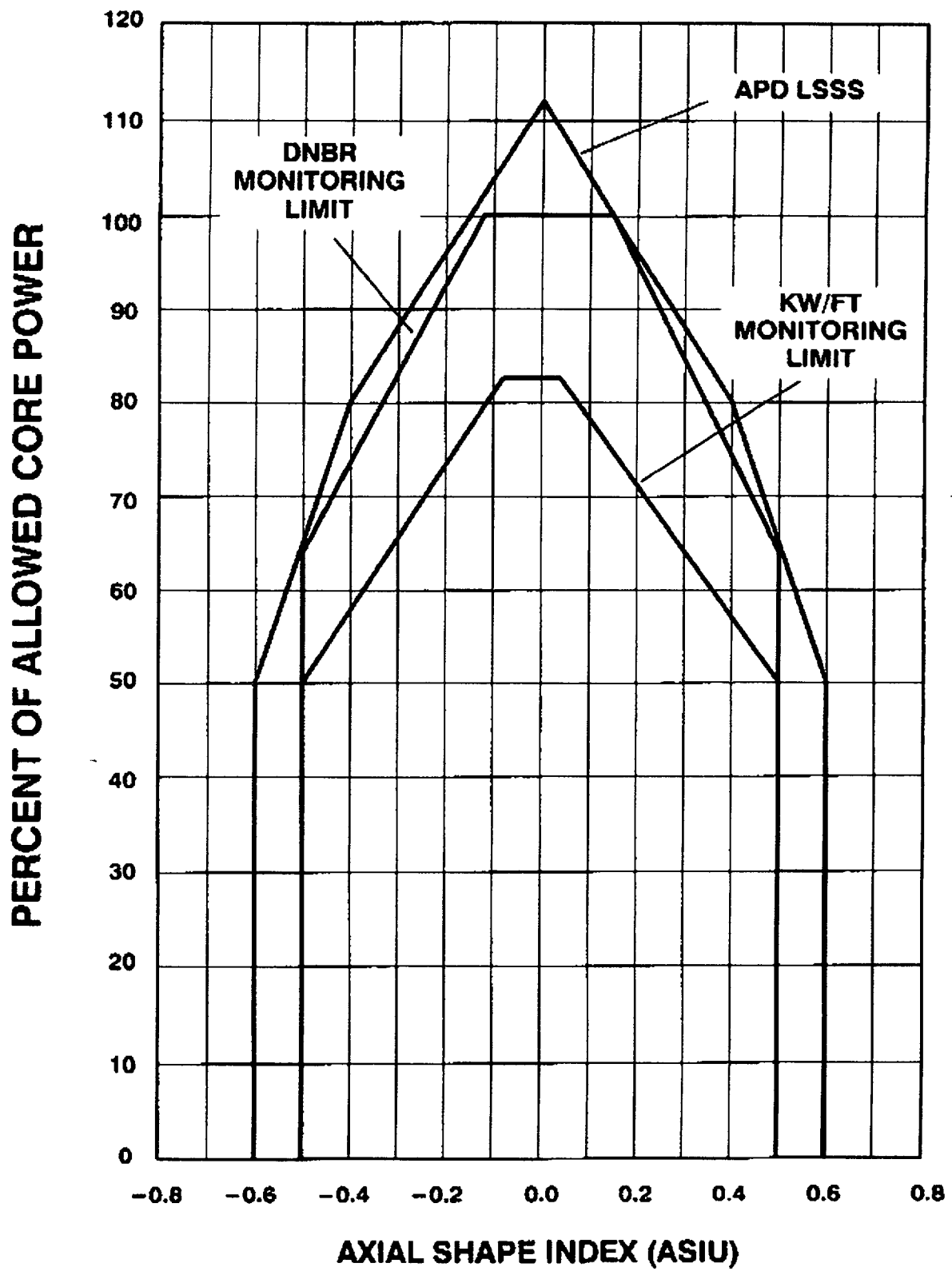
CYCLE 20  
COLR

AXIAL POWER DISTRIBUTION LSSS  
FOR 4 PUMP OPERATION

FIGURE  
8

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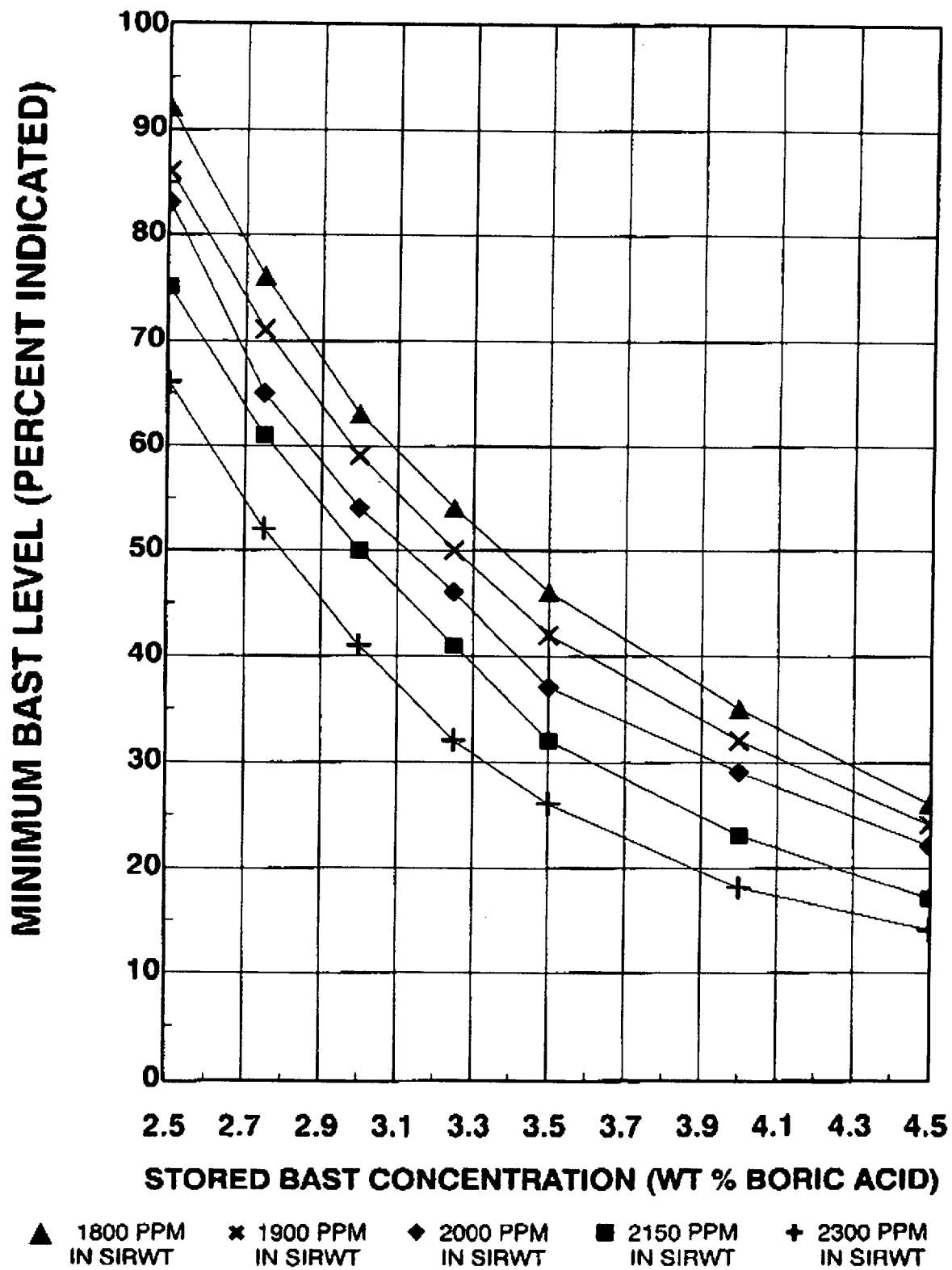
TDB-VI  
PAGE 18 OF 19



CYCLE 20 COLR	AXIAL POWER DISTRIBUTION LIMITS FOR 4 PUMP OPERATION WITH INCORES INOPERABLE	FIGURE 9
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FORT CALHOUN STATION  
TECHNICAL DATA BOOK PROCEDURE

TDB-VI  
PAGE 19 OF 19



CYCLE 20  
COLR

MINIMUM BAST LEVEL vs. STORED  
BAST CONCENTRATION

FIGURE  
10