

July 29, 1985

JMB 016

Docket No. 50-302

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Mr. Walter S. Wilgus
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
& Fuel Management
Post Office Box 14042; M.A.C. H-2
St. Petersburg, Florida 33733

Dear Mr. Wilgus:

The Commission has issued the enclosed Amendment No. 80 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in partial response to your application dated February 14, 1985. You provided additional explanation by letter dated June 19, 1985.

This amendment deletes from the TSs the surveillance requirements for reactor vessel irradiation specimens, as suggested in our letter to the Babcock & Wilcox Owners Group dated May 8, 1985. Instead, you have committed to conform to the Integrated Materials Vessel Surveillance Program (BAW-1543, Rev. 2) approved in that NRC letter. Any changes to that program would require NRC approval.

Other changes requested in your February 14, 1985, submittal will be covered in a separate licensing action.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY

Harley Silver, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 80 to DPR-72
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. W. S. Wilgus
Florida Power Corporation

Crystal River Unit No. 3 Nuclear
Generating Plant

cc:

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and General Counsel
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 80
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated February 14, 1985, as supplemented June 19, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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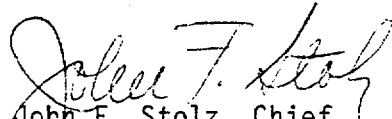
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 80, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 29, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 80

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Page

3/4 4-25

3/4 4-29

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS

4.4.9.1.1 The Reactor Coolant System temperature and pressure shall be determined to be within the limits at least once per 30 minutes during system heatup, cooldown, and inservice leak and hydrostatic testing operations.

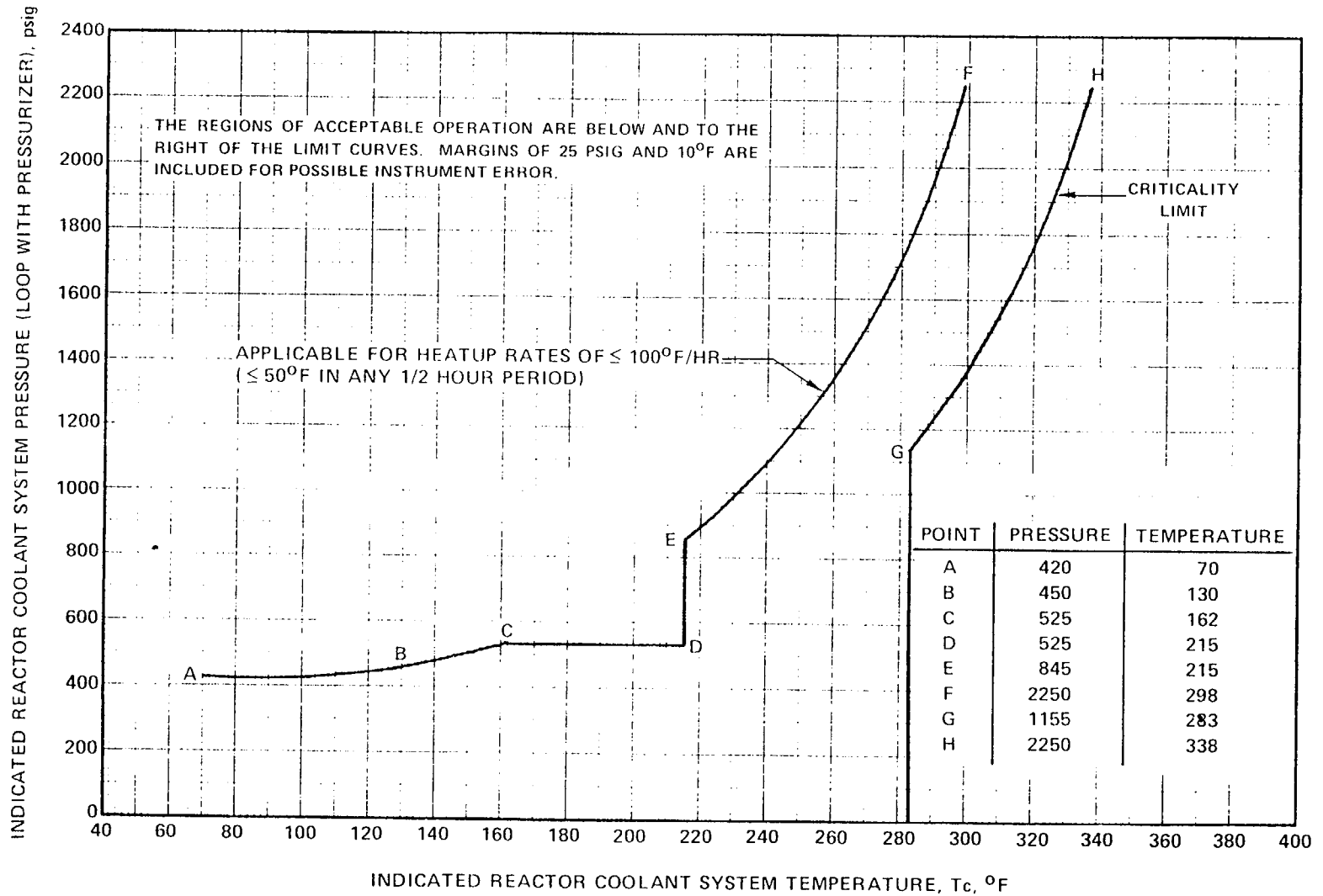


Figure 3.4-2 Reactor Coolant System Pressure- Temperature Limits for Heatup and Core Criticality for the First 5 EFPY

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REACTOR COOLANT SYSTEM

PRESSURIZER

LIMITING CONDITION FOR OPERATION

3.4.9.2 The pressurizer temperature shall be limited to:

- a. A maximum heatup and cooldown of 100°F in any one hour period, and
- b. A maximum spray water temperature differential of 410°F.

APPLICABILITY: At all times.

ACTION:

With the pressurizer temperature limits in excess of any of the above limits, restore the temperature to within limits within 30 minutes; perform an engineering evaluation to determine the effects of the out-of-limit condition on the fracture toughness properties of the pressurizer; determine that the pressurizer remains acceptable for continued operation or be in at least HOT STANDBY within the next 6 hours and reduce pressurizer pressure to less than 500 psig within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.9.2 The pressurizer temperature shall be determined to be within the limits at least once per 30 minutes during system heatup or cooldown. The spray water temperature differential shall be determined to be within the limit once per 12 hours during auxiliary spray operation with pressurizer temperature > 440°F.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET AL.
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

Introduction

In a letter dated February 14, 1985, as supplemented on June 19, 1985, the Florida Power Corporation (the licensee) requested an amendment to the Technical Specifications (TSs) for Crystal River Unit 3 (CR-3) deleting the surveillance requirements in TS Section 4.4.9.1.2 and Table 4.4-5. TS Section 4.4.9.1.2 contains the requirements for installation and removal of the CR-3 reactor vessel material surveillance capsules, and Table 4.4-5 defines the schedule for such actions.

Discussion and Evaluation

Appendix H, 10 CFR 50, requires that each licensee maintain either an in-vessel or integrated reactor vessel material surveillance program. This Appendix also requires that the surveillance schedule be approved by the NRC staff.

The CR-3 reactor vessel material surveillance capsules are part of the Babcock & Wilcox (B&W) Owners Group Integrated Surveillance Program. This program is documented in B&W Topical Report BAW-1543, Rev. 2, which includes an installation and withdrawal schedule for each CR-3 capsule. The topical report was submitted for NRC staff approval in a letter from J. H. Taylor on behalf of the B&W Owners Group Materials Committee, and was approved by the NRC staff in a letter from C. O. Thomas to J. H. Taylor dated March 13, 1985. Deletion of these TSs was suggested in an NRC letter to the B&W Owners Group dated May 8, 1985.

Any revision to the CR-3 capsule withdrawal schedule must be documented by the B&W Owners Group Materials Committee in B&W Topical Report BAW-1543. Since NRC staff approval of the topical report is required prior to implementing any changes to the CR-3 capsule withdrawal schedule and the licensee has a representative on the Materials Committee, the NRC staff and the licensee can monitor and maintain control of the reactor vessel material surveillance program in accordance with the requirements of Appendix H, 10 CFR 50.

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Accordingly, TS Section 4.4.9.1.2 and Table 4.4-5 may be deleted from the CR-3 TSs because:

- the licensee will monitor and maintain control of its surveillance program through its participation in the B&W Owners Group Materials Committee,
- changes to the capsule withdrawal schedule will be documented by the B&W Owners Group Materials Committee in B&W Topical Report BAW-1543, and
- NRC staff approval of the topical report is required prior to implementing any changes to the withdrawal schedule.

Environmental Consideration

This amendment involves a change in surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 29, 1985

Principal contributor: B. Elliot