August 10, 1984

DCR 016

Docket No. 50-302

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Mr. Walter S. Wilgus
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
and Fuel Management
P. O. Box 14042; M.A.C. H-2
St. Petersburg, Florida 33733

Dear Mr. Wilgus:

The Commission has issued Amendment No. 70 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated March 18, 1983, and as subsequently revised by your letter dated January 17, 1984.

This amendment provides for a more realistic method for controlling personnel access to high radiation areas where no enclosures exist which would allow locking of a high radiation boundary and where such enclosures could not reasonably be constructed. The Crystal River Unit 3 TSs heretofor required locked doors to be provided to prevent unauthorized entry into high radiation areas. This amendment adopts a provision of the Commission's Standard Technical Specifications, which allows roping off an area, conspicuously posting the area, and installing a flashing light in cases where a lockable enclosure cannot reasonably be constructed around a high radiation area.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Monthly Federal Register Notice.

Sincerely,

/S/

Harley Silver, Project Manager
Operating Reactors Branch No. 4
Division of Licensing

Enclosures:
1. Amendment No. 70
2. Safety Evaluation

cc w/enclosures:
See next page

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08/2/84	08/5/84	08/7/84	08/5/84	08/10/84

cc w/enclosure(s):

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Board of County Commissioners
Citrus County
Inverness, Florida 36250

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.70
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated March 18, 1983, and as supplemented on January 17, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 70 , are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W Rivenbark for

John F. Stolz, Chief
Operating Reactors Branch No. 4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 10, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 70

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page
6-20

6.12 HIGH RADIATION AREA (Continued)

b. A High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1a above, and in addition locked doors shall be provided to prevent unauthorized entry into such area. The keys shall be maintained under the administrative control of the Health Physics Supervisor with one key assigned to the administrative control of Shift Supervisor on duty.

Individual areas that are accessible to personnel, with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem,** and that are located within large areas such as the Reactor Building where no enclosure exists for purposes of locking and no enclosure can be reasonably constructed around the individual area, shall be roped off and conspicuously posted, and a flashing light shall be activated as a warning device.

6.13 ENVIRONMENTAL QUALIFICATION

6.13.1 By no later than June 30, 1982, all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class 1E Electrical Equipment in Operating Reactors" (DOR Guidelines) or NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," December, 1979. Copies of these documents are attached to Order for Modification of License DPR-72 dated October 24, 1980.

6.13.2 By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

**Measurement made at 18" from source of radioactivity.

Order dtd 10/24/80, Amendment No. 80

6.14 PROCESS CONTROL PROGRAM (PCP)

6.14.1 The PCP shall be approved by the Commission prior to implementation.

6.14.2 Licensee initiated changes to the PCP:

1. Shall be submitted to the Commission in the Semi-annual Radioactive Effluent Release Report for the period in which the change(s) was made.

2. Shall become effective upon review and acceptance by the Plant Review Committee.

6.15 OFFSITE DOSE CALCULATION MANUAL (ODCM)

6.15.1 The OFFSITE DOSE CALCULATION MANUAL (ODCM) shall be approved by the Commission prior to implementation.

6.15.2 Licensee initiated changes to the OFFSITE DOSE CALCULATION MANUAL (ODCM):

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made effective.

2. Shall become effective upon review and acceptance by the Plant Review Committee.

6.16 MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS (Liquid, Gaseous and Solid)

6.16.1 Licensee initiated major changes* to the radioactive waste systems (liquid, gaseous and solid):

1. Shall be reported to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the evaluation was reviewed by the Plant Review Committee or be included as part of the annual FSAR update.

2. May be implemented upon review and acceptance by the Plant Review Committee.

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* A major change to a radioactive waste system shall be any change which would alter the ability of the plant or system to meet the requirements of 10 CFR 50, Appendix I.

6.12 HIGH RADIATION AREA (Continued)

b. A High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1a above, and in addition locked doors shall be provided to prevent unauthorized entry into such area. The keys shall be maintained under the administrative control of the Health Physics Supervisor with one key assigned to the administrative control of Shift Supervisor on duty.

Individual areas that are accessible to personnel, with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem,** and that are located within large areas such as the Reactor Building where no enclosure exists for purposes of locking and no enclosure can be reasonably constructed around the individual area, shall be roped off and conspicuously posted, and a flashing light shall be activated as a warning device.

6.13 ENVIRONMENTAL QUALIFICATION

6.13.1 By no later than June 30, 1982, all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class 1E Electrical Equipment in Operating Reactors" (DOR Guidelines) or NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," December, 1979. Copies of these documents are attached to Order for Modification of License DPR-72 dated October 24, 1980.

6.13.2 By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

**Measurement made at 18" from source of radioactivity.

Order/dtd//10/24/80,
Amendment No. 89 to

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO.70 TO FACILITY OPERATING LICENSE NO. DPR-72

FLORIDA POWER CORPORATION, ET AL.

CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT

DOCKET NO. 50-302

Introduction

By letter dated March 18, 1983, and as subsequently revised by letter dated January 17, 1984, Florida Power Corporation (FPC or the licensee) proposed a change to the Crystal River Unit 3 (CR-3) Technical Specifications (TSs). This change specifies positive controls required to control personnel entry into high radiation areas of greater than 1,000 millirem per hour (mrem/hr) which are located in large areas where no enclosure can be reasonably constructed around the individual area.

Background

Entry into high radiation areas requires positive control of personnel within those areas. Conditions for entry should be based on good radiation protection practices and should provide reasonable protection of personnel. The Standard Technical Specifications (STSs) for high radiation area access control clearly address the manner in which positive control for entry into high radiation areas should be exercised.

The licensee proposed amending Section 6.12.1(b) of the TSs in accordance with the STSs to address procedures for entry into high radiation areas (those areas having radiation levels of greater than 1000 mrem/hr) which are located in large areas where no enclosure can be reasonably constructed. Heretofore, no provision existed in the CR-3 TSs, thereby requiring all such high radiation areas to be controlled by locked doors. This requirement is impractical and could result in unwarranted radiation exposure to personnel as the result of building temporary enclosures that can be locked.

The licensee, in the March 18, 1983 application, requested inclusion of the STS provisions in lieu of certain administrative and monitoring requirements presently in the TSs. To provide a higher degree of protection to personnel and to be more consistent with the STSs, we required the licensee to maintain the previous requirements and implement the alternatives to locking these areas in addition to the administrative and monitoring requirements. The licensee agreed to this modified version of the proposed TS change. A Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing was published in the Federal Register on December 21, 1983. The licensee documented the modified version of the request in their January 17, 1984, submittal.

Evaluation

As provided by the amended TSs, areas with radiation levels greater than 1,000 mrem/hr which are located within large areas, such as the Reactor Building, where no enclosure exists for purposes of locking, and for which no enclosure can be reasonably constructed around the individual area, will be roped off, conspicuously posted, and a flashing light will be activated as a warning device. Entrance to such areas shall be controlled by issuance of a Radiation Work Permit (RWP). In addition, any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area. This proposed change will provide controls equivalent to those contained in the STS. The Crystal River 3 TSs, as amended, provide clear, definitive conditions for positive access control for entry into high radiation areas, and therefore, are acceptable.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 10, 1984

Principal Contributors: C. Hinson, R. Hernan