

FEBRUARY 3, 1978

Docket No. 50-302

Florida Power Corporation
ATTN: Mr. W. P. Stewart
Director, Power Production
P. O. Box 14042, Mail Stop C-4
St. Petersburg, Florida 33733

Gentlemen:

The Commission has issued the enclosed Amendment No. 13 to Facility Operating License No. DPR-72 for Crystal River Unit No. 3. The amendment incorporates Technical Specifications regarding the existing fire protection systems and administrative controls at your facility.

By letter dated July 11, 1977, you proposed Technical Specifications regarding your facility's fire protection program. Based on our review, we determined that revisions to your proposed Technical Specifications were necessary to meet our requirements. By letter dated November 29, 1977, we sent you the revised proposed Technical Specifications and supporting Safety Evaluation and stated our intent to issue those specifications. In our letter we also asked that you inform us within 20 days of any specific requirements to which you objected.

In your response dated December 19, 1977, you objected to Section 6.2.2.f of the revised specifications which requires a minimum 5 members of the Fire Brigade. In order to achieve expeditious implementation of the fire protection Technical Specifications, which add to your license requirements regarding your fire protection program, Specification 6.2.2.f is being issued at this time with the minimum number of onsite fire brigade members specified as 4 as you proposed. This number is less than the minimum number given in the generic staff position, Minimum Fire Brigade Shift Size, which was an attachment to the Safety Evaluation Report issued with our letter to you dated November 29, 1977. However, we are presently evaluating your justification for this smaller brigade size and when the evaluation is completed the minimum number will be increased if we do not agree with your position.

DISTRIBUTION

Docket

NRC PDR

L PDR

ORB#4 Reading

VStello

KRGoller

RIngram

CNelson

OELD

OI&E(5)

BJones(4)

BScharf(10)

JMcGough

BHarless

DEisenhut

ACRS(16)

OPA, Clare Miles

DRoss

TABernathy

JBuchanan

file

XTRA Cy 4

Twambach

*Const. 1
60*

OFFICE >						
SURNAME >						
DATE >						

In order to provide a period of time to modify procedures to conform with the details of the enclosed interim Technical Specifications and to complete required personnel training where necessary, the amendment becomes effective 30 days after the date of issue.

A copy of the Notice of Issuance is enclosed. A copy of the Safety Evaluation supporting this amendment was enclosed with our letter dated November 29, 1977.

Sincerely,

Original signed by
Robert W. Reid

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 13
- 2. Notice

cc w/enclosures:
See next page

[Handwritten initials]
[Handwritten note in a circle: All notes re notice]

OFFICE →	ORB#4:DOR	ORB#4:DOR	OELD	C-ORB#4:DOR	ORB#1:DOR
SURNAME →	RIngram <i>rw</i>	CNeelson <i>dn</i>	L.Brenna <i>ls</i>	RWReid	TW... <i>[unclear]</i>
DATE →	1/19/78	1/25/78	1/24/78	1/3/78	2/1/78

Florida Power Corporation

cc w/enclosure(s):

Mr. S. A. Brandimore
Vice President and General Counsel
P. O. Box 14042
St. Petersburg, Florida 33733

Mr. Wilbur Langely, Chairman
Board of County Commissioners
Citrus County
Iverness, Florida 36250

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N.E.
Atlanta, Georgia 30308

Chief, Energy Systems Analyses
Branch (AM-459)
Office of Radiation Programs
U. S. Environmental Protection Agency
Room 645, East Tower
401 M Street, S.W.
Washington, D. C. 20460

Crystal River Public Library
Crystal River, Florida 32629

cc w/enclosures and incoming
dtd: 7/11/77
Bureau of Intergovernmental Relations
660 Apalchee Parkway
Tallahassee, Florida 32304



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF Ocala
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 13
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated July 11, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

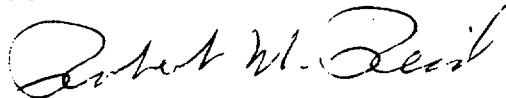
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 13, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective 30 days after the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 3, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 13

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

IV

VII

XII

3/4 3-40 & 3/4 3-41 (new pages)

3/4 7-38 through 3/4 7-47 (new pages)

B 3/4 3-3

B 3/4 7-6

6-1 through 6-3

6-5

6-10

6-12

6-17

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
3/4.0 APPLICABILITY.....	3/4 0-1
<u>3/4.1 REACTIVITY CONTROL SYSTEMS</u>	
3/4.1.1 BORATION CONTROL	
Shutdown Margin.....	3/4 1-1
Boron Dilution	3/4 1-3
Moderator Temperature Coefficient.....	3/4 1-4
Minimum Temperature for Criticality.....	3/4 1-5
3/4.1.2 BORATION SYSTEMS	
Flow Paths - Shutdown.....	3/4 1-6
Flow Paths - Operating.....	3/4 1-7
Makeup Pump - Shutdown.....	3/4 1-9
Makeup Pumps - Operating.....	3/4 1-10
Decay Heat Removal Pump - Shutdown.....	3/4 1-11
Boric Acid Pump - Shutdown.....	3/4 1-12
Boric Acid Pumps - Operating.....	3/4 1-13
Borated Water Sources - Shutdown.....	3/4 1-14
Borated Water Sources - Operating.....	3/4 1-16
3/4.1.3 MOVABLE CONTROL ASSEMBLIES	
Group Height - Safety and Regulating Rod Groups.....	3/4 1-18
Group Height - Axial Power Shaping Rod Group.....	3/4 1-20
Position Indicator Channels.....	3/4 1-21
Rod Drop Time.....	3/4 1-23
Safety Rod Insertion Limit.....	3/4 1-24
Regulating Rod Insertion Limits.....	3/4 1-25
Rod Program.....	3/4 1-33
Xenon Reactivity.....	3/4 1-36

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.2 POWER DISTRIBUTION LIMITS</u>	
3/4.2.1 AXIAL POWER IMBALANCE.....	3/4 2-1
3/4.2.2 NUCLEAR HEAT FLUX HOT CHANNEL FACTOR - F_Q	3/4 2-4
3/4.2.3 NUCLEAR ENTHALPY RISE HOT CHANNEL FACTOR - $F_{\Delta H}^N$	3/4 2-6
3/4.2.4 QUADRANT POWER TILT.....	3/4 2-8
3/4.2.5 DNB PARAMETERS.....	3/4 2-12
<u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 REACTOR PROTECTION SYSTEM INSTRUMENTATION.....	3/4 3-1
3/4.3.2 ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION.....	3/4 3-9
3/4.3.3 MONITORING INSTRUMENTATION	
Radiation Monitoring Instrumentation.....	3/4 3-22
Incore Detectors.....	3/4 3-26
Seismic Instrumentation.....	3/4 3-28
Meteorological Instrumentation.....	3/4 3-31
Remote Shutdown Instrumentation.....	3/4 3-34
Post-accident Instrumentation.....	3/4 3-37
Fire Detection Instrumentation.....	3/4 3-40
<u>3/4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1 REACTOR COOLANT LOOPS.....	3/4 4-1
3/4.4.2 SAFETY VALVES - SHUTDOWN.....	3/4 4-3
3/4.4.3 SAFETY VALVES - OPERATING.....	3/4 4-4

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.7 PLANT SYSTEMS</u>	
3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION.....	3/4 7-13
3/4.7.3 CLOSED CYCLE COOLING WATER SYSTEM	
Nuclear Services Closed Cycle Cooling System.....	3/4 7-14
Decay Heat Closed Cycle Cooling Water System.....	3/4 7-15
3/4.7.4 SEA WATER SYSTEM	
Nuclear Services Sea Water System.....	3/4 7-16
Decay Heat Sea Water System.....	3/4 7-17
3/4.7.5 ULTIMATE HEAT SINK.....	3/4 7-18
3/4.7.6 FLOOD PROTECTION.....	3/4 7-19
3/4.7.7 CONTROL ROOM EMERGENCY VENTILATION SYSTEM.....	3/4 7-20
3/4.7.8 AUXILIARY BUILDING VENTILATION EXHAUST SYSTEM.....	3/4 7-23
3/4.7.9 HYDRAULIC SNUBBERS.....	3/4 7-25
3/4.7.10 SEALED SOURCE CONTAMINATION.....	3/4 7-35
3/4.7.11 FIRE SUPPRESSION SYSTEMS	
Water System.....	3/4 7-38
Deluge and Sprinkler Systems.....	3/4 7-41
Halon System.....	3/4 7-44
Fire Hose Stations.....	3/4 7-45
3/4.7.12 PENETRATION FIRE BARRIERS.....	3/4 7-47
 <u>3/4.8 ELECTRICAL POWER SYSTEMS</u>	
3/4.8.1 A.C. SOURCES	
Operating	3/4 8-1
Shutdown.....	3/4 8-6
3/4.8.2 ONSITE POWER DISTRIBUTION SYSTEMS	
A.C. Distribution - Operating.....	3/4 8-7
A.C. Distribution - Shutdown.....	3/4 8-9
D.C. Distribution - Operating.....	3/4 8-10
D.C. Distribution - Shutdown.....	3/4 8-12

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.9 REFUELING OPERATIONS</u>	
3/4.9.1 BORON CONCENTRATION.....	3/4 9-1
3/4.9.2 INSTRUMENTATION.....	3/4 9-2
3/4.9.3 DECAY TIME.....	3/4 9-3
3/4.9.4 CONTAINMENT PENETRATIONS.....	3/4 9-4
3/4.9.5 COMMUNICATIONS.....	3/4 9-5
3/4.9.6 FUEL HANDLING BRIDGE OPERABILITY.....	3/4 9-6
3/4.9.7 CRANE TRAVEL - SPENT FUEL STORAGE POOL BUILDING.....	3/4 9-7
3/4.9.8 COOLANT CIRCULATION.....	3/4 9-8
3/4.9.9 CONTAINMENT PURGE AND EXHAUST ISOLATION SYSTEM.....	3/4 9-9
3/4.9.10 WATER LEVEL - REACTOR VESSEL.....	3/4 9-10
3/4.9.11 STORAGE POOL.....	3/4 9-11
3/4.9.12 STORAGE POOL VENTILATION.....	3/4 9-12
<u>3/4.10 SPECIAL TEST EXCEPTIONS</u>	
3/4.10.1 GROUP HEIGHT, INSERTION AND POWER DISTRIBUTION LIMITS.....	3/4 10-1
3/4.10.2 PHYSICS TESTS.....	3/4 10-2
3/4.10.3 NO FLOW TESTS.....	3/4 10-3
3/4.10.4 SHUTDOWN MARGIN.....	3/4 10-4

INDEX

BASES

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.6 CONTAINMENT SYSTEMS</u>	
3/4.6.1 PRIMARY CONTAINMENT.....	B 3/4 6-1
3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS.....	B 3/4 6-3
3/4.6.3 CONTAINMENT ISOLATION VALVE.....	B 3/4 6-4
3/4.6.4 COMBUSTIBLE GAS CONTROL.....	B 3/4 6-4

INDEX

BASES

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.7 PLANT SYSTEMS</u>	
3/4.7.1 TURBINE CYCLE.....	B 3/4 7-1
3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION..	B 3/4 7-3
3/4.7.3 CLOSED CYCLE COOLING WATER SYSTEM.....	B 3/4 7-3
3/4.7.4 SEA WATER SYSTEM.....	B 3/4 7-3
3/4.7.5 ULTIMATE HEAT SINK.....	B 3/4 7-4
3/4.7.6 FLOOD PROTECTION.....	B 3/4 7-4
3/4.7.7 CONTROL ROOM EMERGENCY VENTILATION SYSTEM.....	B 3/4 7-4
3/4.7.8 AUXILIARY BUILDING VENTILATION EXHAUST SYSTEM....	B 3/4 7-5
3/4.7.9 HYDRAULIC SNUBBERS.....	B 3/4 7-5
3/4.7.10 SEALED SOURCE CONTAMINATION.....	B 3/4 7-6
3/4.7.11 FIRE SUPPRESSION SYSTEMS.....	B 3/4 7-6
3/4.7.12 PENETRATION FIRE BARRIERS.....	B 3/4 7-6
<u>3/4.8 ELECTRICAL POWER SYSTEMS</u>	
3/4.8.1 A.C. SOURCES.....	B 3/4 8-1
3/4.8.2 ONSITE POWER DISTRIBUTION SYSTEMS.....	B 3/4 8-1
<u>3/4.9 REFUELING OPERATIONS</u>	
3/4.9.1 BORON CONCENTRATION.....	B 3/4 9-1
3/4.9.2 INSTRUMENTATION.....	B 3/4 9-1
3/4.9.3 DECAY TIME.....	B 3/4 9-1
3/4.9.4 CONTAINMENT PENETRATIONS.....	B 3/4 9-1
3/4.9.5 COMMUNICATIONS.....	B 3/4 9-1

CRYSTAL RIVER - UNIT 3

3/4 3-39

TABLE 4.3-7

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Power Range Nuclear Flux	M	Q*
2. Reactor Building Pressure	M	R
3. Source Range Nuclear Flux	M	R*
4. Reactor Coolant Outlet Temperature	M	R
5. Reactor Coolant Total Flow Rate	M	R
6. RC Loop Pressure	M	R
7. Pressurizer Level	M	R
8. Steam Generator Outlet Pressure	M	R
9. Steam Generator Level	M	R
10. Borated Water Storage Tank Level	M	R
11. Startup Feedwater Flow Rate	M	R

* Neutron detectors may be excluded from CHANNEL CALIBRATION.

INSTRUMENTATION

FIRE DETECTION INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.7 As a minimum, the fire detection instrumentation for each fire detection zone shown in Table 3.3-11 shall be OPERABLE.

APPLICABILITY: Whenever equipment in that fire detection zone is required to be OPERABLE.

ACTION:

With one or more of the fire detection instrument(s) shown in Table 3.3-11, inoperable:

- a. Within 1 hour, establish a fire watch patrol to inspect the zone(s) with the inoperable instrument(s) at least once per hour, and
- b. Restore the inoperable instrument(s) to OPERABLE status within 14 days or in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the malfunction and the plans and schedule for restoring the instrument(s) to OPERABLE status.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.7.1 Each of the above fire detection instruments shall be demonstrated OPERABLE at least once per 6 months by performance of a CHANNEL FUNCTIONAL TEST.

4.3.3.7.2 The circuitry associated with the detector alarms listed in Table 3.3-11 shall be demonstrated OPERABLE at least once per 6 months for all National Fire Protection Association (NFPA) Code 72D Class B supervised circuits.

4.3.3.7.3 The non-supervised circuits between the local panels and the control room for the detectors listed in Table 3.3-11 shall be demonstrated OPERABLE at least once per 31 days.

TABLE 3.3-11
FIRE DETECTION INSTRUMENTS

<u>DETECTOR LOCATION</u>	<u>MINIMUM DETECTORS OPERABLE HEAT/SMOKE</u>
1. Control Complex	
a. Elevation 108'0"	
1. Zone 4 (Plant Battery Room 3B)	NA/1
2. Zone 5 (Plant Battery Room 3A)	NA/1
3. Zone 6 (Battery Charger Room 3B)	NA/1
4. Zone 7 (Battery Charger Room 3A)	NA/1
5. Zone 8 (4160V Switchgear Bus Room 3B)	NA/1
6. Zone 9 (4160V Switchgear Bus Room 3A)	NA/1
7. Zone 10 (Inverter Room 3B)	NA/1
8. Zone 11 (Inverter Room 3A)	NA/1
b. Elevation 120'0"	
1. Zone 5 (Control Rod Drive Equipment Room)	NA/2
2. Zone 7 (480V Switchgear Bus Room 3B)	NA/1
3. Zone 8 (480V Switchgear Bus Room 3A)	NA/1
c. Elevation 134'0'	
1. Zone 3A (Cable Spreading Room)	NA/5
or	
2. Zone 3B (Cable Spreading Room)	NA/3
d. Elevation 145'0"	
1. Zone 4 (Satellite Instrument Shop and Office)	NA/2
2. Zone 5 (Control Room)	1/6
e. Elevation 164'0"	
1. Zone 3 (HVAC Equipment Room)	NA/5
2. Zone 4 (HVAC Emergency Equipment 3B)	NA/1
3. Zone 5 (HVAC Emergency Equipment 3A)	NA/1

2. Auxiliary Building

a. Elevation 119'0"	
1. Zone 20 (Emergency Diesel Generator 3B Controls Room)	1/NA
2. Zone 21 (Emergency Diesel Generator 3A Controls Room)	1/NA
3. Zone 27 (Emergency Diesel Generator Room 3B)	5/NA
4. Zone 28 (Emergency Diesel Generator Room 3A)	5/NA

SURVEILLANCE REQUIREMENTS (Continued)

1. With a half-life greater than 30 days (excluding Hydrogen 3) and
 2. In any form other than gas.
- b. Stored sources not in use - Each sealed source and fission detector shall be tested prior to use or transfer to another licensee unless tested within the previous six months. Sealed sources and fission detectors transferred without a certificate indicating the last test date shall be tested prior to being placed into use.
 - c. Startup sources and fission detectors - Each sealed startup source and fission detector shall be tested within 31 days prior to being subjected to core flux or installed in the core and following repair or maintenance to the source.
- 4.7.10.1.3 Reports - A report shall be prepared and submitted to the Commission on an annual basis if sealed source or fission detector leakage tests reveal the presence of ≥ 0.005 microcuries of removable contamination.

PLANT SYSTEMS

3/4.7.11 FIRE SUPPRESSION SYSTEMS

FIRE SUPPRESSION WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.11.1 The fire suppression water system shall be OPERABLE with:

- a. At least two high-pressure pumps each with a capacity of 2000 gpm, with their discharge aligned to the fire suppression header.
- b. Separate water supplies, each with a minimum contained water volume of 345,000 gallons.
- c. An OPERABLE flow path capable of taking suction from the water supply and transferring the water through distribution piping with OPERABLE sectionalizing control or isolation valves to the yard hydrant curb valves and the front valve ahead of the water flow alarm device on each sprinkler, hose standpipe and spray system riser required to be OPERABLE per Specifications 3.7.11.2 and 3.7.11.4.

APPLICABILITY: At all times.

ACTION:

- a. With one pump and/or one water supply inoperable, restore the inoperable equipment to OPERABLE status within 7 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the plans and procedures to be used to provide for the loss of redundancy in this system. The provisions of Specification 3.0.3 and 3.0.4 are not applicable.
- b. With the fire suppression water system otherwise inoperable:
 1. Establish a backup fire suppression water system within 24 hours, and
 2. Submit a Special Report in accordance with Specification 6.9.2;
 - a) By telephone within 24 hours,
 - b) Confirm by telegraph, mailgram, or facsimile transmission no later than the first working day following the event, and
 - c) In writing within 14 days following the event, outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS

4.7.11.1 The fire suppression water system shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying the contained water supply volume.
- b. At least once per 31 days on a STAGGERED TEST BASIS by starting each pump and operating it for at least 15 minutes on recirculation flow.
- c. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path is in its correct position.
- d. At least once per 12 months by cycling each testable valve in the flow path through at least one complete cycle of full travel.
- e. At least once per 18 months by performing a system functional test which includes simulated automatic actuation of the system throughout its operating sequence, and:
 1. Verifying that each automatic valve in the flow path actuates to its correct position,
 2. Verifying that each pump develops at least 2000 gpm at a discharge pressure of ≥ 115 psig,
 3. Cycling each valve in the flow path that is not testable during plant operation through at least one complete cycle of full travel, and
 4. Verifying that each high pressure pump starts (sequentially) to maintain the fire suppression water system pressure ≥ 70 psig.
- f. At least once per 3 years by performing flow tests of the system in accordance with Chapter 5, Section 11 of Fire Protection Handbook, 14th Edition (January 1976) published by National Fire Protection Association.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Cont.)

- g. By demonstrating the fire pump diesel engines OPERABLE:
1. At least once per 31 days by verifying:
 - (a) The fuel storage tank contains at least 175 gallons of fuel, and
 - (b) The diesel starts from ambient conditions and operates for at least 20 minutes.
 2. At least once per 92 days by verifying that a sample of diesel fuel from the fuel storage tank, obtained in accordance with ASTM-D270-65, is within the acceptable limits specified in Table 1 of ASTM D975-74 with respect to viscosity, water content and sediment for the type of fuel specified for use in the diesels.
 3. At least once per 18 months, during shutdown, by:
 - (a) Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for the class of service, and
 - (b) Verifying the diesel starts from ambient conditions on the auto-start signal and operates for \geq 20 minutes while loaded with the fire pump.
- h. By demonstrating the fire pump diesel starting 24-volt battery banks and chargers OPERABLE:
1. At least once per 7 days by verifying that:
 - (a) The electrolyte level of each battery is above the plates, and
 - (b) The overall battery voltage is \geq 24 volts.
 2. At least once per 92 days by verifying that the specific gravity is appropriate for continued service of the battery.
 3. At least once per 18 months by verifying that:
 - (a) The batteries, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration, and
 - (b) The battery-to-battery and terminal connections are clean, tight, free of corrosion and coated with anti-corrosion material.

PLANT SYSTEMS

DELUGE AND SPRINKLER SYSTEMS

LIMITING CONDITION FOR OPERATION

3.7.11.2 The deluge and sprinkler systems shown in Table 3.7-4 shall be OPERABLE.

APPLICABILITY: Whenever equipment in the deluge/sprinkler protected areas is required to be OPERABLE.

ACTION:

- a. With one or more of the above required deluge and sprinkler systems inoperable, establish a continuous fire watch with backup fire suppression equipment for the unprotected area(s) within 1 hour; restore the system to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- b. The provisions of Specification 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.11.2 Each of the above required deluge and sprinkler systems shall be demonstrated OPERABLE:

- a. At least once per 12 months by cycling each testable valve in the flow path through at least one complete cycle of full travel.
- b. At least once per 18 months:
 1. By performing a system functional test which includes simulated automatic actuation of the system, and:
 - a) Verifying that the automatic valves in the flow path actuate to their correct positions.

TABLE 3.7-4

DELUGE AND SPRINKLER SYSTEMS

<u>SYSTEM</u>	<u>LOCATION</u>
1. AHFL-1 Deluge System	Auxiliary Building, Elevation 143'0", Zone 11 (Main Filter Exhaust Room)
2. AHFL-2A Deluge System	Auxiliary Building, Elevation 143'0", Zone 11 (Main Filter Exhaust Room)
3. AHFL-2B Deluge System	Auxiliary Building, Elevation 143'0", Zone 11 (Main Filter Exhaust Room)
4. AHFL-2C Deluge System	Auxiliary Building, Elevation 143'0", Zone 11 (Main Filter Exhaust Room)
5. AHFL-2D Deluge System	Auxiliary Building, Elevation 164'0", Zone 5 (Main Filter Exhaust Room)
6. AHFL-4A Deluge System	Control Complex, Elevation 164'0", Zone 5 (HVAC Emergency Equipment 3A)
7. AHFL-4B Deluge System	Control Complex, Elevation 164'0", Zone 4 (HVAC Emergency Equipment 3B)
8. AHFL-15 Deluge System	Auxiliary Building, Elevation 95'0", Zone 34 (Nuclear Sampling Room)
9. Emergency Diesels Deluge	Auxiliary Building, Elevation 119'0", Zones 27 and 28

PLANT SYSTEMS

HALON SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.11.3 The Halon system in the Cable Spreading room (Control Complex, Elevation 134'0") shall be OPERABLE with the storage tanks having at least 95% of full charge weight and 90% of full charge pressure.

APPLICABILITY: At all times

ACTION:

- a. With the above required Halon system inoperable, establish a continuous fire watch with backup fire suppression equipment for the unprotected area within 1 hour; restore the system to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.11.3 The Halon system shall be demonstrated OPERABLE:

- a. At least once per 6 months by verifying each Halon storage tank weight and pressure.
- b. At least once per 18 months by
 1. Verifying the system, including associated ventilation dampers, would actuate automatically to a simulated test signal.
 2. Verifying the OPERABILITY of the manual initiating system.
 3. Performance of a flow test through headers and nozzles to assure no blockage.

PLANT SYSTEMS

FIRE HOSE STATIONS

LIMITING CONDITION FOR OPERATION

3.7.11.4 The fire hose stations shown in Table 3.7-5 shall be OPERABLE.

APPLICABILITY: Whenever equipment in the areas protected by the fire hose stations is required to be OPERABLE.

ACTION:

- a. With one or more of the fire hose stations shown in Table 3.7-5 inoperable, route an equivalent capacity fire hose to the unprotected area(s) from an OPERABLE hose station within 1 hour.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.11.4 Each of the fire hose stations shown in Table 3.7-5 shall be demonstrated OPERABLE:

- a. At least once per 31 days by:
 1. Visual inspection of the station to assure all required equipment is at the station, and
- b. At least once per 18 months by:
 1. Removing the hose for inspection and re-racking, and
 2. Replacement of all degraded gaskets in couplings.
- c. At least once per 3 years by:
 1. Partially opening each hose station valve to verify valve OPERABILITY and no flow blockage.
 2. Conducting a hose hydrostatic test at a pressure at least 50 psig greater than the maximum pressure available at that hose station.

TABLE 3.7-5

FIRE HOSE STATIONS

<u>Hose Reel Location</u>	<u>Hose Reel Valve No.</u>
1. Control Complex	
a. Elevation 134'	
1. Zone 4 (Entrance to Cable Spreading Room)	FSV-253
2. Turbine Building	
a. Elevation 145'	
1. Entrance to Control Complex	FSV-155
3. Intermediate Building	
a. Elevation 95'	
1. Zone 3 (Steam Driven Emergency Feedwater Pump)	FSV-170
b. Elevation 119'	
1. Zone 2 (Reactor Building Industrial Cooler Circulating Pumps)	FSV-169
2. Zone 5 (Containment Personnel Airlock)	FSV-134
4. Auxiliary Building	
a. Elevation 95'	
1. Zone 1 (Corridor outside Chem-Rad Area)	FSV-135
2. Zone 5 (ES MCC 3A1)	FSV-143
3. Zone 13 (Elevator)	FSV-138
4. Zone 17 (Nuclear Services Sea Water Pumps)	FSV-139
b. Elevation 119'	
1. Zone 18 (Elevator)	FSV-137
2. Zone 23 (Es MCC 3 A B)	FSV-142
3. Zone 26 (Waste Compactor)	FSV-140
c. Elevation 143'	
1. Zone 9 (Entrance to Charcoal Filter Room)	FSV-131
d. Elevation 162'	
1. Zone 1 (North of Spent Fuel Pools)	FSV-136
2. Zone 3 (Elevator)	FSV-133

PLANT SYSTEMS

3/4.7.12 PENETRATION FIRE BARRIERS

LIMITING CONDITION FOR OPERATION

3.7.12 All penetration fire barriers protecting safety related areas shall be functional.

APPLICABILITY: At all times.

ACTION:

- a. With one or more of the above required penetration fire barriers non-functional, establish a continuous fire watch on at least one side of the affected penetration within 1 hour.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.12 Each of the above required penetration fire barriers shall be verified to be functional by a visual inspection: :

- a. At least once per 18 months, and
- b. Prior to declaring a penetration fire barrier functional following repairs or maintenance.

3/4.3 INSTRUMENTATION

BASES

REMOTE SHUTDOWN INSTRUMENTATION (Continued)

HOT STANDBY of the facility from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with General Design Criterion 19 of Appendix "A", 10 CFR 50.

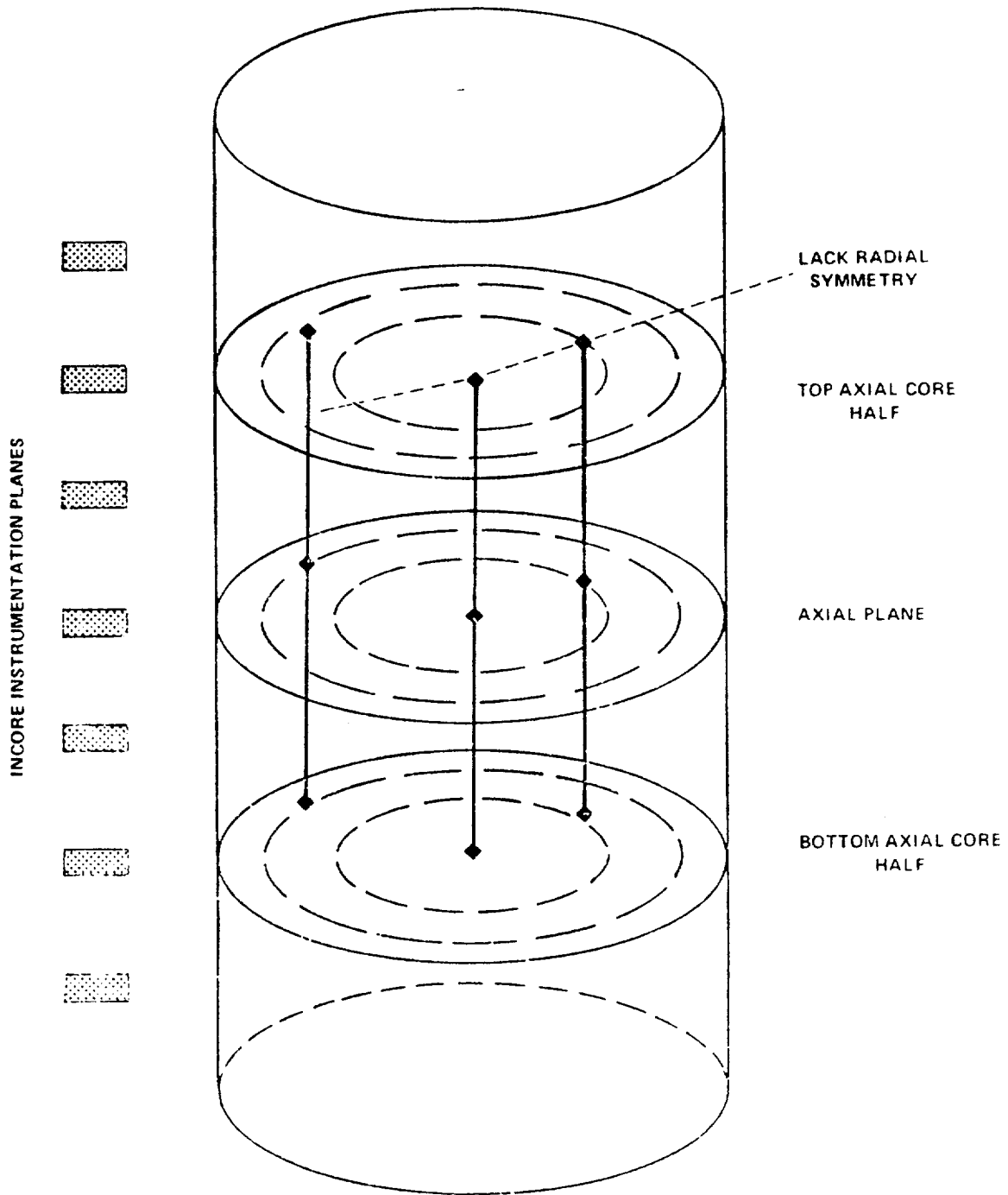
3/4.3.3.6 POST-ACCIDENT INSTRUMENTATION

The OPERABILITY of the post-accident instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light-Water Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident", December 1975.

3/4.3.3.7 FIRE DETECTION INSTRUMENTATION

The OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety related equipment and is an integral element in the overall facility fire protection program.

In the event that a portion of the fire detection instrumentation is inoperable, the establishment of frequent fire patrols in the affected areas is required to provide detection capability until the inoperable instrumentation is returned to service.



Bases Figure 3-1 Incore Instrumentation Specification
 Acceptable Minimum AXIAL POWER IMBALANCE Arrangement

PLANT SYSTEMS

BASES

3/4.7.8 AUXILIARY BUILDING VENTILATION EXHAUST SYSTEM

The OPERABILITY of the auxiliary building ventilation exhaust system ensures that radioactive materials leaking from the ECCS equipment following a LOCA are filtered prior to reaching the environment. The operation of this system and the resultant effect on offsite dosage calculations were assumed in the safety analyses.

3/4.7.9 HYDRAULIC SNUBBERS

The hydraulic snubbers are required OPERABLE to ensure that the structural integrity of the reactor coolant system and all other safety related systems is maintained during and following a seismic or other event initiating dynamic loads. The only snubbers excluded from this inspection program are those installed on nonsafety related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety related system.

The inspection frequency applicable to snubbers containing seals fabricated from materials which have been demonstrated compatible with their operating environment is based upon maintaining a constant level of snubber protection. Therefore, the required inspection interval varies inversely with the observed snubber failures. The number of inoperable snubbers found during an inspection of these snubbers determines the time interval for the next required inspection of these snubbers. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

To provide further assurance of snubber reliability, a representative sample of the installed snubbers will be functionally tested during plant shutdowns at 18 month intervals. These tests will include stroking of the snubbers to verify proper piston movement, lock-up and bleed. Observed failures of these sample snubbers will require functional testing of additional units. To minimize personnel exposures, snubbers installed in high radiation zones or in especially difficult to remove locations may be exempted from these functional testing requirements provided the OPERABILITY of these snubbers was demonstrated during functional testing at either the completion of their fabrication or at a subsequent date.

PLANT SYSTEMS

BASES

3/4.7.10 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from byproduct, source, and special nuclear material sources will not exceed allowable intake values.

3.4.7.11 FIRE SUPPRESSION SYSTEMS

The OPERABILITY of the fire suppression systems ensures that adequate fire suppression capability is available to confine and extinguish fires occurring in any portion of the facility where safety related equipment is located. The fire suppression system consists of the water system, deluge and sprinklers, hose stations and Halon. The collective capability of the fire suppression systems is adequate to minimize potential damage to safety related equipment and is a major element in the facility fire protection program.

In the event that portions of the fire suppression systems are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the affected equipment can be restored to service.

In the event that the fire suppression water system becomes inoperable, immediate corrective measures must be taken since this system provides the major fire suppression capability of the plant. The requirement for a twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued protection of the nuclear plant.

3/4.7.12 PENETRATION FIRE BARRIERS

The functional integrity of the penetration fire barriers ensures that fires will be confined or adequately retarded from spreading to adjacent portions of the facility. This design feature minimizes the possibility of a single fire rapidly involving several areas of the facility prior to detection and extinguishment. The penetration fire barriers are a passive element in the facility fire protection program and are subject to periodic inspection.

During periods of time when the barriers are not functional, a continuous fire watch is required to be maintained in the vicinity of the affected barrier until the barrier is restored to functional status.

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Nuclear Plant Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Nuclear Plant Manager shall be responsible for an annual fire protection inspection which shall be performed utilizing either qualified offsite licensee personnel or an outside fire protection firm. The inspection shall consist of: a) an inspection of safety-related areas of the Plant to verify that they are in conformance with the fire hazards analysis; and b) a review of the Fire Brigade organization, training, and drills to verify their conformance with the requirements of Section 27 of the NFPA Code-1976.

6.1.3 The Nuclear Plant Manager shall be responsible for an inspection of the fire protection program to be performed by a qualified outside fire consultant at least once per 36 months.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A Fire Brigade of at least 4 member shall be maintained onsite at all times. This excludes 5 members of the minimum shift crew necessary for safe shutdown of the plant and personnel required for other essential functions.

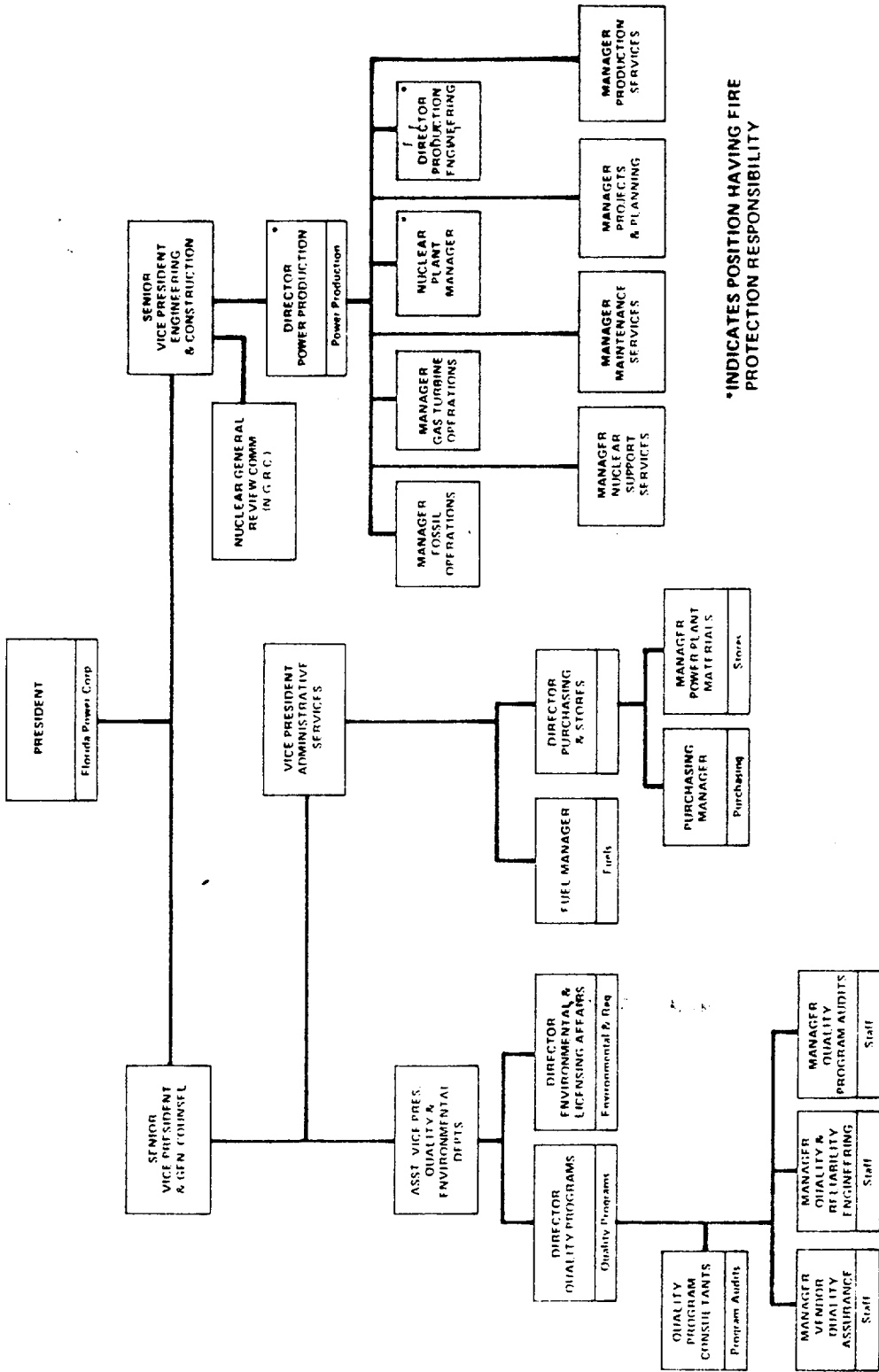


Figure 6.2-1 Offsite Organization

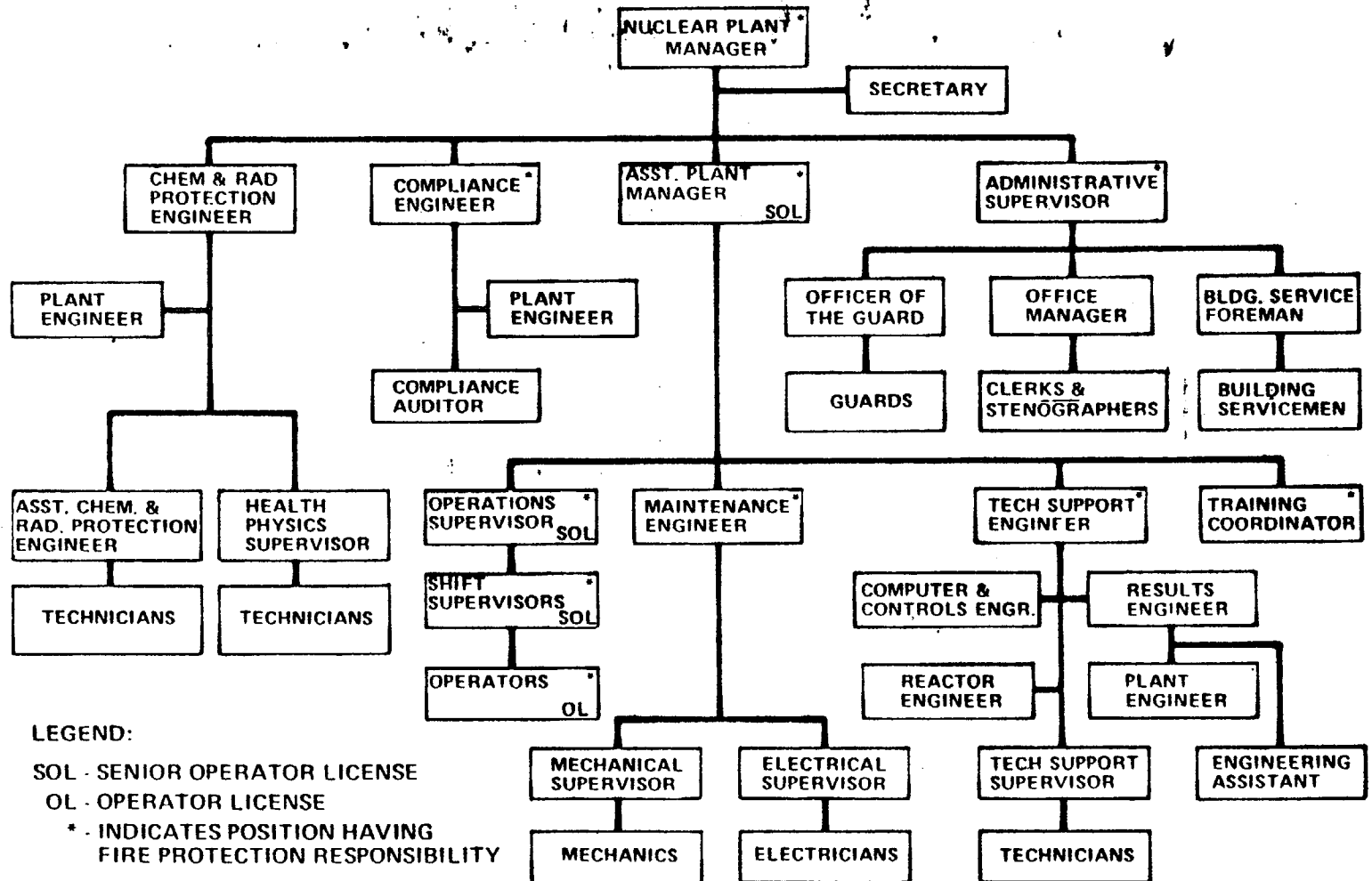


Figure 6.2-2 Facility Organization

TABLE 6.2-1
MINIMUM SHIFT CREW COMPOSITION#

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL	1	1*
OL	2	1
Non-Licensed	3	1

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling Individual supervising CORE ALTERATIONS after the initial fuel loading.

#Shift crew composition may be less than the minimum requirement for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

ADMINISTRATIVE CONTROLS

CONSULTANTS

6.5.2.5 Consultants shall be utilized as determined by the NGRC Chairman to provide expert advice to the NGRC.

MEETING FREQUENCY

6.5.2.6 The NGRC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.7 A quorum of NGRC shall consist of the Chairman or his designated alternate and five additional NGRC members, including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

REVIEW

6.5.2.8 The NGRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.

ADMINISTRATIVE CONTROLS

REVIEW (Continued)

- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Plant Review Committee.

AUDITS

6.5.2.9 Audits of facility activities shall be performed under the cognizance of the NGRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.
- g. The facility fire protection program and implementing procedures at least once per 24 months.
- h. Any other area of facility operation considered appropriate by the NRGC or the Senior Vice President-Engineering and Construction.

AUTHORITY

6.5.2.10 The NGRC shall report to and advise the Senior Vice President-Engineering and Construction on those areas of responsibility specified in Sections 6.5.2.8 and 6.5.2.9.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Protection Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September, 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Assistant Nuclear Plant Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Plant Manager and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW COMMITTEE (PRC)

FUNCTION

6.5.1.1 The Plant Review Committee shall function to advise the Nuclear Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Review Committee shall be composed of the:

Chairman:	Assistant Nuclear Plant Manager
Member:	Operations Supervisor
Member:	Technical Support Engineer
Member:	Maintenance Engineer
Member:	Chemistry and Radiation Protection Engineer

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRC Chairman to serve on a temporary basis; no more than two alternates shall participate as voting members in PRC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRC shall meet at least once per calendar month and as convened by the PRC Chairman or his designated alternate.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 A quorum of the PRC shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Review Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Nuclear Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear Safety.
- c. Review of all proposed changes to the Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Director-Power Production and to the Chairman of the Nuclear General Review Committee.
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Nuclear General Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear General Review Committee.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear General Review Committee.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.2.11 Records of NGRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NRGC meeting shall be prepared, approved and forwarded to the Senior Vice President-Engineering and Construction within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.8 above, shall be prepared, approved and forwarded to the Senior Vice President-Engineering and Construction within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.9 above, shall be forwarded to the Senior Vice-President Engineering and Construction and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the PRC and submitted to the NGRC and the Director-Power Production.

ADMINISTRATIVE CONTROLS

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The Safety Limit violation shall be reported to the Commission, the Director-Power Production and to the NGRC within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRGC and the Director-Power Production within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the PRC and approved by the Nuclear Plant Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

ADMINISTRATIVE CONTROLS

occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- b. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.
- c. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
- d. Abnormal degradation of systems other than those specified in 6.9.1.8.c above, designed to contain radioactive material resulting from the fission process.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement, Region II, within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. ECCS Actuation, Specification 3.5.2 and 3.5.3.
- b. Inoperable Seismic Monitoring Instrumentation, Specification 3.3.3.3.
- c. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- d. Seismic event analysis, Specification 4.3.3.3.2.
- e. Inoperable Fire Detection Monitoring Instrumentation, Specification 3.3.3.7.
- f. Inoperable Fire Suppression System, Specifications 3.7.11.1, 3.7.11.2, 3.7.11.3. and 3.7.11.4.

ADMINISTRATIVE CONTROLS

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time intervals at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. All REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source and fission detector leak tests and results.
- i. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-302

FLORIDA POWER CORPORATION

CITY OF ALACHUA

CITY OF BUSHNELL

CITY OF GAINESVILLE

CITY OF KISSIMMEE

CITY OF LEESBURG

CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH

CITY OF OCALA

ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO

SEBRING UTILITIES COMMISSION

SEMINOLE ELECTRIC COOPERATIVE, INC.

CITY OF TALLAHASSEE

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 13 to Facility Operating License No. DPR-72, issued to the Florida Power Corporation, City of Alachua, City of Bushnell, City of Gainesville, City of Kissimmee, City of Leesburg, City of New Smyrna Beach and Utilities Commission, City of New Smyrna Beach, City of Ocala, Orlando Utilities Commission and City of Orlando, Sebring Utilities Commission, Seminole Electric Cooperative, Inc., and the City of Tallahassee (the licensees) which revised Technical Specifications for operation of the Crystal River Unit No. 3 Nuclear Generating Plant located in Citrus County, Florida. The amendment becomes effective 30 days after its date of issuance.

The amendment incorporates fire protection Technical Specifications on the existing fire protection equipment and adds administrative controls related to fire protection at the facility. This action is

being taken pending completion of the Commission's overall fire protection review of the facility.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.


The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated July 11, 1977, and Florida Power & Light Company's letter dated December 19, 1977, (2) the Commission's letter to the licensee dated November 29, 1977, (3) Amendment No. 13 to License No. DPR-72, and (4) the Commission's related Safety Evaluation issued November 29, 1977. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and at the Crystal River Public

Library, Crystal River, Florida. A copy of items (2) through (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 3rd day of February 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors