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Docket No. 50-302

Mr. Walter S. Wilgus Vice President, Nuclear Operations Florida Power Corporation ATTN: Manager, Nuclear Licensing & Fuel Management Post Office Box 14042; M.A.C. H-2 St. Petersburg, Florida 33733

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Dear Mr. Wilgus:

The Commission has issued the enclosed Amendment No. 74 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in partial response to your application dated May 31, 1984. The remaining portion of the change request pertaining to the Reactor Coolant Vent System will be addressed by separate licensing action.

This amendment consists of changes to the TSs for CR-3 such that the restriction to mode changes is no longer applicable to the Containment Purge and Exhaust Isolation System and the Spent Fuel Storage Pool.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

"CORNAL SIGNED BY."

Harley Silver, Project Manager **Operating Reactors Branch #4** Division of Licensing

Enclosures: 1. Amendment No. 74 to DPR-72 Safety Evaluation 2.

cc w/enclosures: See next page

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PDR

Mr. W. S. Wilgus Florida Power Corporation

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Mr. Wilbur Langely, Chairman Board of County Commissioners Citrus County Inverness, Florida 36250



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 FLORIDA POWER CORPORATION CITY OF ALACHUA CITY OF ALACHUA CITY OF BUSHNELL

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 ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO SEBRING UTILITIES COMMISSION SEBRING UTILITIES COMMISSION SEMINOLE ELECTRIC COOPERATIVE, INC. CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74 License No. DPR-72

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated May 31, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

 Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 74, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief) Operating Reactors Branch #4 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: June 27, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 9-9 3/4 9-11

REFUELING OPERATIONS

CONTAINMENT PURGE AND EXHAUST ISOLATION SYSTEM

1.2.5

LIMITING CONDITION FOR OPERATION

3.9.9 The containment purge and exhaust isolation system shall be OPERABLE.

APPLICABILITY MODE 6.

ACTION:

With the containment purge and exhaust isolation system inoperable, close each of the purge and exhaust penetrations providing direct access from the containment atmosphere to the outside atmosphere. The provisions of Specification 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.9 The containment purge and exhaust isolation system shall be demonstrated OPERABLE within 100 hours prior to the start of and at least once per 7 days during CORE ALTERATIONS by verifying that containment purge and exhaust isolation occurs on manual initiation and on a high radiation test signal from the containment gaseous activity radiation monitoring instrumentation channels.

CRYSTAL RIVER - UNIT 3

3/4 9-9

REFUELING OPERATIONS

WATER LEVEL - REACTOR VESSEL

LIMITING CONDITION FOR OPERATION

3.9.10 As a minimum, 23 feet of water shall be maintained over the top of irradiated fuel assemblies seated within the reactor pressure vessel.

APPLICABILITY: During movement of fuel assemblies or control rods within the reactor pressure vessel while in MODE 6.

ACTION:

With the requirements of the above specification not satisfied, suspend all operation involving movement of fuel assemblies or control rods within the reactor pressure vessel. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.10 The water level shall be determined to be at least its minimum required depth within 2 hours prior to the start of and at least once per 24 hours during movement of fuel assemblies or control rods.

CRYSTAL RIVER - UNIT 3 3/4 9-10

REFUELING OPERATIONS

STORAGE POOL

LIMITING CONDITION FOR OPERATION

3.9.11 All missile shields and at least 23 feet of water shall be maintained over the top of irradiated fuel assemblies seated in the storage racks.

APPLICABILITY Whenever irradiated fuel assemblies are in the storage pool.

ACTION:

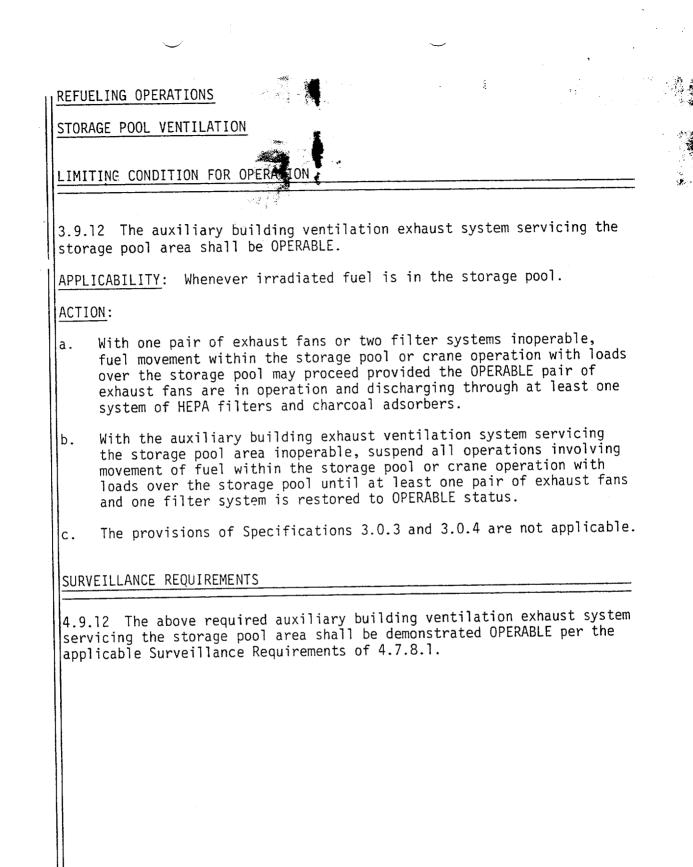
- a. With all missile shields not installed over the storage pool:
 - Immediately install all missile shields upon notification of a Tornado Watch, and
 - 2) Install all missile shields not required to be removed for in-progress handling of fuel assemblies.
- b. With the minimum water level requirement not satisfied, suspend all movement of fuel and crane operations with loads in the fuel storage area and restore the water level to within its limit within 4 hours.
- c. The provisions of Specification 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.11.1 All missile shields shall be determined to be installed over the storage pool when irradiated fuel assemblies are in the fuel storage pool:

- a) Immediately upon notification of a Tornado Watch, and
- b) Upon completion of handling fuel assemblies.

4.9.11.2 The water level in the storage pool shall be determined to be at least its minimum required depth at least once per 7 days when irradiated fuel assemblies are in the fuel storage pool.



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3/4 9-12



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE NO. DPR-72

FLORIDA POWER CORPORATION, ET AL.

CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT

DOCKET NO. 50-302

Introduction

By letter dated May 31, 1984, Florida Power Corporation (the licensee) requested an amendment to Facility Operating License No. DPR-72, revising the Appendix A Technical Specifications to exclude the (1) Reactor Coolant System Vents, (2) Containment Purge and Exhaust Isolation System, and (3) Spent Fuel Storage Pool from the requirement of Section 3.0.4. Only items (2) and (3) above are addressed in this Safety Evaluation; item (1) will be evaluated in a future licensing action.

Section 3.0.4 states that "Entry into an OPERATIONAL MODE or other specified applicability condition shall not be made unless the conditions of the Limiting Condition for Operation are met without reliance on provisions contained in the ACTION statements unless otherwise excepted. This provision shall not prevent passage through OPERATIONAL MODES as required to comply with ACTION statements."

We have reviewed the licensee's request and a summary of our evaluation follows.

Discussion and Evaluation

(1) Reactor Coolant System Vents

Proposed Technical Specifications had previously been submitted by the licensee by letter dated February 24, 1984, as a result of the staff's Generic Letter 83-37. Review of that submittal is ongoing. Parts of the May 31, 1984, submittal serve to revise the February 24, 1984, submittal; we will therefore address these parts in a future evaluation.

(2) Containment Purge and Exhaust Isolation System

The licensee requested that Specification 3.9.9, which addresses Containment Purge and Exhaust Isolation System operability for refueling operations (Mode 6), not be subject to the requirements of Specification 3.0.4. Specification 3.0.4 prohibits entry into operational modes when the Limiting Condition for Operation is not met. Since Specification 3.9.9 permits the Containment Purge and Exhaust Isolation System to be inoperable (with the system penetrations isolated) when in Mode 6, entry

8508010121 850627 PDR ADUCK 05000302 PDR into Mode 6 with the system inoperable will not present a new safety problem. Therefore, revising Specification 3.9.9 to eliminate the mode change restriction is acceptable.

(3) Spent Fuel Storage Pool

The licensee requested that Specification 3.9.11, which addresses the spent fuel storage pool, not be subject to the requirements of Specification 3.0.4. Specification 3.9.11 currently requires that the missile shields and 23 feet of water be maintained over the top of irradiated fuel assemblies in the storage racks. If these two requirements are not satisfied, the reactor cannot be started up from a shutdown until they are met. Separate Technical Specifications governing spent fuel pool safety are provided independent of plant operating mode. Since the state of the storage pool has no impact on the safe operation of the reactor, we conclude that revising Specification 3.9.11, as proposed by the licensee, to eliminate the mode change restriction will not degrade plant safety. Therefore, we conclude that the proposed Technical Specification change is acceptable.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 27, 1985

Principal contributors: J. Guo and R. Anand