

JUL 08 1985

Docket No. 50-302

DISTRIBUTION

Mr. Walter S. Wilgus  
Vice President, Nuclear Operations  
Florida Power Corporation  
ATTN: Manager, Nuclear Licensing  
& Fuel Management  
Post Office Box 14042; M.A.C. H-2  
St. Petersburg, Florida 33733

Docket File  
NRC PDR  
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Dear Mr. Wilgus:

The Commission has issued the enclosed Amendment No. 75 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in partial response to your application dated August 30, 1984, as supplemented on June 17, 1985.

This amendment revises the location of several remote shutdown monitoring instruments from the ES Switchgear Room to the Remote Shutdown Panel. Incorporation of the new Remote Shutdown System into the Technical Specifications will be completed in a separate action.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

**Original signed by**

Harley Silver, Project Manager  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

- 1. Amendment No. 75 to DPR-72
- 2. Safety Evaluation

cc w/enclosures:  
See next page

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JStoltz  
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TWambach  
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Mr. W. S. Wilgus  
Florida Power Corporation

Crystal River Unit No. 3 Nuclear  
Generating Plant

cc:

Mr. R. W. Neiser  
Senior Vice President  
and General Counsel  
Florida Power Corporation  
P. O. Box 14042  
St Petersburg, Florida 33733

Bureau of Intergovernmental Relations  
660 Apalachee Parkway  
Tallahassee, Florida 32304

Mr. Wilbur Langely, Chairman  
Board of County Commissioners  
Citrus County  
Inverness, Florida 36250

Nuclear Plant Manager  
Florida Power Corporation  
P. O. Box 219  
Crystal River, Florida 32629

Mr. Robert B. Borsum  
Babcock & Wilcox  
Nuclear Power Generation Division  
Suite 220, 7910 Woodmont Avenue  
Bethesda, Maryland 20814

Resident Inspector  
U.S. Nuclear Regulatory Commission  
Route #3, Box 717  
Crystal River, Florida 32629

Regional Administrator, Region II  
U.S. Nuclear Regulatory Commission  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

Mr. Uray Clark, Administrator  
Radiological Health Services  
Department of Health and  
Rehabilitative Services  
1323 Winewood Blvd.  
Tallahassee, Florida 32301

Administrator  
Department of Environmental Regulation  
Power Plant Siting Section  
State of Florida  
2600 Blair Stone Road  
Tallahassee, Florida 32301

Attorney General  
Department of Legal Affairs  
The Capitol  
Tallahassee, Florida 32304



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION  
CITY OF ALACHUA  
CITY OF BUSHNELL  
CITY OF GAINESVILLE  
CITY OF KISSIMMEE  
CITY OF LEESBURG  
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH  
CITY OF OCALA  
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO  
SEBRING UTILITIES COMMISSION  
SEMINOLE ELECTRIC COOPERATIVE, INC.  
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75  
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated August 30, 1984, as supplemented June 17, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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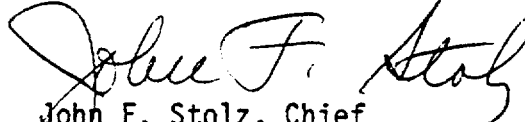
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 75 , are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 3, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page

3/4 3-35

TABLE 3.3-9

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>READOUT LOCATION</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Trip Breaker Indication	CRD switchgear room 124 foot elevation	open-close	1 per trip breaker and 1 per secondary trip breaker
2. Reactor Coolant Temperature - Th	Remote shutdown panel	520-620°F	1 per loop
3. Reactor Coolant Pressure	Remote shutdown panel	0-2500 psig	1
4. Pressurizer Level	Remote shutdown panel	0-320" H <sub>2</sub> O	1
5. Steam Generator Pressure	Remote shutdown panel	0-1200 psig	1 per steam generator
6. Steam Generator Level	4160 ES-B switchgear room 108 foot elevation	0-250" H <sub>2</sub> O	1 per steam generator
7. Decay Heat Removal Temperature	Remote shutdown panel	0-300°F	1 per cooler
8. Motor Driven Emergency Feedwater Pressure	Intermediate Building 95 foot elevation	0-2000 psig	1 per pump
9. Nuclear Services Closed Cycle Cooling Pumps Discharge Pressure	Auxiliary Building 95 foot elevation	0-300 psig	1
10. Nuclear Services Closed Cycle Cooling Cooler Outlet Temperature	Auxiliary Building 95 foot elevation	0-250°F	1 per cooler

TABLE 4.3-6REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Trip Breaker Indication	M	N.A.
2. Reactor Coolant Temperature-Th	M	R
3. Reactor Coolant Pressure	M	R
4. Pressurizer Level	M	R#
5. Steam Generator Level	M	R
6. Steam Generator Pressure	M	R#
7. Decay Heat Removal Temperature	M	R
8. Motor Driven Emergency Feedwater Pressure	M	R
9. Nuclear Services Closed Cycle Cooling Pumps Discharge Pressure	M	R
10. Nuclear Services Closed Cycle Cooling Cooler Outlet Temperature	M	R

#The specified 18 month calibration frequency may be waived for Cycle V provided the surveillance is performed during Refuel V.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. DPR-72  
FLORIDA POWER CORPORATION, ET AL.  
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT  
DOCKET NO. 50-302

Introduction

By letter dated August 30, 1984, Florida Power Corporation (FPC, the licensee) requested an amendment to the Technical Specifications (TSs) to replace the old remote shutdown system with the modified system implemented in accordance with 10 CFR 50, Appendix R. Since the NRC staff has not yet developed criteria for operability and surveillance requirements for Appendix R-related remote shutdown equipment, the licensee requested by letter dated June 17, 1985, that the existing TSs be revised only to show the new location for instrumentation which is now in the new Remote Shutdown Panel. Incorporation of the new remote shutdown system into the TSs will be evaluated in a separate SE.

Evaluation

The proposed revision to TS Table 3.3-9 showing the relocation of instrumentation involved in remote shutdown capability to the new Remote Shutdown Panel is consistent with the licensee's submittals on remote shutdown capability previously accepted by the NRC staff by letter dated January 6, 1983. No other changes to the TSs are involved. Therefore, the staff finds the proposed TS change acceptable.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1)

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there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 3, 1985

Principal contributor: H. Silver