

JUL 03 1985

Docket No. 50-302

DISTRIBUTION

Mr. Walter S. Wilgus  
Vice President, Nuclear Operations  
Florida Power Corporation  
ATTN: Manager, Nuclear Licensing  
& Fuel Management  
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Docket File  
NRC PDR  
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ORB#4 Rdg  
HThompson  
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Gray File  
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WGammill  
BGrimes

Dear Mr. Wilgus:

The Commission has issued the enclosed Amendment No. 76 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 27, 1985, as supplemented June 25, 1985.

This amendment allows for the replacement of six manual valves with eight remote-manual valves to provide capability to install a hydrogen recombiner.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

*Original signed by*

Harley Silver, Project Manager  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

- 1. Amendment No.76 to DPR-72
- 2. Safety Evaluation

cc w/enclosures:  
See next page

*RJD*  
ORB#4:DL  
Ringram  
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*BM*  
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*JST*  
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JST  
6/27/85

*M. J. ...*  
OELD  
6/2/85

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Mr. W. S. Wilgus  
Florida Power Corporation

Crystal River Unit No. 3 Nuclear  
Generating Plant

cc:

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and General Counsel  
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Attorney General  
Department of Legal Affairs  
The Capitol  
Tallahassee, Florida 32304



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION  
CITY OF ALACHUA  
CITY OF BUSHNELL  
CITY OF GAINESVILLE  
CITY OF KISSIMMEE  
CITY OF LEESBURG  
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH  
CITY OF OCALA  
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO  
SEBRING UTILITIES COMMISSION  
SEMINOLE ELECTRIC COOPERATIVE, INC.  
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 76  
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated February 27, 1985, as supplemented June 25, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 736, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 3, 1985

ATTACHMENT TO LICENSE AMENDMENT NO.76

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page

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TABLE 3.6-1 (continued)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME</u> (seconds)
LRV-45	iso. LR test panel from RB	NA
LRV-44	"	NA
LRV-46	"	NA
LRV-87 & 88*	iso. H <sub>2</sub> recombiner ports from RB	NA
LRV-89 & 90*	"	NA
LRV-91 & 92*	"	NA
LRV-93 & 94*	"	NA
3. MSV-146 #	iso. misc. waste storage tank from RCSG-1B	NA
MSV-128 #	"	NA
4. NGV-62	iso. NG system from steam generators	NA
NGV-81 #	"	NA
NGV-82	iso. NG system from pwr.	NA
5. SAV-24	iso. SA from RB	NA
SAV-23 & 122	"	NA
6. SFV-18	iso SF system	NA
SFV-19	"	NA
SFV-119 #	iso. Fuel Transfer tubes from F.T. Canal	NA
SFV-120 #	"	NA
7. WSV-1	iso. containment monitoring system from RB	NA
WSV-2	"	NA
<b>D. PENETRATIONS REQUIRING TYPE B TESTS</b>		
Blind Flange 119	iso. RB	NA
Blind Flange 120	"	NA
Blind Flange 202	"	NA

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME</u> (seconds)
Blind Flange 348	iso. fuel transfer tube from Transfer Canal	NA
Blind Flange 436	"	NA
Equipment Hatch	iso. RB	NA
Personnel Hatch	iso. RB	NA

# Not subject to Type C Leakage Test

\* The provisions of Specification 3.0.4 are not applicable.

- (A) Isolates on Diverse Isolation Actuation Signal A
- (B) Isolates on Diverse Isolation Actuation Signal B
- (A/B) Isolates on Diverse Isolation Actuation Signal A or B



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 76 TO FACILITY OPERATING LICENSE NO. DPR-72  
FLORIDA POWER CORPORATION, ET AL.  
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT  
DOCKET NO. 50-302

1.0 Introduction

By letter dated February 27, 1985, as supplemented June 25, 1985, Florida Power Corporation (FPC, the licensee) made application to amend the Technical Specifications (TSs) for Crystal River Unit 3 (CR-3) to allow the replacement of six manual valves with eight remote-manual valves. The remote-manual valves are being added to provide the capability to install a hydrogen recombiner.

A modification to the current Leak Rate Test System at CR-3 is being performed to install external recombiner capability per 10 CFR 50.44. This modification includes two supply and return lines with two remote manual valves in each line outside the Reactor Building. In the event of a Loss of Coolant Accident, hydrogen recombiners will be obtained and connected to these lines to reduce hydrogen concentrations in the Reactor Building.

Per Table 5-4 of the Final Safety Analysis Report, MSV-146 and MSV-128 are secondary drains from the steam generators. MSV-128 was apparently overlooked when Table 3.6-1 was developed. This amendment would also correct that omission.

2.0 Discussion and Evaluation

The addition of the eight remote-manual valves to replace the six manual valves is an acceptable design improvement. In its letter of June 25, 1985, the licensee has confirmed that the replacement valves satisfy the design and performance requirements of an engineered safety feature both for hydrogen control and containment isolation. We therefore conclude the proposed TSs are acceptable.

3.0 Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure.

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The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### 4.0 Conclusion

We have concluded, based on the considerations discussed above, that:  
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and  
(2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 3, 1985

Principal contributor: J. Huang