

Mr. W. S. Wilgus
Florida Power Corporation

Crystal River Unit No. 3 Nuclear
Generating Plant

cc:

Mr. R. W. Neiser
Senior Vice President
and General Counsel
Florida Power Corporation
P. O. Box 14042
St. Petersburg, Florida 33733

State Planning and Development
Clearinghouse
Office of Planning and Budget
Executive Office of the Governor
The Capitol Building
Tallahassee, Florida 32301

Mr. P. McKee
Nuclear Plant Manager
Florida Power Corporation
P. O. Box 219
Crystal River, Florida 32629

Mr. F. Alex Griffin, Chairman
Board of County Commissioners
Citrus County
110 North Apopka Avenue
Inverness, Florida 32650

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

Resident Inspector
U.S. Nuclear Regulatory Commission
15760 West Powerline Street
Crystal River, Florida 32629

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Mr. Allan Schubert, Manager
Public Health Physicist
Department of Health and
Rehabilitative Services
1323 Winewood Blvd.
Tallahassee, Florida 32301

Administrator
Department of Environmental Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 93
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated August 14, 1986, as supplemented October 6, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8610290121 861021
PDR ADOCK 05000302
PDR

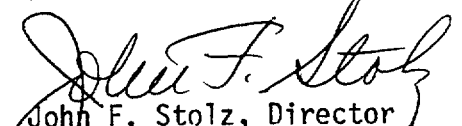
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 93, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Director
PWR Project Directorate #6
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 21, 1986

ATTACHMENT TO LICENSE AMENDMENT NO.93

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

3/4 5-5

Insert

3/4 5-5

EMERGENCY CORE COOLING SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

2. Verifying the correct position of each mechanical position stop for each of the stop check valves listed in Specification 4.5.2.c.
3. Verifying that the flow controllers for the throttle valves listed in Specification 4.5.2.d operate properly.
4. A visual inspection of the containment emergency sump which verifies that the subsystem suction inlets are not restricted by debris and that the sump components (trash racks, screens, etc.) show no evidence of structural distress or corrosion.
5. Verifying a total leak rate less than or equal to 6 gallons per hour for the LPI system at:
 - a) Normal operating pressure or a hydrostatic test pressure of greater than or equal to 150 psig for those parts of the system downstream of the pump suction isolation valve, and
 - b) Greater than or equal to 55 psig for the piping from the containment emergency sump isolation valve to the pump suction isolation valve.
- f. At least once per 18 months, in MODE 6, by
 1. Verifying that each automatic valve in the flow path actuates to its correct position on a high pressure or low pressure safety injection test signal, as appropriate.³
 2. Verifying that each HPI and LPI pump starts automatically upon receipt of a high pressure or low pressure safety injection test signal, as appropriate.³
- g. Following completion of HPI or LPI system modifications that could have altered system flow characteristics¹, by performance of a flow balance test during shutdown to confirm the following injection flow rates into the Reactor Coolant System:

HPI System - Single Pump²

Single pump flow rate greater than or equal to 500 gpm at 600 psig.

While injecting through 4 Injection Legs, the flow rate for all combinations of 3 Injection Legs greater than or equal to 350 gpm at 600 psig.

LPI System - Single Pump

1. Injection Leg A - 2800 to 3100 gpm.
2. Injection Leg B - 2800 to 3100 gpm.

¹ Flow balance tests performed prior to complete installation of modifications are valid if performed with the system change that could alter flow characteristics in effect.

² The HPI Flow Balance Test shall be performed in MODE 3.

³ For Cycle VI, the surveillance frequency shall be at least once per fuel cycle in MODE 6.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 93 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET AL.
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

INTRODUCTION

By letter dated August 14, 1986, as supplemented October 6, 1986, Florida Power Corporation (FPC or the licensee) requested amendment to the Technical Specifications (TSs) appended to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). The proposed amendment would revise TS 4.5.2.f to extend the surveillance interval for the high pressure injection (HPI) and low pressure injection (LPI) pump and valve actuation testing to once per fuel cycle for Cycle 6 only.

The existing specified interval of once per 18 months would require the surveillance be performed prior to the end of the current fuel cycle due to an extended forced outage to repair the reactor coolant pumps at CR-3. The surveillance could not be satisfied during the RCP outage because of the requirement to do the surveillance in Mode 6 when the reactor vessel head is removed. The head was not removed during the forced outage in order to avoid the unnecessary radiation exposure related to head removal.

The amendment would allow a one-time extension of the surveillance interval to eliminate a mid-cycle shutdown to perform the surveillance during Mode 6. The HPI and LPI actuation testing would then be accomplished at the end of the fuel cycle (September 1987).

EVALUATION

The existing TS 4.5.2.f requires that the surveillance interval for HPI and LPI pump and valve actuation testing be at least once per 18 months during Mode 6. The intent of the 18-month surveillance interval was to perform the surveillance testing during each refueling outage. The requirement of the surveillance testing to be performed during Mode 6 when the reactor vessel head is off is to prevent the possible overpressurization of the system during low temperature conditions.

The licensee has actually performed the testing required by TS 4.5.2.f with successful results during startup from the forced RCP outage. The testing was conducted during Mode 3 when the reactor coolant was at a temperature at which low temperature overpressurization is not a concern. Therefore, the licensee has reasonably assured HPI and LPI pump and valve operability for the additional period of time.

8610290127 861021
PDR ADOCK 05000302
PDR

Finally, the proposed change would eliminate the need for an unnecessary removal of the reactor vessel head. This will result in the reduction of unnecessary personnel radiation exposure as well as a reduction in the probability of a reactor vessel head drop. Therefore, we find the proposed TS change acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 21, 1986

Principal Contributors:

C. Liang
B. R. Mozafari